



**Consumers
Power
Company**

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March 15, 1984

Mr John J Harrison
Chief, Midland Project Section
U S Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

MIDLAND ENERGY CENTER GWO 7020
PROGRAM FOR RESUMPTION OF ASME III CODE AND NON-CODE
DIESEL GENERATOR MECHANICAL EQUIPMENT INSTALLATION
File: 0487.4 UFI: 00234(S) Serial: CSC-7468
M-18 42*10*06

The purpose of this letter is to request NRC concurrence with the program discussed at the meeting, March 7, 1984 and attached herein. In attendance at that meeting were the following persons:

<u>NAME</u>	<u>TITLE</u>
H. Livermore	NRC Site Supervisor
Jon S. Kreple	CPCo SMO
Bruce H. Peck	CPCo SMO-Construction
Morris Rubenstein	MPQAD QA Engineer
T. A. Parker	Stone & Webster Michigan, CIO
Bruce Harshe	CPCo-Project Engineering
J. Harrison	NRC, Section Chief
B. Burgess	NRC, Resident Inspector, Midland
P. Hiland	NRC, Resident Inspector, Midland

Consumers Power Company is proposing a program independent of the Construction Completion Program (CCP) for the resumption of ASME, Section III and non-code mechanical work on the portion of the Emergency Diesel Generators where the vendor, Trans-america DeLaval, Inc. (TDI), has the responsibility for the design and installation of the components.

It is our opinion that this proposed program is more stringent than the CCP and can therefore be independent from it.

The evolution of this program is based on the fact that TDI supplied a component in a system. The size of the component required that it be shipped in pieces and reassembled on the site. The reassembly was the responsibility of TDI with Bechtel crafts doing the work. Audits done to TDI QA Program revealed that their N stamp did not extend outside their fabrication facili-

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ties. Transferring of the N stamp responsibility to Bechtel was done by revisions to TDI procedures, generation of a new Code Effective Dates Specification, a SAR change clarifying the ASME criteria and an ASME audit for the purpose of adding authorization to fabricate and install ASME Section III NF supports. The ASME audit is scheduled for June 27-29, 1984. An NCR was written against the assembled component. The NCR states that the violation of ASME Section III and the lack of ANI involvement in the reassembly process requires that all ASME Code pieces be disassembled, inspected to the ASME Code requirements and reassembled. Review of component piece documentation will be done back to its receipt inspection. It is our opinion that this program is more stringent than the CCP and therefore can be used in lieu of it.

Specific elements of the program to do TDI ASME code work include:

- °All mechanical work to be done under the direction of TDI technical representative to TDI procedures.

- °Quality control inspections are to be done by MPQAD QCE's to PQCI's developed specifically for the Diesel Generators. The PQCI's are listed to describe their function.

- P2.11 Pipe Component Support Installation
- PF1.11 Pipe Flange Installation and Rework for Diesel Generator Piping Systems
- P1.01 Piping Completed Line Inspections Emergency Diesel Generators
- P11.41 Installation of ASME III Diesel Generator Instrumentation and Tubing Systems
- PW1.01 Fabrication, Welding, Heat Treating and Non Destructive Examination of ASME III Piping of the Emergency Diesel Generator Complex.
- T1.01 Pressure Testing of ASME III Piping and Instrumentation System Designed by Transamerica DeLaval, Inc.

- °The Bechtel Authorized Nuclear Inspector has reviewed and approved the PQCI's and will be responsible for inspection overview.

- °MPQAD has specifically designated Quality Control Inspectors to work on the Diesel Generators.

- °MPQAD will conduct a complete review of all open and closed PQCI Inspection Reports.

Specific elements of the non code related mechanical work include:

- °Technical direction by TDI

*Quality Control Inspections will be done by TDI Quality Control Inspector to TDI Quality Control Program.

*Inspection Reports will be developed by TDI Quality Control Inspector in accordance with their program.

*MPQAD will provide in line review function of the Inspection Reports and will monitor the work on a regular basis.

Bechtel has committed system team 19 to work exclusively in the Diesel Generator Building. The physical work will be done by Bechtel craft utilizing the construction work package program that was developed as part of the CCP.

Stone and Webster has committed to provide third party overinspection for this program.

This proposed program is independent of Consumers Power Company involvement in the TDI Users Group Task Force to solve the generic problems that have arisen on TDI diesel engines at other nuclear facilities. It is our desire to have as much of the construction complete as possible prior to addressing the modifications that come out of the Users Group Reviews.



