

April 8, 1992

Docket Nos. 50-321
and 50-366

Mr. W. G. Hairston, III
Senior Vice President -
Nuclear Operations
Georgia Power Company
P. O. Box 1295
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING THE PRESSURE SENSOR
ACTUATION FOR THE MAIN STEAM SAFETY RELIEF VALVES - HATCH NUCLEAR
PLANT, UNITS 1 AND 2 (TACS M82709 AND M82723)

By letter dated January 21, 1992, you provided the design summary evaluation
for your planned addition of pressure sensor actuation for the main steam
safety relief valves. As agreed to during our meeting of July 23, 1991, the
NRC staff has reviewed your submittal and finds that responses to the enclosed
request for additional information are needed so that the staff may complete
its review.

We request that you provide the responses no later than 30 days of the date of
this letter in order to maintain the review schedule. This request was
discussed with your staff via a telephone conference on April 2, 1992.

The reporting and/or recordkeeping requirements contained in this letter
affect fewer than ten respondents; therefore, OMB clearance is not required
under P.L. 96-511.

Please call me if you have any questions.

Sincerely,

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Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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Georgia Power Company

Edwin T. Hatch Nuclear Plant

cc:

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Chairman
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Baxley, Georgia 31513

ENCLOSURE

REQUEST FOR ADDITIONAL INFORMATION
PRESSURE SENSOR ACTUATION FOR
MAIN STEAM SAFETY RELIEF VALVES
HATCH NUCLEAR PLANT, UNITS 1 AND 2

1. Please clarify your position regarding the Technical Specifications for the pressure sensor actuation. The first sentence of the second paragraph of the cover letter states that: "As the new system is not safety related, it will not be included in the Technical Specifications." However, Item 11 of the Narrative Design Summary states that: "Calibration and maintenance requirements already established by Plant Hatch Technical Specifications for Nuclear Boiler System (B21) equipment will apply to this modification."
2. Please discuss the Emergency Safety System Division I to Division II interface criterion mentioned in Item 4 of the Narrative Design Summary. Also, justify the use of fuses as isolations between redundant divisions. Include in your justification a description of why such an interface is necessary and how the fuses were qualified to accomplish this task. Furthermore, discuss the periodic testing of these fuses to provide assurance that redundant Class 1E power sources will not be subjected to a single failure.