



Commonwealth Edison
LaSalle County Nuclear Station
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April 10, 1992

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for March, 1992.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MJC/djf

Enclosure

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PIASALLE NUCLEAR POWER STATION
UNIT 1
MONTHLY PERFORMANCE REPORT
MARCH 1992
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. MPF-11

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(UNIT 1)

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I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1130 Mwe.
	1640	Reactor scram due to loss of Main Condenser vacuum.
4	0610	Reactor critical.
5	1455	Generator on-line.
6	0645	Increased power level to 470 Mwe.
	0700	Increased power level to 700 Mwe.
	1100	Increased power level to 850 Mwe.
7	0900	Increased power level to 930 Mwe.
8	1500	Increased power level to 1000 Mwe (level held due to feedwater controller problems).
	1930	Reduced power level to 800 Mwe to place the 'A' Turbine Driven Reactor Feed Pump on-line.
	2100	Increased power level to 1050 Mwe.
9	0000	Increased power level to 1115 Mwe.
	2300	Reduced power level to 1000 Mwe to perform Reactor Protection monthly surveillances and Control Rod Drive exercising.
10	0830	Increased power level to 1130 Mwe.
18	1940	Reduced power level to 1115 Mwe due to low Condensate Booster Pump suction pressure.
19	1100	Increased power level to 1130 Mwe.
20	1800	Reduced power level to 1100 Mwe to place the 'C' Condensate Pump on-line.
21	0500	Increased power level to 1130 Mwe.
23	2330	Reduced power level to 850 Mwe to perform a rod set and to perform a heater bay entry to isolate the '16A' Heater emergency drain valve due to a valve leak.

II. MONTHLY REPORT (CONTINUED)

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
24	1000	Increased power level to 1130 Mwe.
25	0130	Reduced power level to 1000 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
26	0200	Reduced power level to 850 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
31	2400	Reactor critical, Generator on-line at 1130 Mwe.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-002-00	03/11/92	Auto trip of the '1D' Heater Drain System pump causing a voltage spike that was transmitted to the site electrical distribution system.
92-003-00	03/01/92	Automatic reactor scram caused by the loss of Main Condenser vacuum.
92-004-00	03/10/92	A Fire Protection system surveillance was not completed in the required time.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L13637	1B Diesel Generator Fuel Filters	Pressure gauges broken.	Inproper indication.	Replaced gauges.
L13938	Hydraulic Control Unit 14-51	Directional control valve.	Multiple control rod notch outs.	Replaced valve.
L13957	Extraction Steam Instrument Root Valve IES065	Valve stem separated from disc.	Resulted in 1/2 main steam isolation.	Replaced stem and disc assembly.

(No SOR Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 85C 73
UNIT LASALLE ONE
DATE April 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (915) 357-6761

OPERATING STATUS

- | | |
|--|------------|
| 1. REPORTING PERIOD: | March 1992 |
| GROSS HOURS IN REPORTING PERIOD: | 744 |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): | 3,323 |
| MAX DEPENDABLE CAPACITY (Mwe-Net): | 1,036 |
| DESIGN ELECTRICAL RATING (Mwe-Net): | 1,078 |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY): | N/A |
| 4. REASON FOR RESTRICTION (IF ANY): | |

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
	-----	-----	-----
5. REACTOR CRITICAL TIME (HOURS)	682.5	2,102.5	49,479.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	649.8	2,089.8	48,468.1
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh _t)	2,066,741	6,739,752	141,896,731
10. ELECTRICAL ENERGY GENERATED (MWh _e -Gross)	702,763	2,301,840	47,391,615
11. ELECTRICAL ENERGY GENERATED (MWh _e -Net)	679,927	2,234,351	45,429,496
12. REACTOR SERVICE FACTOR (%)	91.7	97.2	68.4
13. REACTOR AVAILABILITY FACTOR (%)	91.7	97.2	70.7
14. UNIT SERVICE FACTOR (%)	87.3	95.7	67.0
15. UNIT AVAILABILITY FACTOR (%)	87.3	95.7	67.0
16. UNIT CAPACITY FACTOR (USING MDC) (%)	88.2	98.8	60.6
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	84.8	94.9	58.3
18. UNIT FORCED OUTAGE FACTOR (%)	8.6	2.9	7.4
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):	Refuel Outage, 09/26/92, 12 Weeks		
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE April 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815) 357-6761