DLQ/JSK/klw

CC BLHarshe, P24-618
RAWells, MPQAD
RJCook, Midland Resident Inspector
DSHood, USNRC
JGKepler, Regional Administrator, Region III

BCC JWCook, P-26-336B
SHHowell, M-1180B
TABuczynski, Midland-207
LGraber, LIS
JNLeech, P-24-506
DFLewis, Bechtel
FJLevandoski, B&W
GALow, P-12-237A
DASommers, P-14-106
P⁷Steptoe, IL&B, Chicago
DJVandeWalle, P-24-614B
BJWalraven, P-24-517
RAWells, Midland
FCWilliams, IL&B, Washington, DC
DTPerry, Midland
NRC Correspondence File, P-24-517
UFI, P-24-511
CMS-Midland

RC DMBudzik, P-24-517A
RJEhardt, P-14-113A
LSGibson, P-24-618A
P-24-505 (Last)



**Consumers
Power
Company**

James W. Cook
Vice President - Projects, Engineering
and Construction

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February 28, 1984

Mr J G Keppler, Regional Administrator
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PRINCIPAL STAFF	
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File Das

MIDLAND NUCLEAR COGENERATION PLANT -
DOCKET NOS. 50-329 AND 50-330
ITEMS FOR NRC CLOSURE
FILE 0.4.2 SERIAL 27328

In accordance with your request, we are submitting the attached list of items which have been reviewed by CP Co and have been determined to be ready for review and closure by the NRC.

This is a complete list which includes the items that were transmitted on October 4, 1983 that have not been closed subsequent to that time. The items marked with an asterisk (*) have been reviewed by the Region III Inspectors and will be removed from the list when closed in an inspection report.

James W. Cook

CC WRBird, P-14-418A
DMBudzik, P-24-517A
RCook, NRC-Midland
JFFremeau, Midland
JHarrison, NRC
JNLeach, P-24-507
DLQuamme, Midland
JARutgers, Bechtel-AA
RAWells, Midland
REWhitaker, Midland

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NRC Inspection Report Items

Miscellaneous Items Ready For Closure

<u>Item</u>	<u>Type</u>	<u>Subject</u>	<u>Closure Location</u>
79-12-05	Unresolved	Pressure Test of Pressurizer Relief Valve	Site
80-09-01	Deficiency	Leveling of Internal Core Support Structure	Site
*80-30/31-01	Unresolved	Battery Rack Acidic Environment	Site
81-12-09/10	Violation	Lack of Approved Procedures Covering Rework	Site
81-20-01	Unresolved	Cable Tray Dividers	Site
82-05-01	Deviation	QA Staffing	Site
82-06-02	Violation	Cable Pulling	Site
82-18-02/01	Violation	Dewatering Fines Monitoring	Site
82-18-03/02	Violation	Slope Layback	Site
82-20-01	Open	Training of RMS Personnel for Emergency Procedures and Training of Crafts	Site
83-03-02	Unresolved	Expansion of Excavation Permit System to Underpinning	Site
83-13/14-03	Deviation	Slope Layback in the Drifts	Site

NRC Inspection Report Items

Items Dealing With Hanger Design

The following items all were initiated by I Yen, and need to be resolved by a scheduled trip to Ann Arbor with a possibility of going to ITT Grinnell in Providence.

<u>Item</u>	<u>Type</u>	<u>Subject</u>	<u>Closure Location</u>
78-19-02	URI	Locking Devices on Bolts used in ITT Grinnell	Ann Arbor
78-19-03	URI	ITT Grinnell Evaluation of Bolt Holes Near Edge of Plate	Ann Arbor
79-01-02	URI	Adequacy of Drawing Hanger Review	Ann Arbor
79-05-02	URI	Additional reviews of ITT Hanger Design	Ann Arbor or Providence
79-05-03	URI	Bechtel System for Design Interface of Hanger Loadings on Existing Structures	Ann Arbor
80-11/12-01	URI	Bolted Joints	Ann Arbor
82-07-01	URI	Review of Other Disciplines for Handling of CPDC's	Ann Arbor

50.55(e) Reports Ready For Closure

<u>NRC Number</u>	<u>No</u>	<u>Description</u>
77-03	0.4.9.12	ITT/Grinnel Pipe Supports
78-01	0.4.9.13	RCP Motor Flange
78-06	0.4.9.18	Small Break Analysis
81-05	0.4.9.53	Shear Reinforcement at Major Containment Penetrations

