July 26, 1991

Docket Nos. 50-269

LICENSEE: DUKE POWER COMPANY

FACILITY: OCONSE NUCLEAR STATION, UNIT 1

SUBJECT: SUMMARY OF JUNE 18, 1991 HEETING TO DISCUSS 10-YEAR ISI VESSEL INSPECTION FOR OCONEE UNIT 1

On June 18, 1991, the NRC staff met with representatives of Duke Power Company (DPC), and a consultant from Babcock and Wilcox at OWFN, Rockville, Maryland. The purpose of the meeting was to discuss DPC's upcoming reactor vessel ISi inspection for Oconee Unit 1. Meeting attendees are listed in Enclosure 1, and the meeting agenda is included in Enclosure 2.

Following brief introductory remarks, Mr. C. B. Cheezem of DPC addressed present ASME Code requirements, the weld layout on the reactor vessel, and the impact of proposed augmented requirements. Mr. M. G. Hacker of B&W presented technology enhancements as applicable to D&W Owners Group and their impact on the forthcoming reactor vessel inspection for Oconee Unit 1. Handouts used during these two presentations are provided as Enclosure 3.

Members of RII/DRS/EB and NRR/DET/EMCB discussed the proposed 10-year ISI vessel inspection program and expressed interest in observing portions of the upcoming reactor vessel inspection. The inspection is scheduled to take place August 13-27, 1991.





UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20556 July 26, 1991

Docket Nos. 50-269

LICENSEE: DUKE POWER COMPANY

FACILITY: OCONEE NUCLEAR STATION, UNIT 1

SUBJECT: SUMMARY OF JUNE 18, 1991 MEETING TO DISCUSS 10-YEAR ISI VESSEL INSPECTION FOR OCONEE UNIT 1

On June 18, 1991, the NRC staff met with representatives of Duke Power Company (DPC), and a consultant from Babcock and Wilcox at OWFN, Rockville, Maryland. The purpose of the meeting was to discuss DPC's upcoming reactor vessel ISI inspection for Oconee Unit 1. Meeting attendees are listed in Enclosure 1, and the meeting agenda is included in Enclosure 2.

Following brief introductory remarks, Mr. C. B. Cheezem of DPC addressed present ASME Code requirements, the weld layout on the reactor vessel, and the impact of proposed augmented requirements. Mr. M. G. Hacker of B&W presented technology enhancements as applicable to B&W Owners Group and their impact on the forthcoming reactor vessel inspection for Oconee Unit 1. Handouts used during these two presentations are provided as Enclosure 3.

Members of RII/DRS/EB and NRR/DET/EMCB discussed the proposed 10-year ISi vessel inspection program and expressed interest in observing portions of the upcoming reactor vessel inspection. The inspection is scheduled to take place August 13-27, 1991.

grance Remald.

Frank Rinaldi, Project Engineer Project Directorate II-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures: As stated

cc/enclosures: See next page Oconee Nuclear Station Duke Power Company

CC:

 Mr. A.V. Carr, Esq. Duke Fower Company 422 South Church Street Charlotte, North Carolina 28242-0001

J. Michael McGarry, III, Esq. Winston and Strawn 1400 L Street, N.W. Washington, D.C. 20005

Mr. Kobert B. Borsum Babcock & Wilcox Nuclear Power Division Suite 525 1700 Rockville Pike Rockville, Maryland 20852

Manager, LIS NUS Corporation 2650 McCormick Drive, 3 Floor Clearwater, Florida 34619-1035

Senior Resident Inspector U.S. Nuclear Regulatory Commission Route 2, Box 610 Seneca, South Carolina 29678

Regional Administrator, Region II U.S. Nuclear Regulatory Commission 101 Marietta Street, N.W., Suite 2900 Atlanta, Georgia 30323

Mr. Heyward G. Shealy, Chief Bureau of Radiological Health South Carolina Department of Health and Environmental Control 2600 Rull Street Columbia, South Carolina 29201

Office of Intergovernmental Relations 116 West Jones Street Raleigh, North Carolina 27603

County Supervisor of Oconee County Walhalla, South Carolina 29621 Mr. Stephen Benesole Duke Power Company Post Office Box 1007 Charlotte, North Carolina 28201-1007

Mr. Alan R. Herdt, Chief Project Sranch #3 U.S. Muclear Regulatory Commission 101 Marietta Street, NW, Suite 2900 Atlanta, Georgia 30323