REPORTING PERIOD: March 1992

DAY	POWER	DAY	POWER
---	-----	---	-----
1	754	17	1,101
2	-10	18	1,100
3	-11	19	1,095
4	-13	20	1,091
5	37	21	1,095
6	622	22	1,098
7	860	23	1,099
8	955	24	1,060
9	1,090	25	1,080
10	1,070	26	1,076
11	1,099	27	1,096
12	1,100	28	1,096
13	1,103	29	1,096
14	1,103	30	1,096
15	1,102	31	1,093
16	1,099		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
1	920301	F	94.25	A	3	Reactor scram due to loss of main condenser vacuum. LER #92-003-00 DVR #1-1-92-0014

SUMMARY OF OPERATION:

The unit started the month at full power operation. At approximately 1640 on 03/01/92 a reactor scram occurred due to loss of main condenser vacuum. The unit was returned to service on 03/05/92. The unit remained on line at high power for the remainder of the month. Several minor power reductions were required due to system loading and maintenance activities.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
03/01/92	2B21-F013J	Automatic	1	Scram(LER #92-003-00)

2. ECCS System Outages
(See table 5)

3. Changes to the Off-Site Dose Calculation Manual
(None.)

4. Major changes to Radioactive Waste Treatment Systems.
(None.)

5. Indications of Failed Fuel Elements.
(None.)

(Unit 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(J-0)		
0155	ODG01K	Lubrication.
0160 0161	ODG01K	Install interposing relay in ACB 1413.
(U-1)		
0164	1E51-C004 1E51-C005	Motor inspection.
0180	1E22-C001	Megger motor.
0183	1E22-S001	Lubrication.
0187	1E22-C001	Inspection and calibration.
0194	1DG01K	Lubrication.

LASALLE NUCLEAR POWER STATION
UNIT 2
MONTHLY PERFORMANCE REPORT
MARCH 1992
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPY-18

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(Unit 2)

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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number MPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor subcritical, Generator off-line, Refuel outage (L2R04) in progress.
31	2400	Reactor subcritical, Generator off-line, Refuel outage (L2R04) in progress.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
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92-003-00*	03/22/92	During performance of the Reactor Instrument Line High Pressure Excess Flow Check Valve Test an actuation of Reactor water level switches caused the '2B' HPCS Diesel Generator to auto start.
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92-005-00	03/22/92	Initiation of the '2B' and '2C' Low Pressure Core Injection system and a '2A' Diesel Generator auto start due to a instrument line pressure spike.
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* LER #92-003-00, Unit 2 output breaker (ACB 2413) failed to close, as reported in the February 1992 report was, upon further investigation, determined to be a non-reportable item. LER #92-003-00 was reissued and a summary is included in this report.

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 2)
2. Average Daily Unit Power Level.
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L03289	Automatic Depressurization System Bottle Pressure Regulator Stop Valve 2IN090	Valve packing.	None.	Replaced valve packing.
L04383	Low Power Range Monitor 56-25	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L04391	Suppression Pool Temperature Elements 2TE-OM057A-B 2TE-OM057E-A 2TE-OM057V-A	RTD input amplifier.	Inaccurate temperature indication.	Replace RTD input amplifier.
L05384	Reactor Core Isolation Valve packing leakage. Cooling Steam Supply Valve 2E51-F008	Valve packing leakage.	None.	Installed new spring packing.
L08060	Main Steam Line Drain Valve 2B21-F019	Torque switch.	Torque switch failed when valve was closed.	Replaced torque switch.
L08812	250 Volt Battery Cells #77 and 114	Cells #77 and 114 had low ICV.	None.	Replaced cells #77 & 114.
L09371	Low Power Range Monitor 48-41	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09372	Low Power Range Monitor 32-49	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09373	Low Power Range Monitor 32-09	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.

