| NRC For<br>(9-83)                            | m 366                      |  |  |  | LIC   | ENSEE   | EVEN  | T RE  | PORT  | (LER)  |  | U.S. NUC<br>API<br>EXI  | LEAR REGUI   | NO  | 3150-01   | AISSION<br>04                |  |  |
|--|----------------------------|--|--|--|---|---|---|---|---|--|--|---|--|---|---|------------------------------|--|--|
| FACILIT                                      | -                          | 1)   |  |  |   |   |   |   |   | 1  | OCKET N  | UMBER (   | 2)   |   | PAC   | SE (3)                       |  |  |
| CR   | YSTAI                      | L RIV  | ER UI  | NIT 3  |   |   |   |   |   |  | 0 15 1   | 0101  | 01310  | 12  | 1 OF  | 013                          |  |  |
| TITLE (4                                     | Dr.                        | 0000   | o In   | inction A  | ctuation  | Causo   | d Ry  | Rict  | ablo  | Malfuncti  | on   |   |  |   |   |                              |  |  |
| LC   | ENT DAT                    | essur  | e in   | JECTION A  | c cua c Ton   | REPO  | RT DATE   | m   | able  | OTHER  | FACILITI   | ES INVOLV   | (ED (8)  |   |   |                              |  |  |
| MONTH DAY YEAR YEAR SEQUENTIAL REVISION      |                            |  |  |  |   | MONTH DAY YEAR FACILITY N   |   |   |   |  | DOCKET NUMBER(S)   |   |  |   |   |                              |  |  |
|  |                            |  |  |  |   |   |   |   |   |  | 0 15 0 0 0 0   |   |  |   | 101   |                              |  |  |
| 0 4  | 2 4                        | 8 4  | 84   | - 0 0 8  | -00   | 0 5 2   | 2 4 8   | 8 4   | N/A   |  |  |   | 0   5   0  | 0   | 101   |                              |  |  |
| OPI  | RATING                     |  | THIS R   | EPORT IS SUBMITT   | ED PURSUANT   | TO THE REG  | UIREMEN   | NTS OF 10   | CFR §: 10   | Check one or more o  | if the follo   | owingi (11)   |  | _   |   |                              |  |  |
| POWE<br>LEVE<br>(10)                         |                            | 1<br>1910  | 20 20 20 20 20 20 20 20 20 20 20 20 20 2   | 0.402(b)<br>0.405(a)(1)(i)<br>0.405(a)(1)(ii)<br>0.405(a)(1)(iii)<br>0.405(a)(1)(iv)<br>0.405(a)(1)(iv)  |   | 20.405(c)<br>50.36(c)(1<br>50.36(c)(2<br>50.73(c)(2<br>50.73(c)(2<br>50.73(c)(2<br>50.73(c)(2                   | )<br>)<br>)(i)<br>)(ii)   |   | ×   | 50.73(a)(2)(iv)<br>50.73(a)(2)(v)<br>50.73(a)(2)(vii)<br>50.73(a)(2)(viii)(4<br>60.73(a)(2)(viii)(8<br>50.73(a)(2)(vii)(8            | U<br>D   |   | 73.71(6<br>73.71(6<br>0THER<br>below a<br>386A)  | )<br>(Spind in                              | icity in AD   | etract<br>C Form             |  |  |
|  |                            |  |  |  | L   | ICENSEE CO  | NTACT P   | OR THIS   | LER (12)  |  |  |   |  |   |   |                              |  |  |
| WAME<br>W.                                   | к. I                       | Bandh  | auer   | , Nuclear  | Safety  | Superv  | isor  |   |   |  | AREA<br>9  | CODE<br>0 4   | 71915  | 1-  | 16 14   | 18 16                        |  |  |
|  | -                          |  |  | COMPLET  | ONE LINE FOR  | EACH COM  | PONENT  | FAILURE   | DESCRIBE  | D IN THIS REPOR  | 7 (13)   |   | 1  | -   |   |                              |  |  |
| CAUSE  | E SYSTEM COMPONENT MANUFAC |  | TO NPRDS   | E  |   | CAUSE   | SYSTEM  | COMPONENT   | MANUFAC-<br>TURER   |  | TO NPROS   |   |  |   |   |                              |  |  |
| x  | JIE                        | Ĕ I I  | 151  | B 101410   | YES   |   |   |   |   |  | 111  |   |  |   |   |                              |  |  |
| 1  |                            |  |  |  | 14. C. C. C.  |   |   |   |   |  | 1  | 1.1   |  |   |   | *                            |  |  |
|  |                            |  |  | SUPPLEN  | IENTAL REPORT   | EXPECTED  | (14)  |   |   |  | -  | EXPECTED  | MO   | NTH   | DAY   | YEAR                         |  |  |
| YES UP VAL COMPLETE EXPECTED SUBMISSION DATE |                            |  |  |  | XI NG   |   |   |   |   | S  | DATE (15)  | N   |  |   |   |                              |  |  |
|  |                            | On A<br>Safeg<br>Surve<br>false<br>order<br>the r<br>React<br>action<br>minim<br>With<br>was of<br>Proce | pril<br>uards<br>illand<br>low p<br>to t<br>equir<br>tor C<br>n, app<br>nal.<br>the r<br>discove | 24, 1984<br>s System<br>ce Procedu<br>pressure al<br>est it, a la<br>ed two ou<br>coolant Sys<br>proximatel<br>reactor coo<br>vered in a<br>was satisf | at 1040,<br>Actuation<br>ire, "Engi<br>and high p<br>ow pressu<br>it of thro-<br>stem from<br>y 30 gall<br>plant press<br>tripped<br>actorily p | Cryst<br>n. The<br>neered<br>ressure<br>are bist<br>ee act<br>n the l<br>ons we<br>ssure a<br>condit<br>perform | tal R<br>safe<br>inje-<br>table<br>uatio<br>Borat<br>re inj<br>bove<br>ion.<br>ned. | tiver<br>t was<br>guard<br>ction<br>in Ch<br>n log<br>ed Wa<br>jected<br>setpo<br>The | Unit<br>at S<br>Is Mor<br>occur<br>iannel<br>ic. B<br>ater S<br>d, and<br>int, ti<br>bistal | 3 experie<br>7% react<br>of the react<br>red when,<br>2 inadver<br>forated wa<br>torage Ta<br>the effec<br>ne low pre-<br>ble was re | nced<br>or po<br>tiona<br>with<br>tently<br>ater<br>nk.<br>t on<br>ssure<br>eplace | a pa<br>ower<br>l Test<br>Char<br>y actu<br>was in<br>Due t<br>the pr<br>bista<br>ed. | rtial E<br>(865 M<br>" in pro-<br>nal 3 f<br>iated co<br>njected<br>o quick<br>rimary<br>ble of (<br>The Sur | ng<br>We<br>ogr<br>trij<br>om<br>ir<br>opla | ineere<br>e) wit<br>ess.<br>oped i<br>pletin<br>ito th<br>perato<br>ant wa<br>annel<br>eillance | d<br>hA<br>in georas<br>2 ce |  |  |
|  |                            | 8P.5   | DR A   | DOCK 05  | 000302<br>PDR   |   |   |   |   |  |  |   |  |   |   |                              |  |  |

