

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 2 4	PAGE (3) 1 OF 0 1
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TITLE (4)
Automatic Actuation of Engineered Safety Feature While No Fuel in Reactor

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 5	0 4	8 4	8 4	0 0 6	0 0	0 5	2 9	8 4			0 5 0 0 0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR, § (Check one or more of the following) (11)											

OPERATING MODE (9) 5	POWER LEVEL (10) 0 1 0 1 0	20.402(b)	20.406(e)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
		20.406(a)(1)(i)	50.38(e)(1)		50.73(a)(2)(v)	73.71(c)
		20.406(a)(1)(ii)	50.38(e)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
		20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	
		20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
		20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Mr. J. Pastva, Jr., Regulatory Technician		AREA CODE 9 1 9	NUMBER 4 5 7 1 - 9 5 2 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During a Unit 2 refuel/maintenance outage, while decreasing reactor level for reactor feedwater sparger repairs with the reactor defueled and the fuel pool gates installed, the reactor level was decreased to the low level (LL-2) setpoint. As per design, a Group 1 Primary Containment Isolation Signal (PCIS) and Standby Gas Treatment (SbGT) System initiation occurred and the Reactor Water Cleanup (RWCU) System and the Reactor Building Ventilation (RBV) System isolated. Reactor level was increased to greater than the LL-2 setpoint, the subject isolations and initiations were reset and the affected systems were returned to service. Within approximately six hours, the Unit 2 reactor level decreased to the LL-2 setpoint and the same isolations and initiations, which occurred earlier, were received. The reactor level was increased and the affected systems were once again reset and returned to their initial status.

The procedure used for the Unit 2 reactor level decreasing evolution did not provide for jumpering the reactor LL-2 signal, thereby resulting in the events when level was decreased to the LL-2 setpoint. The subject procedure was revised to provide for jumpering the LL-2 signal.

This event occurred in the most conservative plant condition. There are no credible alternative initial plant conditions for this event.

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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

May 29, 1984

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NRC Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-84-6

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within 30 days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/sdl/LETKB1

Enclosure

cc: Mr. J. P. O'Reilly

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