

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER & LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 162 License No. DPR-50

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al. (the licensee) dated March 25, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

The Technical Specifications contained in Appendix A, as revised through Amendment No. 162, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

 This license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Director Project Directorate I-4

Davision of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: May 20, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 162

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Remove	Insert
3-25 4-5a	3+25 4+5a

3.4 DECAY HEAT REMOVAL CAPABITATY

Applicability

Applies to the operating status of systems and components that function to remove decay heat when one or more fuel bundles are located in the reactor vessel.

Objective

To define the conditions necessary to assure continuous capability of decay heat removal.*

Specification

- 3.4.1 Reactor Coolant System temperature greater than 250°F.
- 3.4.1.1 With the Reactor Coolant temperature greater than 250°F, three independent EFW pumps and associated flow paths shall be OPERABLE** with:
 - a. Two EFW pumps, each capable of being powered from an OPERABLE emergency bus, and one EFW pump capable of being powered from an OPERABLE steam supply system.
 - (1) With one pump or flow path inoperable, restore the inoperable pump or flow path to OPERABLE status within 72 hours or be in COLD SHUTDOWN within the next 12 hours.
 - (2) With more than one EFW pump or flow path inoperable, restore the inoperable pumps or flow paths to OPERABLE status or be subcritical within 1 hour, in at least HOT SHUTDOWN within the next 6 hours, and in COLD SHUTDOWN within the following 6 hours.
 - b. Four of the six turbine bypass valves OPERABLE. With more than two turbine bypass valves inoperable, restore operability of at least four turbine bypass valves within 72 hours.
 - c. The condensate storage tanks (CSTs) OPERABLE with a minimum of 150,000 gallons of condensate available in each CST.
 - (1) With a CST inoperable, restore the CST to operability within 72 hours or be in at least HOT SHUTDOWN within the next 6 hours, and COLD SHUTDOWN within the next 30 hours.
 - (2) With more than one CST inoperable, restore the inoperable CST to OPERABLE status or be subcritical within 1 hour, in at least HOT SHUTDOWN within the next 6 hours, and in COLD SHUTDOWN within the following 6 hours.

^{*}These requirements supplement the requirements of Sections 3.1.1.1.c, 3.1.1.2, 3.3.1, and 3.8.3.

^{**}HSPS operability is specified in Section 3.5.1.

			TABLE 4.	1-1 (Continued)	
200	CHANNEL DESCRIPTION	CHECK	TEST	CALIBRATE	REMARKS
28.	Radiation Monitoring Systems*	W(1)(3)	M(3)	Q(2)	(1) Using the installed check source when background is less than twice the expected increase in cpm which would result from the check source alone. Background reading greater than this value are sufficient in themselves to show that the monitor is functioning.
0 40 40 40 00					(?) Except area gamma radiation monitors RM-G5, RM-G6, RM-G7 and RM-G21 which are located in the Reactor Building. When purging is permitted per T.S. 3.6, RM-G5 and RM-G21 will be calibrated quarterly. If purging is not permitted per T.S. 3.6, RM-G5 and RM-G21 shall be calibrated at the next scheduled reactor shutdown following the quarter in which calibration would normally be due. Rid-G6 and RM-G7, which are in high radiation areas shall be calibrated at the next scheduled reactor shutdown following the quarter in which calibration is due, if a shutdown during the quarter does not occur. (3) Surveillances are required to be performed only when containment integrity is required This applies to monitors which initiate containment isolation only.

29. High and Low pressure Injection Systems: Flow Channels

N/A N/A

*Does not include the monitors covered under Specification 3.5.5.2 and 4.1.3 or Specifications 3.21.1, 3.21.2 and 4.21.1, 4.21.2.

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