The Light company
Houston Lighting & Power South Texas Project Electric Generating Station P. O. Box 289 Wadsworth, Texas 77483

April 10, 1992
ST-HL-AE-4061
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10CFR50.71

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

South Texas Project
Units 1 & 2
Docket Nos. STN 50-498 & 50-499
Monthly Operating Reports for March, 1992

Pursuant to 10CFR50.71(a) and South Texas Project Electric Generating Station (STPEGS) Technical Specification 6.9.1.5, attached are the Monthly Operating Reports for March, 1992.

If you should have any questions on this matter, please contact Mr. C. A. Ayala at (512) 972-8628.

William J. Jump
William J. Jump
Manager
Nuclear Licensing

MKJ/lf

Attachments: 1) STPEGS Unit 1 Monthly Operating Report - March, 1992
2) STPEGS Unit 2 Monthly Operating Report - March, 1992

IE24
1/1

cc:

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Revised 10/11/91

L4/NRC/

SOUTH TEXAS PROJECT

ELECTRIC GENERATING STATION

UNIT 1

MONTHLY OPERATING REPORT

MARCH 1992

HOUSTON LIGHTING AND POWER CO.

NRC DOCKET NO. 50-498

LICENSE NO. NPF-76

Reviewed By: _____ 4-7-92
Supervisor Date

Reviewed By: D.C. Logan 4-7-92
Plant Engineering Manager Date

Approved By: _____ 4/8/92
Plant Manager Date

Monthly Summary

STPEGS Unit 1 began the reporting period at 100% reactor power.

On 3/14/92 at 1109 a reactor trip occurred due to a momentary false reactor coolant flow trip signal. Maintenance was being performed to calibrate a LOOP 2 Reactor Coolant Flow, Protection Set flow transmitter. Following calibration, when returning the transmitter to service, the high pressure side of the transmitter was opened first instead of the low pressure side to pressurize the transmitter. This caused a momentary low (below setpoint) differential pressure to be detected by two adjacent flow transmitters which initiated a reactor trip caused by a reactor coolant low flow indication.

The cause of the trip was attributed to failure to follow procedures. The unit was returned to service on 3/17/92 at 2359 and concluded the reporting period at full power.

OPERATING DATA REPORT

DOCKET NO. 50-498
UNIT 1
DATE Apr. 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

OPERATING STATUS

1. REPORTING PERIOD: 03/01-03/31 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3800
   MAX.DEPEND.CAPACITY (MWe-Net): 1250.6
   DESIGN ELECTRICAL RATING (MWe-Net): 1250.6
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY)(MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY): N/A

		THIS MONTH	YR TO DATE	CUMULATIVE
5.	NUMBER OF HOURS REACTOR WAS CRITICAL	668.6	2108.6	22128.5
6.	REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7.	HOURS GENERATOR ON LINE	659.2	2099.2	21311.4
8.	UNIT RESERVE SHUTDOWN HOURS	0	0	0
9.	GROSS THERMAL ENERGY GENERATED (MWt)	2470851	7960369	78114691
10.	GROSS ELECTRICAL ENERGY GENERATED (MWH)	834460	2718120	26426120
11.	NET ELECTRICAL ENERGY GENERATED (MWH)	798605	2606156	24934525
12.	REACTOR SERVICE FACTOR	89.9%	96.5%	70.1%
13.	REACTOR AVAILABILITY FACTOR	89.9%	96.5%	70.1%
14.	UNIT SERVICE FACTOR	88.6%	96.1%	67.5%
15.	UNIT AVAILABILITY FACTOR	88.6%	96.1%	67.5%
16.	UNIT CAPACITY FACTOR (Using MDC)	85.8%	95.4%	63.2%
17.	UNIT CAPACITY FACTOR (Using Design MWe)	85.8%	95.4%	63.2%
18.	UNIT FORCED OUTAGE RATE	11.4%	3.9%	13.4%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

    Scheduled 8 day maintenance outage to begin April 6, 1992. Refueling and scheduled maintenance outage to begin September 19, 1992.

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-498
UNIT 1
DATE Apr. 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

MONTH MARCH

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1258	17	0
2	1257	18	652
3	1258	19	1215
4	1258	20	1217
5	1258	21	1217
6	1255	22	1219
7	1253	23	1221
8	1252	24	1221
9	1254	25	1221
10	1256	26	1219
11	1255	27	1207
12	1258	28	1131
13	1256	29	1216
14	555	30	1219
15	0	31	1218
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-4[illegible]
UNIT 1
DATE Apr. [illegible]
COMPLETED BY A.P. [illegible]
TELEPHONE 512/9[illegible]

REPORT MONTH MARCH

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
92-01	920314	F	84.8	G	3	1-92-003	AB	FIT	A reactor trip occurred due to a momentary false reactor coolant flow trip signal. Maintenance was being performed to calibrate a LOOP 2 Reactor Coolant Flow Protection Set flow transmitter. Following calibration, when returning the transmitter to service, the high side of the transmitter was opened first instead of the low pressure side to pressurize the transmitter. This caused a momentary low (below setpoint) differential pressure to be detected by two adjacent flow transmitters which initiated a reactor trip caused by a reactor coolant low flow indication. Corrective actions to prevent recurrence will be addressed in the LER.

