	CONTROL BLOCK: 1_1_1_1_1_1_1_1 (PLEASE PRINT OR TYPE	ALL REQUIRED INFORMATION)
$\frac{10}{7} \frac{11}{8}$	9 LICENSEE CODE 14 15 101-10101010101-1010101-101013 1 LICENSEE CODE 14 15 101-1010101010101-1010125 25	4 1 1 1 1 1 1 1 1 1 1 1 4 1 1 15 6 LICENSE TYPE 30 57 CAT 58
10118	REPORT I L I O I O I <td>1 2 18 1 0 1 5 1 2 1 5 1 8 1 4 19 74 75 REPORT DATE 80</td>	1 2 18 1 0 1 5 1 2 1 5 1 8 1 4 19 74 75 REPORT DATE 80
10121	EVENT DESCRIPTION AND PROBABLY CONSEQUENCES 10 10n 12/10/82 during power ascension testing at 80% full power, it was determi	
10131	lannealing matrix for all four core protection calculators (CPC) had been imp	
10141	lof an incorrect shape annealing matrix caused the CPC channels to calculate	a more bottom peaked core power
10151	idistribution than actual as calculated with the incore detector system. The	
10161	Iconservative DNBR and LPD calculations by all four CPC channels. This occur	
10171	1 <u>6.9.1.8.f.</u>	
10181	9	
	SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE 1 <th>19 20</th>	19 20
17 R	REPORT I B I <thi< th=""> I <thi< th=""> <thi< th=""></thi<></thi<></thi<>	TYPE NO
TA	$ \begin{array}{c ccccccccccccccccccccccccccccccccccc$	SUB SUPPLIER MANUFACTURER
11101	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27 The cause of this occurrence was personnel error. At the 50% full power asce	ension test plateau nuclear t
	lengineering personnel determined shape annealing matrices (SAM) for each CPC	
11121	Itransposed and input incorrectly into each CPC channel. The cause of the tra	
11131	I ing of the individual matrix elements as listed by a SAM analysis code. The	
$\frac{1}{7}$ $\frac{1}{4}$ $\frac{1}{8}$	lwithin 1 hour of identification of the error. This item was previously report	
$\frac{1}{7}\frac{1}{1}\frac{5}{8}$	FACILITY METHOD OF STATUS % POWER OTHER STATUS DISCOVERY DIS 1 10 12 13 130 1 Nuclear En 9 10 12 13 44 45 46	COVERY DESCRIPTION gineer Observation [32 80
R	ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY	OCATION OF RELEASE
$\frac{1}{7}$ $\frac{1}{1}$ $\frac{1}{6}$ $\frac{1}{8}$	9 10 11 135 1	NA 36 80
	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 1 0 1 0 137 1 2 138 1 NA 9 11 12 13	139
	RSONNEL INJURIES NUMBER DESCRIPTION	80
$\frac{1}{7} \frac{1}{8} \frac{1}{8}$	9 1 0 1 0 140 1 NA 9 11 12	
	LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 1_Z_142 1NA	80
	PUBLICITY	143
	ISSUED DESCRIPTION <u>1 N</u> 144 <u>1 NA</u> 9 10	NRC USE ONLY
1	NAME OF PREPARER Patrick Rogers	PHONE: (501) 964-3100
		1110116. (001) 304 3100

8405300187 840525 PDR ADDCK 05000368 S PDR

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IEAR

LER No. 50-368/82-042/01X-1

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Cause Description and Corrective Actions (cont'd):

from AF&L (Levine) to NRC (Collins) dated December 10, 1982. The computer code utilized to calculate the shape annealing matrix was modified to provide individual labels for each matrix element.



ARKANSAS POWER & LIGHT COMPANY POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

May 25, 1984

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U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

> Subject: Arkansas Nuclear One - Unit 2 Docket No. 50-368 License No. NPF-6 Licensee Event Report No. 82-042/01X-1

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 2 Technical Specification 6.9.1.8.f, attached is the subject report concerning incorrect loading of shape annealing matrices (SAM) into the core protection calculators (CPC). This is an update to a previous submittal dated December 21, 1982.

Very truly yours, favan

John R. Marshall Manager, Licensing

JRM: RJS: ac

Attachment

cc: Mr. Richard P. Denise, Director Division of Resident Reactor Projects and Engineering Programs U. S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, TX 76011