

ENCLOSURE 1

PROPOSED ANO-1 OPERATING LICENSE CHANGE

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PDR ADOCK 05000313
P PDR

1. Identification of a sampling schedule for the critical parameters and control points for these parameters;
2. Identification of the procedures used to measure the values of the critical parameters;
3. Identification of process sampling points;
4. Procedures for the recording and management of data;
5. Procedures defining corrective actions for off-control point chemistry conditions; and
6. A procedure identifying the authority responsible for the interpretation of the data and the sequence and timing of administrative events required to initiate a corrective action.

(8) FIRE PROTECTION

EOI shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in Amendment 9A to the Safety Analysis Report and as approved in the Safety Evaluation Report dated _____, subject to the following provision:

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This license is effective as of the date of issuance and shall expire at midnight, May 20, 2014.

FOR THE ATOMIC ENERGY COMMISSION

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachment:
Appendices A and B - Technical
Specifications

Date of Issuance: May 21 1974

ENCLOSURE 2

PROPOSED ANO-1 TECHNICAL SPECIFICATION CHANGES

3.5.5 Fire Detection Instrumentation

DELETED

Table 3.5-5

SAFETY-RELATED AREAS PROTECTED BY HEAT/SMOKE DETECTORS

DELETED

3.17 Fire Suppression Water System

DELETED

3.18 FIRE SUPPRESSION SPRINKLER SYSTEM

DELETED

3.19 CONTROL ROOM AND AUXILIARY CONTROL ROOM HALON SYSTEMS

DELETED

3.20 FIRE HOSE STATIONS

DELETED

3.21 FIRE BARRIERS

DELETED

4.19 FIRE DETECTION INSTRUMENTATION

DELETED

1

4.20 Fire Suppression Water System

DELETED

|

DELETED

|

Bases

DELETED

|

4.21 SPRINKLER SYSTEMS

DELETED

4.22 Control Room and Auxiliary Control Room Halon Systems

DELETED

4.23 Fire Hose Stations

DELETED

4.24 Fire Barriers

DELETED

Table 6.2-1

ARKANSAS NUCLEAR ONE

MINIMUM SHIFT CREW COMPOSITION #

UNIT 1

LICENSE CATEGORY	ABOVE COLD SHUTDOWN	COLD AND REFUELING SHUTDOWNS
SOL	2	1*
OL	2	1
NON-LICENSED	3	1
SHIFT TECHNICAL ADVISOR	1	None required

*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising refueling operations after the initial fuel loading.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

Additional Requirements:

1. At least one licensed Operator shall be in the control room when fuel is in the reactor.
2. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
3. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
4. All refueling operations after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
5. DELETED

- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant Manager, ANO-1, General Manager, Plant Operations or the Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the General Manager, Plant Operations and the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the General Manager, Plant Operations and the Safety Review Committee.
- k. Review of changes to the Offsite Dose Calculation Manual and the Process Control Program.
- l. Review of changes to the Fire Protection Program and implementing procedures and submittal of recommended changes to the General Manager, Plant Operations and Safety Review Committee.

AUTHORITY

6.5.1.7. The Plant Safety Committee shall:

- a. Recommend in writing their approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President, Operations ANO and the Safety Review Committee of disagreement between the PSC and the Plant Manager, ANO-1 or the General Manager, Plant Operations; however, the General Manager, Plant Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Safety Committee shall maintain written minutes of each PSC meeting that, at a minimum, document the results of all PSC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Plant Manager, ANO-1, General Manager, Plant Operations and Chairman of the Safety Review Committee.

6.5.2 Safety Review Committee (SRC)

FUNCTION

6.5.2.1 The Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry

ENCLOSURE 3

PROPOSED ANO-2 OPERATING LICENSE CHANGE

- (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) EOI, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This amended license shall be deemed to contain and is subject to conditions specified in the following Commission regulations in 10 CFR Chapter I; Part 20, Section 30.34 of part 30, Section 40.41 of part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

EOI is authorized to operate the facility at steady state reactor core power levels not in excess of 28 megawatts thermal. Prior to attaining this power level EOI shall comply with the conditions in Paragraph 2.C.(3).

(2) Technical Specifications

Amendment
#122
08/06/91

The Technical Specifications contained in Appendix A, as revised through Amendment No. 122 are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications.

Exemptive 2nd paragraph of 2.C.2 deleted per Amendment 20, 3/3/81.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the issuance of the license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the license supported by a favorable evaluation by the Commission.

2.C.(3)(a) Deleted per Amendment 24, 6/19/81.

(b) Fire Protection

EOI shall implement and maintain in effect all provisions of the approved fire protection program as

described in Amendment 9A to the Safety Analysis Report
and as approved in the Safety Evaluation Report dated
_____, subject to the following provision:

The licensee may make changes to the approved fire
protection program without prior approval of the
Commission only if those changes would not
adversely affect the ability to achieve and
maintain safe shutdown in the event of a fire.