C. TABLE i (Unit 2) —CONTINUED

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L09374	Low Power Range Monitor 24-49	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09375	Low Power Range Monitor 24-09	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09376	Low Power Range Monitor 16-49	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09377	Low Power Range Monitor 16-41	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09376	Low Power Range Monitor 16-25	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09379	Low Power Range Monitor 16-17	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09380	Low Power Range Monitor 08-41	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09383	Low Power Range Monitor 32-57	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09385	Low Power Range Monitor 32-25	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09387	Low Power Range Monitor 16-33	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09386	Low Power Range Monitor 48-33	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.

C. TABLE 1 (Unit 2) —CONTINUED

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L09389	Low Power Range Monitor 48-25	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09390	Low Power Range Monitor 40-49	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09391	Low Power Range Monitor 40-09	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L09392	Low Power Range Monitor 40-33	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L11867	Low Power Range Monitor 08-33	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L12730	Primary Containment Ventilation Outboard Isolation Valve 2VP053A	Disc and seat leakage.	Valve failed local leak rate testing.	Lapped disc and seat and replaced packing.
L12781	Reactor Building Closed Cooling Water Drywell Isolation Valve 2MR179 and 2MR180	Valve control switch.	None.	Replaced control switch.
L12828	Drywell Equipment Drain Sump Isolation Valve 2RE024	Valve seat leakage.	Valve failed local leak rate testing.	Replaced seat ring and gasket.
L12954	Reactor Recirculation Pump Seal Cooling Check Valve 2B33-F017A	Valve seat leakage.	Valve failed local leak rate testing.	Lapped disc and seat.

C. TABLE 1 (Unit 2) --CONTINUED

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L13597	Reactor Vessel Low Pressure Injection Line Stop Valve 2B21-M413A	Valve stem bent.	Valve leakage.	Replaced valve stem.
L13669	Residual Heat Removal Heat Exchanger Isolation Valve 2E12-F011A	Valve packing leakage.	None.	Adjusted packing.
L13815	Hydraulic Control Unit 46-55	Nitrogen cylinder leakage.	None.	Replaced cylinder.
L13854	Hydraulic Control Unit 34-19	Scram pilot solenoid valve leaking air.	None.	Replaced cover gasket.
L13964 L13965	Residual Heat Removal Suction Cooling Isolation Valve 2E12-F009	Motor contactor.	Closed contactor experienced short circuit current when valve was cycled.	Replaced motor contactor.
L13969	Scram Air Header Pressure	Regulator valve 2IA186	Outlet header pressure not maintained during 1/2 scrams.	Replaced valve.
L14015	Low Power Range Monitor 16-25C	LPRM card.	LPRM indicating downscale.	Replaced LPRM card.
L14253 L14254	Reactor Core Isolation Cooling Steam Supply Inboard Isolation Valve 2E51-F063	Valve seat leakage.	Valve failed local leak rate testing.	Lapped disc and seats and installed a new seal ring.
L14275	Switchgear Heat Removal System Damper 1VX07Y	Damper actuator 2TZ-VX098B.	Damper failed to cycle.	Replaced damper actuator.

C. TABLE 1 (Unit 2) - CONTINUED

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L14302	Hydraulic Control Unit 46-15	Directional control valve 122.	Solenoid valve failed to operate.	Replaced solenoid valve coil.
L14445	Hydraulic Control Unit 46-47	Scram pilot solenoid valve leaking air.	None.	Rebuilt valve.
L14610	Core Standby Cooling Systems Ventilation Damper 2VY01Y	Damper actuator 2TZ-VY023A.	Damper actuator inoperable.	Replaced damper actuator.
L14611	Core Standby Cooling Systems Ventilation Damper 2VY02Y	Damper actuator 2TZ-VY023B.	Damper actuator inoperable.	Replaced damper actuator.
L97852	Low Power Range Monitor 08-49	LPRM string reached neutronic end of life.	LPRM string inoperable.	Replaced LPRM string.
L99035	Hydraulic Control Unit 34-43	Directional control valve 120.	None.	Replaced directional control valve.
L99622	Reactor Core Isolation Valve Cooling Valve Inboard Isolation Valve 2E51-F063	Valve packing.	Seal leak off temperature reached alarm point.	Replaced valve packing.