Ms. Karen E. Long Assistant Attorney General N. C. Department of Justice P.O. Box 629 Raleigh, North Carolina 27602

Mr. R.L. Gill, Jr. Nuclear Production Department Duke Power Company P.O. Box 1007 Charlotte, North Carolina 28201-1007

Mr. M. S. Tuckman Vice President -Nuclear Operations Duke Power Company P. O. Box 1007 Charlotte, NC 28201-1007

Enclosure 1

#### ATTENDANCE LIST

OCONEE - UNIT 1

#### MEETING WITH DUKE POWER COMPANY

## REGARDING ISI REACTOR VESSEL INSPECTION PROGRAM

#### NAME

#### ORGANIZATION

Frank Rinaldi Jerome Blake George Johnson James L. Coley, Jr. David E. Smith M. G. Hacker C. B. Cheezem R. G. Rouse Lynn Conner Luellen Jones C. Y. Cheng R. A. Hermann S. G. Benesole

ø

NRC/NPR/DRPE/PDI1=3 NRC/NRR/DRS/EB NRC/NRR/DET/EMCB NRC/RI1/DRS/EB NRC/NRR/DET/EMCB B&W NUCLEAR SERVICE CO. DUKE POWER CO. DUKE POWER CO. SOUTHERN TECHNICAL SERVICES DUKE POWER CO. NRC/NRR/DET/EMCB NRC/NRR/DET/EMCB DUKE POWER COMPANY

## MEETING AGENDA OCONEE UNIT 1 ARIS - II EXAMINATION

.

ENCLOSURE 2

JUNE 18, 1991

Introduction and Purpose	C. B. Cheezem
Present Code Requirements	C. B. Cheezem
Impact of the Proposed Augmented Requirements	C. B. Cheezem
Technology Enhancements	M. G. Hacker
Discussion	All

## Review Of The Present Code Requirements

10CFR 50.55 ¶ a.b.2 requires use of 1980 Edition of ASME Section XI, Division 1 to include 1980 Winter Addenda

Welds' to be examined include those outlined in Categories:

B-A Reactor Vessel B-D Full Penetration Welds and Nozzles in Reactor Vessel B-F Lissimilar Metal Welds - Core Flood Nozzles B-J Main Loop Piping

Reference page 4 for diagram of Reactor Vessel

## Supplemental Information For Present Requirements

ASME Section XI states that Category B-D items shall be examined by the end of the first inspection period and the remainder by the end of the third inspection period of each inspection interval.

#### ASME Section XI ¶ IWB-2420(a) states that the sequence of component examinations established during the first inspection interval shall be repeated during each successive inspection interval to the extent practical.

Request for Relief serial number ONS-014 was originated in 1990 for Units 1, 2 and 3 at Oconee. This request allows Duke Power Company to take credit for the 3rd interval, 1st period requirement for the 36" outlet nozzle-to-vessel welds, as well as nozzle-to-pipe welds. We plan to examine these weids during RFO#13 in August 1991 using an enhanced ultrasonic technique that will meet the 1989 edition of ASME Section XI. This will establish the examination sequence for future intervals.

# ASME Section XI Table IWB-2500-1 requires that Category B-F welds receive a volumetric and surface examination.

Request for Relief serial number ONS-001 was originated in 1984 for Units 1, 2, and 3 at Oconee. This request allows Duke Power Company to examine Category B-F (Item B5.10,Core Flood Nozzle) welds using ultrasonics from the inside surface, in lieu of performing a surface examination. This examination will provide adequate assurance of weld integrity at the OD surface.

# ASME Section XI Table IWB-2500-1 requires that Catego B-Jwelds receive a volumetric and surface examination.

Request for Relief serial number ONS-002 was originated in 1984 for Units 1, 2, and 3 at Oconee. This request allows Duke Power Company to examine Category B-J (Item B9.11,Reactor Vessel Nozzle to Pipe) welds using ultrasonics from the inside surface, in lieu of performing a surface examination. This examination will provide adequate assurance of weld integrity at the OD surface.

Reference Page 6 for Diagram of areas to be examined

ENCLOSURE 3



## Impact Of The Proposed Augmented Requirements

The proposed augmented examination will require inspection of 100% of all B1.10 Category welds (see Page 6) during this inspection, or within 3 years after this inspection.

We will examine all of the accessible length of all B1.10 Category welds during this examination. The first interval examination of this vessel in 1981 included 100% of the length of these welds with the exception of WR18.