(c) Less Than Four Reactor Coolant Pump Operation

EOI shall not operate the reactor in operational Modes 1 and 2 with fewer than four reactor coolant pumps in operation, except as allowed by Special Test Exception 3.10.3 of the facility Technical Specifications.

2.C.(3)(d) Deleted per Amendment 24, 6/19/81.

(e) Fire Protection

AP&L shall complete the following modifications by the indicated dates in accordance with the staff's findings as set forth in the fire protection evaluation report, NUREG-0223 "Fire Protection Safety Evaluation Report."

Implementation Dates for Proposed Modifications

<u>Applicable Section of NUREG-0223</u>		<u>Date</u>
3.1	Portable Radio Communication Equipment	March 31, 1979
3.2	Separation of Power Cables in Manholes	*
3.3	Protection from Water Spray	*
3.4	Protection of Redundant Cables in the MCC Room (2096-M)	December 30, 1978
3.5	Protection of Redundant Cables in the Hallway - Elevation 372 (2109-U)	*,**
3.6	Protection of Redundant Cables in the Cable Spreading Room (2098-L)	*
3.7	Protection of Redundant Cables in the Switchgear Room (2103-Z)	*
3.8	Protection of Redundant Cables in the Electrical Equipment Room (2091-BB)	September 30, 1978

ENCLOSURE 4

PROPOSED ANO-2 TECHNICAL SPECIFICATION CHANGES

INSTRUMENTATION

FIRE DETECTION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

DELETED

TABLE 3.3-11

FIRE DETECTION INSTRUMENTS

DELETED

PLANT SYSTEMS

3/4.7.10 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

LIMITING CONDITION FOR OPERATION _____

DELETED

PLANT SYSTEMS

ACTION (Continued)

DELETED

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

DELETED

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

DELETED

PLANT SYSTEMS

SPRAY AND/OR SPRINKLER SYSTEMS

LIMITING CONDITION FOR OPERATION

DELETED

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS

DELETED

PLANT SYSTEMS

FIRE HOSE STATIONS

LIMITING CONDITION FOR OPERATION

DELETED

TABLE 3.7-7

FIRE HOSE STATIONS

DELETED

PLANT SYSTEMS

3/4.7.11 FIRE BARRIERS

LIMITING CONDITION FOR OPERATION

DELETED

- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. DELETED
- g. Administrative control shall be established to limit the amount of overtime worked by plant staff performing safety-related functions. These administrative controls shall be in accordance with the guidance provided by the NRC Policy Statement on working hours (Generic Letter No. 82-12).
- h. The Manager, Operations and Shift Supervisor shall hold a senior reactor operator license.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the designated radiation protection manager, who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and (2) the Shift Technical Advisor who shall have a bachelors degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager, Training and Emergency Planning and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 DELETED

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY COMMITTEE (PSC)

FUNCTION

6.5.1.1 The Plant Safety Committee shall function to advise the General Manager, Plant Operations and Plant Manager, ANO-2 on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Safety Committee shall be composed of eight members of ANO onsite management organization (except as discussed under 6.5.1.3) at the superintendent level or above. The PSC Chairman shall ensure that adequate expertise is present during meetings to evaluate material before the PSC.

In addition, the General Manager, Plant Operations shall designate in writing a PSC Chairman and at least one Alternate Chairman.

6.5.1.3 If Core Protection Calculator (CPC) Software is being reviewed a nuclear software expert shall be present as a voting member. If one of the members of the Plant Safety Committee meets the qualification requirements for this position, the requirement to have this member is satisfied. This membership may be filled by two appropriately qualified individuals who shall ballot with a single combined vote. Generic qualifications for this membership shall be as follows:

ADMINISTRATIVE CONTROLS

- f. Review of all REPORTABLE EVENTS.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses, and reports thereon as requested by the Plant Manager, ANO-2, General Manager, Plant Operations or the Safety Review Committee.
- i. Review of the Plant Security Plan and implementing procedures and submittal of recommended changes to the General Manager, Plant Operations and the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the General Manager, Plant Operations and Safety Review Committee.
- k. Review of changes to the Offsite Dose Calculation Manual and Process Control Program.
- l. Review of changes to the Fire Protection Program and implementing, procedures and submittal of recommended changes to the General Manager, Plant Operations and Safety Review Committee.

AUTHORITY

6.5.1.8 The Plant Safety Committee shall:

- a. Recommend in writing their approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President, Operations ANO and the Safety Review Committee of disagreement between the PSC and the Plant Manager, ANO-2 or the General Manager, Plant Operations; however, the General Manager, Plant Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.9 The Plant Safety Committee shall maintain written minutes of each PSC meeting that, at a minimum, document the results of all PSC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Plant Manager, ANO-2, General Manager, Plant Operations and Chairman of the Safety Review Committee.