Bulletins - Circulars - Notices Ready For Closure

*BULLETIN	73-01	Faulty Overcurrent Trip Delay Device in Circuit Breakers for Engineered Safety Systems
*BULLETIN	74-01	Valve Deficiencies
*BULLETIN	74-06	Defective Westinghouse Type W-2 Control Switch Component
*BULLETIN	74-08	Deficiency in ITE Molded Case Circuit Breakers, Type HE-3
*BULLETIN	74-09	Deficiency in General Electric Model 4kV Magna-Blast Breakers
*BULLETIN	74-12	Incorrect Coils in Westinghouse Type SG Relays at Trojan
*BULLETIN	74-15	Misapplication of Cutler-Hammer Three Positioned Maintained Switch, Model 10250T
BULLETIN	74-16	Improper Machining of Pistons in Colt Industries (Fairbanks-Morse) Diesel Generators
BULLETIN	78-12	Atypical Weld Material in Reactor Pressure Vessel Welds
BULLETIN	80-16	Potential Misapplication of Rosemount, Inc Models 1151 and 1152 Pressure Transmitters with either "A" or "D" Output Codes.
BULLETIN	82-01	Alteration of Radiographs of Welds in Piping Assemblies.
BULLETIN	82-03	Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants.
BULLETIN	82-04	Deficiencies in Primary Containment Penetration Assemblies.
BULLETIN	83-02	Stress Corrosion Cracking in Large Diameter Stainless Steel Recirculation System Piping at BWR Plants.
CIRCULAR	76-01	Crane Hoist Control Circuit Modifications
CIRCULAR	76-05	Hydraulic Shock and Sway Suppressors Maintenance of Bleed and Locking on ITT Grinnell's Model Number Figure 200 and 201 Catalog PH-74-R

*CIRCULAR	77-02	Potential Heavy Spring Flooding
CIRCULAR	77-06	Effects of Hydraulic Fluid on Electrical Cables
CIRCULAR	78-05	Inadvertent Safety Injection during Cooldown
*CIRCULAR	78-13	Inoperability of Multiple Service Water Pumps
CIRCULAR	78-15	Check Valves Fail to Close in Vertical Position
CIRCULAR	79-05	Moisture Leakage in Stranded Wire Conductors
CIRCULAR	79-11	Design/Construction Interface
CIRCULAR	79-12	Potential Diesel Generator Turbocharger Problem
CIRCULAR	80-16	Operational Deficiency on Rosemount Model 510DU Trip Units and Model 1152 pressure Transmitters
CIRCULAR	80-17	Fuel Pin Damage to Water Jet from Baffle Plate Corner
CIRCULAR	80-21	Regulation of Refueling Crews
CIRCULAR	81-05	Self-Aligning Rod End Bushings for Pipe Supports
CIRCULAR	81-06	Potential Deficiency Affecting Certain Foxboro 10 to 50 Milliampere Transmitters
CIRCULAR	81-07	Control of Radioactively Contaminated Material
CIRCULAR	81-08	Foundation Materials
CIRCULAR	81-09	Containment Effluent Water Heat Bypasses Radioactivity Monitor
CIRCULAR	81-11	Inadequate Decay Heat Removal During Reactor Shutdown

CIRCULAR	81-12	Inadequate Periodic Test Procedure of PWR Protection System
CIRCULAR	81-14	Main Steam Isolation Valve Failures to Close
NOTICE	79-06	Stress Analysis of Safety-Related Piping
NOTICE	79-09	Spill of Radioactively Contaminated Resin
NOTICE	79-11	Lower Reactor Vessel Head Insulation Support Problem
NOTICE	79-12	Attempted Damage to New Fuel Assemblies
NOTICE	79-13	Indication of Low Water Level in the Oyster Creek Reactor (BWR)
NOTICE	79-16	Nuclear Incident at Three Mile Island
NOTICE	79-17	Source Holder Assembly Damage from Misfit between Assembly and Reactor Upper Grid Plate (Westinghouse Units only)
NOTICE	79-18	Skylab Reentry
NOTICE	79-28	Overloading of Structural Elements due to Pipe Support Loads
NOTICE	79-29	Loss of Nonsafety-Related Reactor Coolant System Instrumentation during Operation.
NOTICE	79-32	Separation of Electrical Cables for HPCI and ADS (BWR only)
NOTICE	79-37	Cracking in Low Pressure Turbine Discs
NOTICE	80-04	BWR Fuel Exposure in Excess of Limits
NOTICE	80-06	Notification of Significant Event and Supplement

NOTICE	80-17	Potential Hazards Associated with Interchangeable Parts on Radiographic Equipment
NOTICE	80-20	Loss of Decay Heat Removal Capability at Davis-Besse Unit I while in Refueling Mode
NOTICE	80-22	Breakdowns in Contamination Control Programs
NOTICE	80-23	Loss of Suction to Emergency Feedwater Pumps
NOTICE	80-24	Low Level Radioactive Waste Burial Criteria
NOTICE	80-25	Transportation of Pyrophoric Uranium
NOTICE	80-27	Degradation of Reactor Coolant Pump Studs
NOTICE	80-29	Broken Studs on Terry Turbine Steam Inlet Flange
NOTICE	80-30	Potential for Unacceptable Interaction between the Control Rod Drive Scram Function and Nonessential Control Air at Certain G E BWR Facilities
NOTICE	80-33	Determination of Teletherapy Timer Accuracy
NOTICE	80-35	Leaking and dislodged Iodine-125 Implant Seeds
NOTICE	80-37	Containment Cooler Leaks and Reactor Cavity Flooding at Indian Point 2
NOTICE	80-45	Potential Failure of BWR Backup Manual Scram Capability
NOTICE	81-02	Transportation of Radiography Devices
NOTICE	81-04	Cracking in Main Steam Lines
NOTICE	81-06	Failure of ITE Model K-600 Circuit Breaker

