

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) NORTH ANNA UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 3 8	PAGE (3) 1 OF 0 1
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TITLE (4)
10 CFR 50 APPENDIX R REANALYSIS EQUIPMENT DISCREPANCIES

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 5	0 2	8 4	8 4	0 0 4	0 0	0 5	2 4	8 4	NORTH ANNA UNIT 2		0 5 0 0 0 3 3 9
									0 5 0 0 0		

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 1 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(e)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)

NAME E. WAYNE HARRELL	TELEPHONE NUMBER
	AREA CODE: 7 0 3 8 9 4 5 1 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH: DAY: YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (18)

On May 1, 1984, the preliminary results of a Fire Protection Program reanalysis, prompted by further information of the 10CFR 50 Appendix R requirements was submitted to NRC/NRR Division of Licensing. This report included a description of proposed modifications necessary to meet the Appendix R requirements and several instances where previously installed or proposed modifications require upgrading. In addition, 17 exemption requests with the required justification were also submitted.

The reanalysis findings indicated that there are six instances of previously installed equipment not meeting the requirements of 10CFR 50 Appendix R. These determinations were made after a reappraisal of the Appendix R requirements and recent regulatory classifications.

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TEXT: If more space is required, use additional NRC Form 366A's (17)

On May 1, 1984, a preliminary reanalysis report on the compliance to the design requirements of 10 CFR 50, Appendix R was submitted to the Nuclear Regulatory Commission (NRC). The inspection of existing fire protection equipment performed during the reanalysis identified six cases of equipment which did not meet Appendix R requirements due to a previous misunderstanding of the requirements. Equipment discrepancies along with a discussion of remedial actions and scheduled corrective actions to be taken by North Anna is given below. Since an in-depth report of the Appendix R reanalysis will be submitted to the NRC in the near future, this report will not cover equipment to be installed to meet Appendix R but only discrepancies in existing equipment.

FINDING

1. Inadequate separation of Unit 1 Residual Heat Removal (EIIS system identifier BP) Pumps (EIIS component identifier P).

COMMENT

North Anna Unit 1 uses two redundant RHR pumps for the purpose of maintaining residual heat removal capabilities when the unit is in cold shutdown or refueling Modes (Modes 5 and 6). These two pumps are not separated by the minimum distance of twenty feet as required by Appendix R. The previous analysis only considered the power supply cables and not the pump motors. A radiant energy shield will be installed between the pump motors prior to startup from the 1984 Unit 1 refueling.

FINDING

2. Inadequate fire damper (EIIS component identifier BDMP) between Unit 1 and Unit 2 Emergency Switchgear Rooms (ESGR).

COMMENT

North Anna Units 1 and 2 ESGR's house the emergency power supply equipment for the plant. The rooms are side by side divided by a qualified fire barrier. The construction of a fire damper in one of the wall penetrating ductworks does not satisfy requirements for a 3 hour fire rated damper. A continuous fire watch was established until the damper can be replaced which is expected to occur prior to startup from Unit 1, 1984 refueling outage.

FINDING

3. Inadequate fire doors (EIIS component identifier DR) from Health Physics area to the Auxiliary Building and from the Turbine Building to the Unit 2 ESGR.

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TEXT (If more space is required, use additional NRC Form 368A (17))

COMMENT

Both doors separate fire zones and are three hour rated doors. However there has been damage to the doors that may adversely affect this 3 hour rating. The door frames are not qualified for three hours but are embedded in concrete walls. An exemption for the door frames is being pursued due to the nature of construction being capable of withstanding the three hour requirement. The doors are being repaired as an interim measure until replacement doors designed for heavy usage can be procured and installed.

FINDING

- Inadequate fire dampers between the Control Room and the Cable Tray Room.

COMMENT

The duct work penetration between the Control Room and Cable Tray Room which provides normal supply and exhaust air for the Control Room, was found to have the fire damper fusible links located out of the main air stream. The fusible links will be moved into the flow area prior to startup from the 1984, Unit 1 refueling outage.

FINDING

- Inadequate fire suppression (EIIS component identifier SRNK) in portions of the Auxiliary Building near the Charging Pump (EIIS system identifier BQ component identifier P) and Component Cooling Pumps (EIIS system identifier CC component identifier P).

COMMENT

Power leads to the Charging Pumps and Component Cooling Pumps for Units 1 and 2, located in the Auxiliary Building, do not have the required 20 foot separation without intervening combustibles in accordance with Appendix R. A modification will be installed to provide; (a) fire protective wrapping of cables as needed to meet the separation requirements of Appendix R, Section III.G.2.; (b) solid metal tray covers and fire stops as needed for intervening cable trays; (c) extension of the existing automatic sprinkler system on Elev. 259'-6" in the Auxiliary Building; and (d) additional smoke detectors on Elev. 244' 6" and Elev. 259' 6". These modifications are scheduled to be completed by October 1, 1984. A continuous fire watch in this area will be maintained until these modifications are completed.

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FINDING

- 6. Inadequate emergency lighting (EIS component identifier IL) previously installed to meet Appendix R para. III J.

COMMENT

A test was conducted to determine the adequacy of existing emergency lighting to provide adequate illumination in all areas needed for operation of safe shutdown equipment and in access and egress routes thereto. Some areas were found inadequate. Six portable lanterns have been provided for the operations staff as a remedial action. The emergency lighting system discrepancies are scheduled to be remedied for Unit 1 and common areas during the current Unit 1 refueling outage and for Unit 2 during the Unit 2 1984 refueling outage.

Since remedial and scheduled actions are being taken on all the discrepancies the health and safety of the public are not affected.



VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

May 24, 1984

U. S. Nuclear Regulatory Commission
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016 Phillips Building
Washington, D.C. 20555

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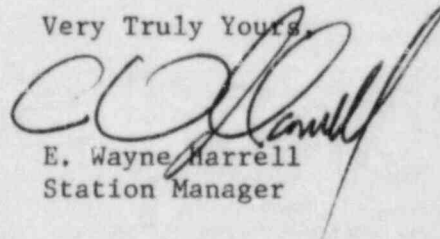
Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No. LER 84-004-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours



E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

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