

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT 1 DOCKET NUMBER (2) 0 5 0 0 0 3 2 1 1 OF 0 2 PAGE (3)

TITLE (4) Missed Surveillances

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)		
04	30	84	84	008	00	05	25	84	E. I. HATCH, UNIT 1	0 5 0 0 0 3 2 1		
										0 5 0 0 0		

OPERATING MODE (9) 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(e)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME T. L. Elton, Acting Superintendent of Regulatory Compliance TELEPHONE NUMBER 9 1 2 3 6 7 1 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 04/30/84, the Hatch site surveillance coordinator was informed of an apparent anomaly concerning the due date of the "STANDBY GAS TREATMENT SYSTEM VENTILATION AND VALVE OPERABILITY" procedure (HNP-2-3655). Further investigation revealed that the surveillance tracking computer had miscalculated the due date for both HNP-2-3655 and other surveillance procedures (refer to continuation sheet for listing of other procedures affected). This resulted in their being performed past their latest allowed date (i.e., the due date plus the 25% grace period allowed by Unit 1 Tech. Specs. section 1.0, definition II and Unit 2 Tech. Specs. section 4.0.2.a).

The correct due dates for the applicable procedures were calculated by hand to prevent eminent recurrence of the error. Additionally, the organization responsible for the programming error was immediately contacted and notified of the discrepancy. The programming error was discovered and corrected, and was linked into production on 05/11/84.

Additionally, an investigation will be conducted to determine if there is a problem with quality control for computer software.

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PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) EDWIN I. HATCH, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 1 8 4 -	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		0 0 8 -	0 0 0 0	2	OF	0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On 04/30/84, with the reactor mode switch in the RUN position and reactor power at 2428 MWT (approximately 100%), the Hatch site surveillance coordinator was informed of an apparent anomaly concerning the due date of the "STANDBY GAS TREATMENT SYSTEM VENTILATION AND VALVE OPERABILITY" procedure (HNP-2-3655). Further investigation revealed that the surveillance tracking computer had miscalculated the due date for HNP-2-3655 (ref. Tech. Specs. sections 4.6.6.1.a & 4.0.5) and the following procedures. This error resulted in these procedures being performed past their latest allowed date (i.e., the due date plus the 25% grace period allowed by Unit 1 Tech. Specs. section 1.0, definition II and Unit 2 Tech. Specs. section 4.0.2.a). Note: This computer program error began on approximately November 29, 1983; thus, the procedures marked with an * were performed late the previous cycle.

1. "RECIRCULATION PUMP DISCHARGE VALVE OPERABILITY" procedure (HNP-1-3161), ref. Tech. Specs. section 4.5.B.1.f
2. *"RHR VALVE OPERABILITY" procedure (HNP-1-3162), ref. Tech. Specs. section 4.5.B.1.e
3. *"RHR SERVICE WATER VALVE OPERABILITY" procedure (HNP-1-3166), ref. Tech. Specs. section 4.5.C.1.a
4. "PRIMARY CONTAINMENT ISOLATION VALVE OPERABILITY" procedure (HNP-1-3962), ref. Tech. Specs. section 4.7.D.1.c.(1)
5. "REACTOR BUILDING TO SUPPRESSION CHAMBER VACUUM RELIEF SYSTEM OPERABILITY" procedure (HNP-1-3956), ref. Tech. Specs. section 4.7.A.3
6. *"FUEL POOL COOLING SYSTEM (RHR INTERTIE) VALVE OPERABILITY" procedure (HNP-1-3977), ref. Tech. Specs. section N/A

No actual or potential safety consequences or implications resulted from this event. The above referenced procedures, although performed late, were performed satisfactorily.

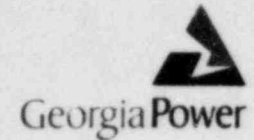
The health and safety of the public were not affected by this non-repetitive event.

The cause of this event is personnel error. A programming error occurred when computer program changes were made which included adding 16 week and 120 day frequencies. A "GOTO" statement should have been inserted at the end of the subroutine that calculates the next due date for 3 month frequencies to bypass the 16 week next due date subroutine. Without this "GOTO" statement, certain procedures having 3 month frequencies that were given completion dates after the program had been revised were assigned next due dates that were approximately 16 weeks after the previous due date.

The correct due dates for the applicable procedures were calculated by hand to prevent eminent recurrence of the error. Additionally, the organization responsible for the programming error was immediately contacted and notified of the discrepancy. The programming error was discovered and corrected, and was linked into production on 5/11/84.

Additionally, an investigation will be conducted to determine if there is a problem with quality control for computer software.

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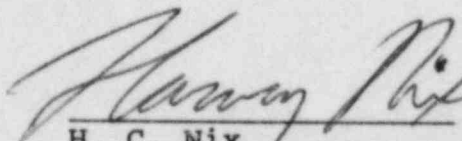
Edwin I. Hatch Nuclear Plant

May 25, 1984
GM-84-413

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Attached is Licensee Event Report No. 50-321/1984-08. This report is required by 10 CFR 50.73 (A)2(i)(B).



H. C. Nix
General Manager

HCN/TLE/djs

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