NOTICE	81-07	Potential Problem with Water Soluble Purge Dam Materials used during Inert Gas Welding
NOTICE	81-08	Repetitive Failures of Limitorque Operator SMB-4 Motor-to-Shaft Key
NOTICE	81-10	Inadvertent Containment Spray due to Personnel Error
NOTICE	81-11	Alternate Rod Insertion for BWR Scram Represents a Potential Path for Loss of Primary Coolant.
NOTICE	81-12	Guidance on Order Issued January 9, 1981, Regarding Automatic Control Rod Insertion on Low Control Air Pressure (BWR only).
NOTICE	81-13	Jammed Source Rack in a Gamma Irradiator
NOTICE	81-14	Potential Overstress of Shafts on Fisher Series 9200 Butterfly Valves with Expandable T-Rings
NOTICE	81-15	Degradation of Automatic ECCS Actuation Capability by Isolation of Instrument Lines.
NOTICE	81-16	Control Rod Drive System Malfunction (BWR only)
NOTICE	81-17	Never Issued
NOTICE	81-18	Excessive Radiation Exposure to Fingers
NOTICE	81-19	Lost Parts in Primary Coolant System
NOTICE	81-20	Test Failures of Electrical Penetration Assemblies
NOTICE	81-21	Potential Loss of Direct Access to Ultimate Heat Sink
NOTICE	81-22	Section 235 and 236 Amendments to the Atomic Energy Act of 1954

NOTICE	81-23	Fuel Assembly Damage due to Improper Positioning of Fuel Handling Equipment
NOTICE	81-24	AFW Pump Turbine Bearing Failures
NOTICE	81-26	Compilation of Health Physics Related Information Items
NOTICE	81-27	Flammable Gas Mixtures in the Waste Decay Tanks in BWR Plants.
NOTICE	81-28	Failure of Rockwell-Elward Main Steam Isolation Valves
NOTICE	81-29	Equipment Qualification Testing Experience
NOTICE	81-30	Velan Swing Check Valves
NOTICE	81-31	Failure of Safety Injection Valve to Operate Against Differential Pressure.
NOTICE	81-32	Transfer and/or Disposal of Spent Generators
NOTICE	81-33	Locking Devices Inadequately Installed on Main Steam Isolation Valves
NOTICE	81-34	Accidental Actuation of Prompt Public Notification System
NOTICE	81-35	Check Valve Failures
NOTICE	81-36	Replacement Diaphragms for Robertshaw Valve (Model No VC-210)
NOTICE	81-37	Unnecessary Radiation Exposure to Public and Workers during Events Involving Thickness and Limit Measuring Devices.
NOTICE	81-38	Potentially Significant Equipment Failures Resulting from Contamination of Air-Operated Systems
NOTICE	81-39	EPA Crosscheck Program Low-Level Radiiodine in Water Intercomparison Study

NOTICE	82-01	AFW Pump Lockout Resulting from Westinghouse W-2 Switch Circuit Modification
NOTICE	82-02	Westinghouse NBFDR Relay Failures in Reactor Protection Systems at Certain Nuclear Power Plants
NOTICE	82-03	Environmental Tests of Electrical Terminal Blocks
NOTICE	82-04	Potential Deficiency of Certain Agastat E-7000 Series Time Delay Relays
NOTICE	82-05	Increasing Frequency of Drug-Related Incidents
NOTICE	82-07	Inadequate Security Screening Programs
NOTICE	82-09	Cracking in Makeup Coolant Lines at B&W Plants
NOTICE	82-10	Following Up Symptomatic Repairs to Assure Resolution of the Problem
NOTICE	82-11	Potential Inaccuracies in Wide Range Pressure Instruments Used in Westinghouse Designed Plants
NOTICE	82-12	Surveillance of Hydraulic Snubbers
NOTICE	82-13	Failure of General Electric Type HFA Relays
NOTICE	82-15	Notification of Nuclear Regulatory Commission (NRC)
NOTICE	82-16	HPCI/RCIC High Steam Flow Set Points
NOTICE	82-18	Assessment of Intakes of Radioactive Materials by Workers
NOTICE	82-19	Loss of High Head Safety Injection Emergency Boration and Reactor Coolant Makeup Capability
NOTICE	82-20	Check Valve Problems

