

NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

July 26, 1991

Docket No. 50-341

Mr. William S. Orser Senior Vice President - Nuclear Operations Detroit Edison Company 6400 North Dixie Highway Newport, Michigan 48166

Dear Mr. Orser:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION RE: GENERIC LETTER (GL) 89-10, SUPPLEMENT 3: "CONSIDERATION OF THE RESULTS OF NRC-SPONSORED TESTS OF MOTOR-OPERATED VALVES" (TAC NO. 77775)

By letter dated December 10, 1990, you submitted a response to the 120-day, 10 CFR 50.54(f), reporting requirements of GL 89-10, Supplement 3, for Fermi-2. After reviewing your response, the staff has concluded that the additional information identified in the enclosure is necessary to make a safety determination regarding the subject motor-operated valves.

Response to the enclosed Request for Additional Information (RAI) is requested within 30 days from receipt of this letter.

This request is covered by Office of Management and Budget Clearance (OMB) Number 3150-0011, which expires June 30, 1991. A clearance extension for Part 50 is under review by OMB and is expected to be granted. The estimated average burden hours are 80 person-hours per licensee response, including searching data sources, gathering and analyzing the data, performing data evaluations, and preparing the required letters. (These estimated average burden hours pertain only to the identified response-related matters and do not include the time for actual implementation of the requested action.) Send comments regarding this burden estimate or any other aspect of this collection of information, including suggestions for reducing this burden, to the Information and Records Management Branch, Division of Information Management, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to the Paperwork Reduction Project (3150-0011), Office of Management and Budget, Washington, D.C. 20503.

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Should you have any questions regarding this RAI, please contact me at (301) 492-1345.

Sincerely,

John F. Stang, Project Manager

Project Directorate III-I Division of Reactor Projects III/IV/V Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

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Document Name: RAI TAC NO. 77775

Should you have any questions regarding this RAI, please contact me at (301) 492-1345.

Sincerely,

John F. Stang, Project Manager Project Directorate III-1 Division of Reactor Projects III/IV/V Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

cc: John Flynn, Esq. Senior Attorney Detroit Edison Company 2000 Second Avenue Detroit, Michigan 48226

Nuclear Facilities and Fnvironmental Monitoring Section Office Division of Radiological Health P. O. Box 30195 Lansing, Michigan 48909

Mr. Walt Rogers U.S. Nuclear Regulatory Commission Resident Inspector's Office 6450 W. Dixie Highway Newport, Michigan 48166

Monroe County Office of Civil Preparedness 963 South Raisinville Monroe, Michigan 48161

Regional Administrator, Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

Ms. Lynne Goodman Director - Nuclear Licensing Detroit Edison Company Fermi Unit 2 6400 North Dixie Highway Newport, Michigan 48166 REQUEST FOR ADDITIONAL INFORMATION
RE: SUPPLEMENT 3 TO GENERIC LETTER 89-10,
"CONSIDERATION OF THE RESULTS OF NRC-SPONSORED TESTS OF
MOTOR-OPERATED VALVES"
FERMI

- 1. Identify any modifications (e.g., torque switch setting adjustments, gearing changes, or motor/actuator replacement) for each MoV within the scope of Supplement 3 to GL 89-10 since June 1990 or planned for the future.
- 2. Identify the particular valves that are flex wedge, split wedge, or parallel disk gate design.
- 3. Provide information necessary to confirm motor adequacy for each MOV within the scope of Supplement 3.
- 4. Explain the "10-2" percent closure of a valve. Are the leakage rates within the limits of Appendix J and the ASME Code at this percent closure?
- 5. What is the schedule for the testing of the RWCU MOV? The staff's safety assessment discussed in Supplement 3 to GL 89-10 supports continued operation for 18 months or the first refueling outage. If the RWCU test schedule is beyond those dates, provide the safety assessment to justify continued operation beyond the recommended schedule.
- 6. How have you addressed the rate of loading phenomenon in MOV sizing and torque switch settings?