

NRC FORM 366
(12-81)
10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORT

APPROVED BY OMB
3150-0011

CONTROL BLOCK: [] [] [] [] [] [] [] [] [] [] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

[0][1] [P][A][S][E][S][1] [2] [0][0]-[0][0][0][0]-[0][0] [3] [4][1][1][1][1] [4] [] [] [5]
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CONT

[0][1] REPORT SOURCE [L] [6] [0][5][0][0][0][3][8][7] [7] [1][2][2][1][8][3] [8] [0][5][2][1][8][4] [9]
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

[0][2] Test performed by Teledyne, requested by PP&L, indicated the 3-way clamps on the
[0][3] CRD insert and withdrawal lines may not be able to withstand worst case axial
[0][4] water hammer loads. LCO per T.S. 3.1.2.1.C was entered to prevent rod movement in
[0][5] order to restrain loads on the lines. A review of Design/As-built information re-
[0][6] vealed additional supports outside containment requiring design drawing/hanger
[0][7] modifications.
[0][8]

[0][9] SYSTEM CODE [I][C] [11] CAUSE CODE [B] [12] CAUSE SUBCODE [A] [13] COMPONENT CODE [P][I][P][E][X][X] [14] COMP. SUBCODE [A] [15] VALVE SUBCODE [Z] [16]
7 8 9 10 11 12 13 14 15 16 17 18 19 20
[17] LER/NO REPORT NUMBER [8][3] [] [1][6][4] [] [0][1] [X] [] [2]
21 22 23 24 25 26 27 28 29 30 31 32
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER [26]
[X] [18] [X] [19] [Z] [20] [Z] [21] [0][0][0][0] [N] [23] [Y] [24] [A] [25] [0][0][6][8]
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

[1][0] Modification to CRD insert/withdrawal line supports in containment resulted from
[1][1] Teledyne analysis based on newly defined hydrodynamic loads. Modifications to de-
[1][2] sign drawings and supports outside containment were a result of changes in CRD in-
[1][3] stallers during Unit 1 construction; and re-analysis using newly defined hydrody-
[1][4] namic loads.

[1][5] FACILITY STATUS [G] [28] % POWER [0][0][0] [29] OTHER STATUS [NA] [30] METHOD OF DISCOVERY [C] [31] DISCOVERY DESCRIPTION [Waterhammer Testing] [32]
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1][6] ACTIVITY CONTENT RELEASED OF RELEASE [Z] [33] AMOUNT OF ACTIVITY [NA] [35] LOCATION OF RELEASE [NA] [36]
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1][7] PERSONNEL EXPOSURES NUMBER [0][0][0] [37] TYPE [Z] [38] DESCRIPTION [NA] [39]
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1][8] PERSONNEL INJURIES NUMBER [0][0][0] [40] DESCRIPTION [NA] [41]
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[1][9] LOSS OF OR DAMAGE TO FACILITY TYPE [Z] [42] DESCRIPTION [NA] [43]
7 8 9 10 11 12 13 14 15 16 17 18 19 20

[2][0] PUBLICITY ISSUED DESCRIPTION [N] [44] DESCRIPTION [NA] [45]
7 8 9 10 11 12 13 14 15 16 17 18 19 20

8405300324 840521
PDR ADOCK 05000387
S PDR

TEAD
VW

NAME OF PREPARER B.L. Wilks

PHONE (717) 542-3239

NRC USE ONLY

ATTACHMENT

LER # 83-164/01X-2

Pennsylvania Power & Light Company
Susquehanna Steam Electric Station
Docket Number: 50-387

Modification to supports on the Insert/Withdrawal Lines for the Control Rod Drive (CRD) System have been completed at Unit 1. In containment, these modifications required changing two way clamps to three way clamps at the CRD Insert/Withdrawal Line outer pedestal supports. Various other changes were required for insert/withdrawal line supports outside containment (such as individual hanger corrections, and as-built drawing corrections).

Modifications to supports for the CRD Insert/Withdrawal Lines, inside and outside of containment, conform to stresses that result from water hammer loads based on newly defined "Fast Scram" Hydrodynamic Loads on CRD systems for the Susquehanna Steam Electric Station.

Further information is being provided pursuant to 10CFR50.55e thru PLA-2118 "Final Report on Deficiency Involving Clamps on CRD Insert and Withdrawal Lines".



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

May 21, 1984

Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 83-164/01X-2
ER 100450 FILE 841-23
PLA-2212

Dear Dr. Murley:

Attached, please find a copy of Licensee Event Report No. 83-164/01X-2. This event was originally determined to be reportable per Technical Specification 6.9.1.8.1, in that clamps used to restrict movement of Control Rod Drive Insert and Withdrawal Lines were modified to conform to newly devised stress analysis for the system. Revision 2 of this Licensee Event Report provides additional information.

H.W. Keiser
Superintendent of Plant-SSES

BLW/pjg

Attachment

cc: Mr. R.H. Jacobs
Senior Resident Inspector
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