NOTICE	82-21	Buildup of Enriched Uranium in Effluent Treatment Tanks
NOTICE	82-22	Failures in Turbine Exhaust Lines
NOTICE	82-23	Main Steam Isolation Valve Leakage
NOTICE	82-24	Water Leakage from Uranium Hexafluoride Overpacks
NOTICE	82-26	RCIC and HPCI Turbine Exhaust Check Valve Failure (BWR only)
NOTICE	82-27	Fuel Rod Degradation Resulting from Water-Jet Baffle Impingement
NOTICE	82-28	Hydrogen Explosion while Grinding in Vicinity of Drained and Open RCS
NOTICE	82-29	Control Rod Drive (CRD) Guide Tube Support Pin Failures at Westinghouse PWR (Westinghouse Plants only)
NOTICE	82-30	Loss of Thermal Sleeves in RCS Piping at Certain Westinghouse PWR Plants
NOTICE	82-31	Overexposure of Diver during Work in Fuel Storage Pool
NOTICE	82-33	Control of Radiation Levels in Unrestricted Areas Adjacent to Brachytherapy Patients (Medical Institutions only)
NOTICE	82-35	Failures of Three Check Valves on HPI Lines to Pass Flow
NOTICE	82-36	Respirator Users Warning for Certain 5-Minute Emergency Escape Self-Contained Breathing Apparatus
NOTICE	82-37	Cracking in the Upper Shell to Transition Cone Girth Weld of a Steam Generator on an Operating PWR
NOTICE	82-38	Change in Format and Distribution System for IE Bulletins, Circulars and Information Notices

NOTICE	82-39	Service Degradation of Thick Wall Stainless Steel Recirculation System Piping at BWRs
NOTICE	82-40	Deficiencies in Primary Containment Electrical Penetration Assemblies
NOTICE	82-41	Failure of Safety Relief Valve to Open at a BWR
NOTICE	82-42	Defects Observed in Panasonic Model 801 and Model 802 Thermoluminescent Dosimeters
NOTICE	82-43	Deficiencies in LWR Air Filtration/Ventilation Systems
NOTICE	82-44	Clarification of Emergency Plan Exercise Requirements
NOTICE	82-45	PWR Low Temperature Overpressure Protection
NOTICE	82-46	Defective and Obsolete Combination Padlocks
NOTICE	82-51	Overexposure in PWR Cavities
NOTICE	82-52	Equipment Environmental Qualification Testing Experience - Update of Test Summaries Previously Issued in IN 81-29
NOTICE	82-54	Westinghouse NBFD Relay Failures in Reactor Protection Systems
NOTICE	82-55	Seismic Qualification of Westinghouse AR Relay with Latch Attachment used in Westinghouse Solid-State Protection System.
NOTICE	83-01	Ray Miller, Inc.
NOTICE	83-07	Nonconformities with Materials Supplied by Tube-Line Corp
NOTICE	83-09	Safety and Security of Irradiators
NOTICE	83-12	Incorrect Boron Standards

NOTICE	83-13	Design Misapplication of Bergen Patterson Standard Strut Restraints
NOTICE	83-14	Dewatered Spent Ion Exchange Resin Susceptability to Exothermic Chemical Reaction
NOTICE	83-15	Falsified Pre-Employment Screening Records
NOTICE	83-16	Contamination of Auburn Steel Company with Cobalt-60
NOTICE	83-20	ITT Grinnel Figure 306/307 Mechanical Snubber Attachment Interferences
NOTICE	83-21	Defective Emergency Use Respirators
NOTICE	83-22	BWR Safety/Relief Valve Failures
NOTICE	83-26	Failure of Safety/Relief Valve Discharge Line Vacuum Breakers
NOTICE	83-29	Fuel Binding Caused by Fuel Rack Deformation
NOTICE	83-32	Rupture of Americium-241 Source(s) Contained in Well Logging Device
NOTICE	83-34	Event Notification Information Worksheet
NOTICE	83-35	Fuel Movement with Control Rods Withdrawn
NOTICE	83-37	Transformer Failure Resulting from Degraded Internal Connection Cables
NOTICE	83-39	Failure of Safety/Relief Valves to Open at a BWR - Interim Report
NOTICE	83-42	Reactor Mode Switch Malfunction
NOTICE	83-44	Potential Damage to Redundant Safety Equipment as a Result of Backflow through the Equipment and Floor Drain System
NOTICE	83-52	Radioactive Waste Gas System Events