ENICLOSURE 3 Weld Layout Sketch of Oconee Unit 1 **Reactor Vessel** Category B1.10 Welds WR54 (W-Axis) 100% VID Core Flood Nozzle WR54A (Y-Axis) 100% VID WR12 (W-X Axis) WR12A (X-Y Axis) WR19 - 100% WR12B (Y-Z Axis) WR12C (Z-W Axis) 100% VID Outlet WR18 - 95% Inlet. Nozzle Nozzle Bore 100% WR17 - 100% WR13 (X-Axis) WR13A (Z-Axis) WR2 (Z-W Axis) WR2A (X-W Axis) 100% VID -100% 100% --WR1,4 - 100% WR2 (X-Y Axis) WR2A (Z-W Axis) 100% -100% × -WR1A - 100% WR2 (X-Y Axis) WR2A (ZW Axis) - 100% 100% -WR34 - All accessible areas WR35 - None

Gray Indicates B1.10 Category Welds

## **Technology Enhancements**

## BACKGROUND

- B&W and the B&W Owners Group (BWOG) have conducted several programs to evaluate and improve examination capability for flaw detection and sizing for reactor vessels.
- . Development efforts utilized EPRI and BWOQ test blocks:
  - Near Surface Blocks (EPRI) 45 flaws
  - Heavy Section Blocks (EPRI)
    34 flaws
  - Reactor Nozzle Mock-Up (BWOG)
    20 flaws
- · Programs evaluated UT parameters:
  - Transducer designs
    - Beam Spread Frequency Element Size Element Arrangements
  - Scanning Sensitivity Recording Threshold Signal Processing Techniques Sizing Methods

### **Technology Enhancements**

## ALTERNATE EXAMINATION METHODS

#### REACTOR COOLANT NOZZLE TO PIPE WELDS

Section XI requires a surface examination of the O. D. surface. UT from the I. D. is proposed to satisfy this requirement.

Detection capability has been demonstrated on axial and circumferential notches in a pipe section. Minimum detectable depth determined to be 0.034".

This is the same technique used to satisfy previous relief request from Toledo Edison for the Davis Besse Plant.

#### CORE FLOOD NOZZLE TO SAFE END AND SAFE END TO PIPE

Section XI requires a surface examination of the O. D. surface. UT from the I. D. is proposed to satisfy this requirement.

Detection capability has been demonstrated on axial and circumferential notches in a pipe section. Minimum detectable depth determined to be 0.030".

This is the same technique used to satisfy previous relief request from Toledo Edison for the Davis Besse Plant.

## **Technology Enhancement**

## IMPACT OF PERFORMING AN ENHANCED EXAMINATION

## TRANSDUCER REQUIREMENTS

- Contact Method
- Dual Element
- Low Frequency
- 50% Bandwidth

#### SCANNING REQUIREMENTS

- 25% Transducer Overlap (.56" index)
- · 20 dB (MIN) Gain Increase over ASME Calibration
- · Recording Threshold set between 5 and 10 % DAC

## ANALYSIS

#### VOLUMETRIC INDICATIONS

Detection data is evaluated to determine acceptability to IWB-3500. Any indication exceeding IWB-3500 is rescanned at .2" increments for better characterization.

#### LAMINAR INDICATIONS

Detection data is evaluated to determine acceptability to IWB-3500. Any indication exceeding IWB-3500 is rescanned at .2" increments for better characterization.

#### PLANAR INDICATIONS

Detection data is evaluated. If a planar flaw is detected, it is rescanned with characterization transducers for tip diffraction techniques.

## **Technology Enhancement**

## RESULTS OF PROGRAMS

#### DETECTION

- Near surface flaw detection rate = 98% (1986)
- Balance of volume flaw detection rate = 100%
- · High confidence exists for flaw detection
- · Techniques used would probably meet Appendix VIII criteria.

#### SIZING

- Tip diffraction techniques more accurate than amplitude based sizing prescribed by ASME Section XI for planar flaws.
- Volumetric flaws are the most difficult to accurately size because time based methods are not generally possible.

July 26, 1991

DISTRIBUTION: Docket File NRC & Local PDRS FMiraglia, 12G18 JPartlow, 12G18 SVarga GLainas PDII-3 R/F Oconee R/F FRinaldi DMatthews LBerry OGC, 15B18 EJordan, MNBB3701 ACRS, P-315 JWechselberger, 17G21 JBlake, RII GJohnson, 7D4 JColey, RII DSmith, 7D4 CYCheng, 7D4 RHermann, 7D4

×