[1] F: Forced
S: Scheduled

[2] Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Exam
F-Administrative
G-Operational Error (Explain)

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Cont. of Existing Outage
5-Reduction
9-Other

[4] IEEE 805-1983

[5] IEEE 803A-1983

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PORVs and Safety Valves Summary

There were no PORV or Safety Valves challenged during the reporting period.

SOUTH TEXAS PROJECT

ELECTRIC GENERATING STATION

UNIT 2

MONTHLY OPERATING REPORT

MARCH 1992

HOUSTON LIGHTING AND POWER CO.

NRC DOCKET NO. 50-499

LICENSE NO. NPF-80

Reviewed By: _____ 4-7-92
Supervisor Date

Reviewed By: D. C. Logan 4-7-92
Plant Engineering Manager Date

Approved By: _____ 4/8/92
Plant Manager Date

Monthly Summary

STPEGS Unit 2 began the reporting period at 100% reactor power.

On 3/9/92 Steam Generator Feedwater Pump Turbine 23 was tripped due to an electro-hydraulic control fluid leak on the line to the low pressure stop valve. The fluid leak caused the stop valve to stick in the open position. A weld had cracked causing a leak to develop. The weld and a portion of the line was replaced. Two hydraulic actuators were replaced to allow the stop valve to return to service. During the loss of the feedwater pump turbine, the Master Controller was responding erratically. Reactor power was reduced to 49% to minimize the possibility of a unit trip during troubleshooting. A spike was detected on an auxiliary relay card in the 7300 cabinet. The card was replaced and the unit was returned to full power on 3/12/92.

On 3/30/92 Steam Generator Feedwater Pump Turbine 22 tripped on low oil pressure when a main oil pump tripped due to a fault in the cabling. Reactor power was reduced to 48% while investigating the cause of the pump trip. The cable at the oil pump was repaired. The unit completed the reporting period with the unit approaching full power.

OPERATING DATA REPORT

DOCKET NO. 50-499
UNIT 2
DATE Apr. 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

OPERATING STATUS

1. REPORTING PERIOD: 03/01-03/31 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3800
   MAX.DEPEND.CAPACITY (MWe-Net): 1250.6
   DESIGN ELECTRICAL RATING (MWe-Net): 1250.6
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY)(MWe-Net): None
4. REASONS FOR RESTRICTION (IF ANY): N/A

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	2035.6	17458.2
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	744.0	1998.3	16480.5
8. UNIT RESERVE SHUTDOWN HOURS	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWt)	2663411	7366291	59759562
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	908680	2513490	20180850
11. NET ELECTRICAL ENERGY GENERATED (MWH)	870309	2407855	19111858
12. REACTOR SERVICE FACTOR	100.0%	93.2%	71.5%
13. REACTOR AVAILABILITY FACTOR	100.0%	93.2%	71.5%
14. UNIT SERVICE FACTOR	100.0%	91.5%	67.5%
15. UNIT AVAILABILITY FACTOR	100.0%	91.5%	67.5%
16. UNIT CAPACITY FACTOR (Using MDC)	93.5%	88.2%	62.6%
17. UNIT CAPACITY FACTOR (Using Design MWe)	93.5%	88.2%	62.6%
18. UNIT FORCED OUTAGE RATE	0.0%	8.5%	15.5%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):

    N/A

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-499
UNIT 2
DATE Apr. 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

MONTH MARCH

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1251	17	1252
2	1254	18	1255
3	1255	19	1257
4	1256	20	1255
5	1255	21	1256
6	1254	22	1256
7	1255	23	1252
8	1255	24	1250
9	976	25	1250
10	543	26	1255
11	545	27	1256
12	962	28	1253
13	1252	29	1256
14	1251	30	1252
15	1249	31	649
16	1249		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-499
UNIT 2
DATE Apr. 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

REPORT MONTH MARCH

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
92-03	920309	F	0.0	B	5	N/A	JC	P	Steam Generator Feedwater Pump Turbine 23 was tripped due to an electro-hydraulic control fluid leak on the line to the low pressure stop valve. The fluid leak caused the stop valve to stick in the open position. A weld had cracked causing a leak to develop. The weld and a portion of the line was replaced. Two hydraulic actuators were replaced to allow the stop valve to return to service. During the loss of the feedwater pump turbine, the Master Controller was responding erratically. The reactor was reduced to 49% to minimize the possibility of a unit trip during troubleshooting. A spike was detected on an auxiliary relay card in the 7300 cabinet. The card was replaced.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Cont. of Existing Outage
5-Reduction
9-Other

4 IEEE 805-1983

5 IEEE 803A-1983

ATTACHMENT 2
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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-499
UNIT 2
DATE Apr. 2, 1992
COMPLETED BY A.P. Kent
TELEPHONE 512/972-7786

REPORT MONTH MARCH

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
92-04	920330	F	0.0	B	5	N/A	SJ	P	Steam Generator Feedwater Pump Turbine 22 tripped on low oil pressure when a main oil pump tripped due to a fault in the cabling. Reactor power was reduced to 48% while investigating the cause of the pump trip. The cable at the oil pump was repaired.

[1] F: Forced
S: Scheduled

[2] Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Cont. of Existing Outage
5-Reduction
9-Other

[4] IEEE 805-1983

[5] IEEE 803A-1983

ATTACHMENT 2
ST-HL-AE-4061
PAGE 6 OF 7

PORVs and Safety Valves Summary

There were no PORV or Safety Valves challenged during the reporting period.