NOTICE	83-53	Primary Containment Isolation Valve Discrepancies
NOTICE	83-54	Common Mode Failure of Main Steam Isolation Nonreturn Check Valves
NOTICE	83-55	Misapplication of Valves by Throttling beyond Design Range
NOTICE	83-59	Dose Assignment for Workers in Non-Uniform Radiation Fields
NOTICE	83-60	Falsification of Test Results for Protection Coatings
NOTICE	83-61	Alleged Use of Stand-Ins for Welder Qualification Tests
NOTICE	83-62	Failure of Redundant Toxic Gas Detectors Positioned at Control Room Ventilation Air Intakes
NOTICE	83-63	Potential Failure of Westinghouse Electric Corp Type SA-1 Differential Relays
NOTICE	83-64	Lead Shielding Attached to Safety-Related Systems without 10CFR50.59 Evaluations
NOTICE	83-66	Fatality at Argentine Critical Facility
NOTICE	83-67	Emergency Use Respirator Material Defect Causes Production of Noxious Gases
NOTICE	83-68	Respirator User Warning - Defective Self-Contained Breathing Apparatus Air Cylinders
NOTICE	83-73	Radiation Exposure from Gloves Contaminated with Uranium Daughter Products.
NOTICE	83-74	Rupture of Cesium-137 Source used in Well-Logging Operations
NOTICE	83-80	Use of Specialized "Stiff" Pipe Clamps
NOTICE	83-82	Failure of Safety Relief Valves to Open at BWR - Formal Report

NOTICE 83-84 Cracked and Broken Piston Rods in Brown Boveri Electric Type 5HK Circuit Breakers

THE LINES

April 9, 1984

For the past several weeks we have seen a large amount of speculation and suggestions from the media and various government officials and other organizations concerning the future of the Midland Nuclear Plant. Because of the variety of organizations involved in one way or another in issuing public statements about Midland, there has been some confusion.

The Company has been meeting for several weeks with representatives of the attorney general and the staff of the Michigan Public Service Commission. These meetings, which have involved discussing various scenarios relating to completion of the Midland Plant, resulted from an agreement between Consumers Power Chairman John D Selby and Attorney General Frank Kelley. No other parties have been involved in these talks and no firm proposals have been made by either side.

Company officials met last week with representatives of large industrial customers to brief them on our forecasts which show the need to complete Midland. Although participants in that meeting included members of the industrial customer organization called ABATE, the session also included major customers who do not hold membership in that organization.

The only concrete proposal offered to the Company by any outside party was a so-called "survival plan" which was proposed by a coalition that included some of the above-named parties along with the Michigan Citizens' Lobby. That proposal, which was studied, was rejected in its entirety because it was designed as a blueprint for disaster for the Company.

For some months, the Company has been conducting a study of a new schedule for completion of Unit 2. The results of that study will include an initial cost estimate for completing the unit. The Company's board of directors will be presented with the results of that study prior to the Company's annual meeting on Tuesday, April 10. The results of any board action are scheduled to be announced at the annual meeting. We plan to communicate that information to all Midland workers as quickly as possible.

The major issues to be reviewed by the board include:

- *Schedule and cost for the Midland Nuclear Plant
- *Action on the payment of the Company's common stock dividend

During the meeting itself, stockholders will vote on a proposal submitted by an individual shareholder that would halt construction of the plant for one year.

Time limitations Tuesday afternoon will now allow us to give you an immediate detailed analysis of the decisions and their impact on Midland. We will provide this full accounting to you as soon afterwards as possible.

Amid the pages of newspaper publicity and hours of radio and television coverage of the Midland Nuclear Plant, we should not lose perspective of our individual jobs and roles to successfully complete this project. The site-wide role and responsibility is clear: we need to continue with the progress we are accomplishing in completing our task.

The Midland completion schedule is achievable. It will be accomplished not by the board of directors or government officials in Lansing; it must be made by all workers wanting to share in the success of Midland.



**Consumers
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TELETYPE TRANSMISSION

COLLECTED

'84 MAR -9 P4:42

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March 7, 1984

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Bethesda, MD 20014

Dear Board Members:

At Tr. 22401 Robert Wheeler offered to provide information concerning certain work which was performed in connection with an underground fireline at the Midland site. Mr. Wheeler has been able to determine that after NRC Staff approval was obtained, physical connections were made to the fireline in question on or about October 16, 1982 to restore fire service to portions of the jobsite. The physical connections were a mechanical operation not involving excavations or backfilling. The entire fireline excavation was backfilled at a later date, pursuant to NRC authorization under the Region III work authorization procedure.

Very truly yours,

James E. Brunner

James E. Brunner
Attorney for Consumers Power Company

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DS03

March 21 1984

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

Warrick

APR 9 1984

MEMORANDUM FOR: D. G. Eisenhut, Director, Division of Licensing

FROM: C. E. Norelius, Director, Division of Project and Resident Programs

SUBJECT: RECOMMENDATION FOR NOTIFICATION OF LICENSING BOARD - FCR/FCN STOP WORK STATUS AND NRC MIDLAND STAFFING CHANGES.

In accordance with present NRC procedures regarding Board notifications, the following information is being provided as constituting new information relevant and material to the Midland OM/OL proceedings.

Design Control Stop Work Status

On January 31, 1984, Region III notified you of the status of the stop work orders regarding the design control (FCR/FCN) problems at Midland. On February 14, 1984, a Supplemental Board Notification (BN 84-023) forwarded this information to the Hearing Board. Following this notification the licensee lifted the remaining stop-works by area engineering discipline. On March 23, 1984, at 7:30 a.m., the final stop work was lifted. Attached you will find a copy of a matrix developed by Consumers Power Company that identifies key dates for various releases including Stone and Webster releases and CP&Co final release.