(No. 500 Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE April 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

- | | |
|--|------------|
| 1. REPORTING PERIOD: | March 1992 |
| GROSS HOURS IN REPORTING PERIOD: | 744 |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): | 3,323 |
| MAX DEPENDABLE CAPACITY (Mwe-Net): | 1,036 |
| DESIGN ELECTRICAL RATING (Mwe-Net): | 1,078 |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY): | N/A |
| 4. REASON FOR RESTRICTION (IF ANY): | |

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	0.0	84.0	45,019.8
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	0.0	73.9	44,278.5
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh _t)	0.0	180,986	132,423,385
10. ELECTRICAL ENERGY GENERATED (MWh _e -Gross)	0.0	54,274	43,954,024
11. ELECTRICAL ENERGY GENERATED (MWh _e -Net)	-8,759	37,720	42,169,224
12. REACTOR SERVICE FACTOR (%)	0.0	3.8	68.9
13. REACTOR AVAILABILITY FACTOR (%)	0.0	3.8	71.5
14. UNIT SERVICE FACTOR (%)	0.0	3.4	67.8
15. UNIT AVAILABILITY FACTOR (%)	0.0	3.4	67.8
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-1.1	1.7	62.3
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	-1.1	1.6	59.0
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	13.1
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			04/09/92

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE April 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815) 357-6761

REPORTING PERIOD: March 1992

DAY	POWER	DAY	POWER
---	-----	---	-----
1	-11	17	-11
2	-10	18	-11
3	-11	19	-12
4	-13	20	-12
5	-13	21	-12
6	-12	22	-12
7	-12	23	-12
8	-12	24	-12
9	-12	25	-12
10	-12	26	-12
11	-12	27	-12
12	-12	28	-12
13	-12	29	-12
14	-11	30	-12
15	-11	31	-12
16	-11		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL DATE NUMBER	(YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
01	920104	S	744.0	C	1	Refueling outage L2R04.

SUMMARY OF OPERATION:

The unit had a scheduled refueling outage 01/04/92.

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

(None.)

2. ECCS System Outages

(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.

(None.)

4. Major changes to Radioactive Waste Treatment Systems.

(None.)

5. Indications of Failed Fuel Elements.

(None.)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit has been removed from the outage number.

OUTAGE NO.	EQUIPMENT	PURPOSE
1075	2E51-F076	Perform VOTES diagnostic testing
1178	2E51-F059	Remove blank flange for modification support.
1200	2E22-F031	Removal of flushing water spool piece.
1320	2DG01K	Calibrations.
1381	2E12-F411B	Valve disassembly and repair.
1383	2E12-C300A	Pump seal repair.
1391	2E12-F024B 2E12-F027B	Performance of operational testing.
1424	2E12-F059B	Install valve packing.
1440	2DG01K	Change soakback oil filter.
1441 1442	2E51-F063	Valve disassembly and repair.
1444	2DG01K	Inspection of the 'A' phase overcurrent relay.
1449	2E51-F360	Inspection and repair.
1456	2E51-F065	Valve disassembly and repair.
1463	2E51-F063	Administrative OOS for the backfilling of the main steam lines.
1480	2E12-F024A	The 'A' Residual Heat Removal system in shutdown cooling mode.
1487	2E22-F077	Valve disassembly and repair.
1495 1498	2E51-F010 2E51-F046 2E51-F059	Valve testing.
1510	2E12-F024B 2E12-F027B	Valve testing.