

Docket Nos.: STN 50-482  
and STN 50-483

MAY 14 1984

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Gentlemen:

Subject: Request for Additional Information

Reference: B. J. Youngblood (NRC) letter to Donald F. Schnell (UE)  
and Glenn L. Koester (KG&E), Subject: "Request for  
Additional Information," dated May 7, 1984

In the referenced letter, the staff forwarded 16 questions on the Callaway/  
Wolf Creek Technical Specifications. Enclosed are two additional questions which  
were inadvertently omitted from the May 7, 1984 request. Please provide your  
responses to the enclosure on a schedule which is consistent with the one in  
the referenced letter.

Sincerely,

ORIGINAL SIGNED BY:

B. J. Youngblood, Chief  
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Enclosure:  
As stated

cc: See next page

DISTRIBUTION:

<del>Docket File</del>	MRushbrook
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L PDR	PO'Connor
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PRC System	ACRS (16)
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TMarsh, RSB	NGrace

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ENCLOSURE

17. Reactor Trip System Instrumentation (Section 3/4.3.1, Table 3.3-1 page 3/4 3-2)

This table specifies the minimum number of reactor trip system instrumentation channels operable and the applicable modes of their operation.

Item 6.c on this table specifies that only one Source Range Monitor (SRM) channel is required to be operable during Modes 3, 4 and 5. During these modes the SRM does not provide a reactor trip function. However, it provides a boron dilution mitigation function by sensing the increase in neutron flux, and upon flux doubling it initiates the valve motion that isolates the unborated water sources and aligns the borated water source (RWST) to the charging pumps suction. One operable SRM represents a single point of vulnerability for the boron dilution mitigation system (BDMS). During the FSAR review stage, the applicant committed to install BDMS identical in design to that of the Comanche Peak Steam Electric Station (CPSSES). The applicant also committed to meet the same design criteria and standards as those for the CPSSES system. The CPSSES BDMS, as reviewed and approved by the staff, is a fully redundant system which relies on two SRMs powered by two different

class 1E buses. Allowing only a single channel of source range monitor for Boron Dilution mitigation is a deviation from the FSAR analysis assumptions as reviewed approved by the staff and should be corrected.

18. Special Test Exceptions, Reactor Coolant Loops (Section 3/4.10.4, page 3/4 10-4)

This technical specification permits plant operation without any reactor coolant pumps operating up to 10% thermal power on fission heat for startup or physics tests. The staff is unaware of any safety analysis that demonstrates that transients or accidents initiated from this operating condition would be acceptable. Both the steady state and transient reactor coolant system temperature profiles, margin to saturation, core DNBR, and other related thermal-hydraulic and fuel performance parameters, particularly thermal-hydraulic stability, should be assessed. The acceptability of the reactor protective system setpoints during various transients and accidents initiated from this condition must also be justified.