As a result of the design control stop work orders and the resultant review of approximately 60,000 documents by Consumers Power Company, 12 nonconformance reports were issued. Attachment 2 identifies the nonconformance reports issued by number, engineering discipline, and a description of the nonconformance. The licensee has determined that the nonconformances have no significant impact on hardware.

The NRC staff plans to perform an inspection on the FCR/FCN area later this year to review the new design control system, problems identified by the licensee, and corrective actions taken. The Board will be informed of the results of this inspection via a copy of the inspection report when issued. This will complete all required followup action by the NRC.

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NRC Staffing Status

The NRC staff assigned to perform inspections at the Midland facility has been increased this fiscal year. The increase is in accordance with the Region III staffing plan and is due to the backlog of open inspection items and the increased inspection workload associated with the Construction Completion Program.

The NRC now has three resident inspectors located at the Midland site, an additional three individuals in the Region III office assigned full time to the Midland project, and three inspection specialists from the Region III Division of Engineering who spend time onsite as needed. Additionally, an NRC Resident Site Supervisor has been selected for Midland and will report to the site in the near future.

NRC is in the process of approving a contract with a national laboratory for ~~assistance with the technical inspection~~ program at Midland, approximately two and one-half man-years of effort. In the interim, two Argonne contract engineers, formerly assigned to Zimmer, have been temporarily assigned to provide inspection assistance at Midland.

With the announced decision by the owners of the Zimmer plant to not complete it as a nuclear facility, Region III dissolved the Zimmer Section and the Office of Special Cases and transferred the Midland Section together with all its people to the Division of Project and Resident Programs (DPRP). The Director of the Office of Special Cases was reassigned as Chief, Projects Branch 1, DPRP, with responsibility for RIII plants under construction, including Midland.

We believe the increased NRC staffing will help to reduce the backlog of open inspection items and will enable RIII to better monitor the ongoing plant activities. The organization changes should have no impact on NRC inspection activities at Midland. No followup actions are required.

A. Bert Davis
for C. E. Norelius, Director
Division of Project and
Resident Programs

Enclosures: As stated

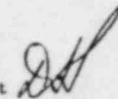
cc w/encls:
J. G. Keppler

STOP WORK RELEASE SUMMARY
STOP WORK ORDERS FSW-33 THROUGH 41

Attachment 1

STOP WORK NO.	FSW-33	FSW-34	FSW-35	FSW-36	FSW-37	FSW-38	FSW-39	FSW-40	FSW-41
SUBJECT	HVAC Zack & MPQAD-HVAC Inspection	MPQAD Hanger Reinspection	GEO Construction Testing	Procurement	B&W Fabrication, Installation & Inspection (Utilizing BPCo Spec's & Dwg's.	Soils Q-Related Work Fabrication, Installation, construction & Inspection	Standish Fabrication Facility Fabrication Inspection	GSO Fabrication & Installation; MPQAD Inspection in support of GSO	Balance of Plant Q-related work including CCP Status Assessment & QVP Act.
PARTIAL RELEASE	N/A	N/A	N/A	Ann Arbor Procurements (Only) Verified as not affected 11-3-83 11 20 a.m.	N/A	N/A	N/A	N/A	N/A
PARTIAL RELEASE FOR FCN/FCR PROCESSING	12-9-83 6:10 p.m.	12-9-83 6:19 p.m.	12-9-83 6:20 p.m.	12-9-83 6:19 p.m.	12-9-83 6:20 p.m.	12-9-83 6:12 p.m.	12-9-83 6:21 p.m.	12-9-83 6:22 p.m.	12-9-83 6:21 p.m.
STONE & WEBSTER ARCHITECTURAL RELEASE	1-20-84	1-20-84	1-20-84	1-10-84	1-20-84	1-20-84	1-20-84	1-20-84	Dwg only at Station 14 12-11-83 (Verbal) Dwg & Specs. DCC Stations 14, 48a, b&d & 55 12-13-83 Print concept @ Station 59 1-20-84