NAC Form 366 (9-83)

.

|                   |     |       | CON       | ITINU     | OITAI         | N                       |                       | A)<br>8)                    | PPROVED O                   | MB NO. 3                        | 150-0104   |  |
|-------------------|-----|-------|-----------|-----------|---------------|-------------------------|-----------------------|-----------------------------|-----------------------------|---------------------------------|--|--|
| DOCKET NUMBER (2) |     |       |           |           |               | LE                      |                       | PAGE (3)                    |                             |                                 |  |  |
|                   |     |       |           |           | YEAR          | SEQUENTIAL RE NUMBER NU |                       |                             | REVISION                    |                                 | T  |  |
| 0 5               | 101 | 0 0   | 3         | 0 2       | 8 4           | -                       | 0 10 18               | 3 -                         | 010                         | 0 2                             | OF 0 1   |  |
| 654's/ (17)       | 0 5 | 0 5 0 | 0 5 0 0 0 | 0 5 0 0 3 | 0 5 0 0 3 0 2 | 0 5 0 0 3 0 2 8 4       | 0 5 0 0 0 3 0 2 8 4 - | 0 5 0 0 0 3 0 2 8 4 - 0 0 8 | 0 5 0 0 3 0 2 8 4 - 0 0 8 - | 0 5 0 0 3 0 2 8 4 - 0 0 8 - 0 0 | 0 5 0 0 0 3 0 2 8 4 - 0 0 8 - 0 0 0 2<br>0 5 0 0 0 0 3 0 2 8 4 - 0 0 8 - 0 0 0 2 |  |

#### TIME EVENT DESCRIPTION 09:30:00 Technicians commenced Channel Functional Test Surveillance Procedure on Engineered Safeguards System (JE). 10:30:00 Technicians placed "B" train Channel 3 of Engineered Safeguards in "test" which tripped Channel 3. Channel 2 Low Pressure Injection bistable failed in tripped condition. 10:40:15 This completed 2 out of 3 logic and actuated "B" train Low Pressure Injection (BP) and High Pressure Injection (BQ) Systems. 10:52:00 Operators stabilized the plant. 11:25:00 Technicians discovered failed Channel 2 Low Pressure Injection bistable. Failed bistable was replaced, and the replacement satisfactorily 15:00:00 tested.