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STOP WORK NO.	FSM-27	FSM-28	FSM-25	FSM-26	FSM-27	FSM-28	FSM-29	FSM-40	FSM-81
PARTIAL RELEASE FOR Q-RELATED WORK ARCHITECTURAL DOC'S	2-9-84 11:25 a.m. # Station 59 (FD0C)	1-20-84 11:17 a.m. # Station 59 (FD0C)	2-9-84 2:48 p.m. # Station 59 (FD0C)	1-20-84 11:15 a.m. # Station 59 (FD0C)	2-8-84 3:48 p.m. # Station 59 (FD0C)		1-20-84 11:15 a.m. # Station 59 (FD0C) NOTE: Fabrication, inspection, & related work in support of Soils re-leased via SMO FSM-38	1-20-84 11:18 a.m. # Station 59 (FD0C)	12-11-83 3:00 p.m. Dwg only # Station 14 12-13-83 9:15 a.m. Dwg & Spec's DCC Sta- tions 14, 40a, 44d & 55 1-20-84 11:15 a.m. All via work prin & Status Assess- ment Prin concept # Station 59 (FD0C)
STONE & WEBSTER CIVIL RELEASE	1-20-84	1-20-84	1-20-84	1-20-84	1-20-84	1-19-84	1-20-84	1-20-84	1-20-84
PARTIAL RELEASE FOR Q-RELATED WORK CIVIL DOC'S	2-9-84 11:25 a.m. # Station 59 (FD0C)	1-20-84 11:17 a.m. # Station 59 (FD0C)	2-9-84 2:48 p.m. # Station 59 (FD0C)	1-20-84 11:15 a.m. # Station 59 (FD0C)	2-8-84 3:48 p.m. # Station 59 (FD0C)	1-19-84 9:40 a.m. Soils DCC Stations 8, 10, 13, 24 & 64	1-20-84 11:15 a.m. # Station 59 (FD0C) NOTE: Fabrication, inspection, & related work in support of Soils re-leased via SMO FSM-38	1-20-84 11:18 a.m. # Station 59 (FD0C)	1-20-84 11:15 a.m.
STONE & WEBSTER I & C RELEASE	1-24-84	1-24-84	1-24-84	1-24-84	1-24-84	1-24-84	1-24-84	1-24-84	1-24-84

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STOP WORK NO	FSW-33	FSW-34	FSW-35	FSW-36	FSW-37	FSW-38	FSW-39	FSW-40	FSW-41
PARTIAL RELEASE FOR Q-RELATED WORK I&C DOC's*	2-9-84 11:25 a.m. @ Station 59 (FDDC)	1-24-84 12:32 p.m. @ Station 59 (FDDC)	2-9-84 2:48 p.m. @ Station 59 (FDDC)	1-24-84 12:30 p.m. @ Station 59 (FDDC)	2-8-84 3:48 p.m. @ Station 59 (FDDC)	(Soils G Series to support Civil Soils released 1-24-84 @ 11:55 a.m.)	1-24-84 12:32 p.m. @ Station 59 (FDDC)	1-24-84 12:33 p.m. @ Station 59 (FDDC)	1-24-84 12:35 a.m.
STONE & WEBSTER ELECTRICAL RELEASE	1-26-84	1-26-84	1-26-84	1-26-84	1-26-84	1-26-84	1-26-84	1-26-84	1-26-84
PARTIAL RELEASE FOR Q-RELATED WORK ELECTRICAL DOC's.	2-9-84 11:25 a.m. @ Station 59 (FDDC)	1-26-84 2:36 p.m. @ Station 59 (FDDC)	2-9-84 2:48 p.m. @ Station 59 (FDDC)	1-26-84 2:37 p.m. @ Station 59 (FDDC)	2-8-84 3:48 p.m. @ Station 59 (FDDC)	2-6-84 1:20 p.m. Stations 8 & 10	1-26-84 2:38 p.m. @ Station 59 (FDDC)	1-26-84 2:39 p.m. @ Station 59 (FDDC)	1-26-84 2:12 p.m. @ Station 59 (FDDC)
STONE & WEBSTER MECHANICAL RELEASE	2-8-84	2-8-84	2-8-84	2-8-84	2-8-84	2-8-84	2-8-84	2-8-84	2-8-84
PARTIAL RELEASE FOR Q-RELATED WORK MECHANICAL DOC's	2-9-84 11:25 a.m. @ Station 59 (FDDC)	2-8-84 3:47 p.m. @ Station 59 (FDDC)	2-9-84 2:48 p.m. @ Station 59 (FDDC)	2-8-84 3:47 p.m. @ Station 59 (FDDC)	2-8-84 3:48 p.m. @ Station 59 (FDDC)	3-23-84 7:30 a.m. @ Stations 8 & 10	2-8-84 3:48 p.m. @ Station 59 (FDDC)	2-8-84 3:49 p.m. @ Station 59 (FDDC)	2-8-84 3:45 p.m. @ Station 59 (FDDC)
FINAL COMPLETE RELEASE	2-9-84 11:25 a.m.	2-8-84 3:47 p.m.	2-9-84 2:48 p.m.	2-8-84 3:47 p.m.	2-8-84 3:48 p.m.	Soils Pri- mary re- lease 1-19-84 9:40 a.m.	2-8-84 3:48 p.m.	2-8-84 3:49 p.m.	2-8-84 3:45 p.m.

*NOTE: G Series documents were included @ I & C Doc's, G-Doc were released in conjunction as when the discipline was released starting 1-24-84.

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