## PLANT PERFORMANCE

SEQUENCE OF EVENTS

## 1. Pre-trip Review

Crystal River Unit 3 was operating at 97% reactor power while generating 865 MWe. All Engineered Safeguards equipment was operable with the exception of the "B" Decay Heat Pump Discharge Valve (BP, ISV) which was tagged out for modification. Technicians were performing a Channel Functional Test of the Engineered Safeguards System.

## 2. Initiating Event

Channel 3 of the "B" train of Engineered Safeguards was in the tripped condition due to the channel functional test. The Channel 2 Low Pressure Injection bistable failed to the tripped condition which completed the "2 out of 3" logic to actuate the "B" train of Low Pressure Injection and High Pressure Injection systems.

| NRC Form 366A<br>19-831  | APPROVED C<br>EXPIRES 8/3 | A REGULATORY COMMISSION<br>/ED OMB NO. 3150-0104<br>S. 8/31/85 |            |   |          |  |          |      |   |   |
|--|---------------------------|--|------------|---|----------|--|----------|------|---|---|
| FACILITY NAME (1)  | DOCKST NUMBER (2)         | 1  | LE         |   | UMBER (6 | 9  | PAGE (3) |      |   |   |
| CRYSTAL RIVER UNIT 3   |                           | YEAR   | SEQUENTIAL |   | UMBER    | REVISION<br>NUMBER   |          |      | T |   |
|  | 0 5 0 0 0 3 0 2           | 814  | -          | 0 | 8 0      | -010   | 01       | 3 OF | 0 | 3 |
| TEXT Iff more space is required, use additional NRC Form 366A's. | / (17)                    |  | -          |   |          | and the second s |          |      | - | - |

3. Plant Response

With the exception of the "B" Decay Heat Pump Discharge Valve (inoperable for modification), all affected Engineered Safeguards equipment actuated properly. Plant operators responded quickly, evaluated the actuation as false, terminated the Engineered Safeguards actuation, and stabilized the plant.

## SAFETY CONSIDERATIONS

## 1. Low Pressure Bistable Failure

This bistable failure did not render any portion of the Engineered Safeguards System inoperable as Low Pressure and High Pressure Injection Systems of the "B" train were placed in the actuated condition. Additionally, if this failure of Channel 2 occurred without another channel in test, either Channel 1 or 3 would have automatically actuated Engineered Safeguards, if required.

# 2. "B" Decay Heat Pump Discharge Valve

This valve was intentionally disabled for modification. The "A" Decay Heat train was available throughout this event if Low Pressure Injection was actually required.

#### CORRECTIVE ACTIONS

During an inspection of the Engineered Safeguards instrumentation following the event, technicians discovered the failure of the bistable by noting improper status lights on the bistable. The Low Pressure Injection bistable was replaced. The replacement bistable was tested satisfactorily.

The failure was not reproducible during a bench test of the failed bistable. This type of bistable uses mercury-wetted relays. Because any physical movement of these relays tends to recoat the contacts, it makes it difficult to reproduce this type of failure.

#### PREVIOUS SIMILAR EVENTS

There has been one prior occurrence of an event similar to this one. It occurred on March 13, 1984 and was reported as LER 84-006. The distinct possibility exists that the bistable which caused this failure was also the component which initiated that event.





May 24, 1984 3F0584-12

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555

Subject: Crystal River Unit 3 Docket No. 50-302 Operating License No. DPR-72 Licensee Event Report No. 84-008-00

Dear Sir:

Enclosed is Licensee Event Report (LER) No. 84-008-00 which is submitted in accordance with IOCFR50.73.

Should there be any questions, please contact this office.

Sincerely,

agnard al P. Y. Baynard

Assistant to Vice President Nuclear Operations

AEF/feb

Enclosure

cc: Mr. James P. O'Reilly Regional Administrator, Region II Office of Inspection & Enforcement U.S. Nuclear Regulatory Commission IOI Marietta Street N.W., Suite 2900 Atlanta, GA 30303

TEL