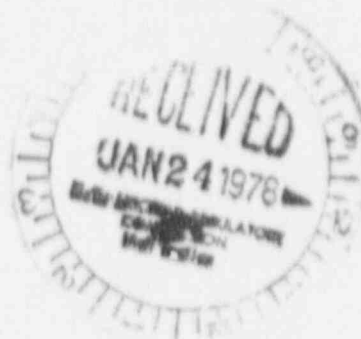


NSP

REGULATORY DOCKET FILE COPY

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401



January 19, 1978

Mr J G Keppler, Director, Region III
Office of Inspection & Enforcement
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Mr Keppler:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 263 License No. DPR-22

Nonconforming Bolts in Radwaste Shipping Cask

The Licensee Event Report for this occurrence is reproduced on the back of this letter. Enclosed are three copies.

This event is reported in compliance with Technical Specification 6.7.B.1.i, components that require corrective measures to prevent operation in a manner less conservative than assumed in the accident analysis.

Yours very truly,

L O Mayer, PE
Manager of Nuclear Support Services

LOM/MEIV/ak

cc: Director, IE, USNRC (40)
-Director, MIPC, USNRC (3)
MPCA: Attn: J W Ferman

780250005

A002/S
0/1
HEAD TO
HEAD

9105230321 780119
PDR ADOCK 05000263
S PDR

LICENSEE EVENT REPORT

CONTROL BLOCK _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | M | N | M | N | P | 1 | 2 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 4 | 1 | 1 | 1 | 1 | 1 | 4 | _____ (9)

CONT
0 1 | REPORT SOURCE 1 | 6 | 0 | 5 | 1 | 0 | 1 | 0 | 1 | 0 | 2 | 6 | 3 | 7 | 0 | 1 | 1 | 0 | 5 | 7 | 8 | 8 | 0 | 1 | 3 | 1 | 3 | 9 | 7 | 1 | 8 | _____ (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | WHILE PREPARING TO SHIP RADIOACTIVE WASTE IT WAS DISCOVERED THAT ONE OF THE TWO _____
0 3 | SHIPPING CASKS USED TO TRANSPORT IRRADIATED COMPONENTS WAS PROVIDED WITH 1 INCH _____
0 4 | DIAMETER LID BOLTS RATHER THAN THE 1 1/4 INCH SIZE AS REQUIRED. BOLT SIZE IS _____
0 5 | LESS CONSERVATIVE THAN REQUIRED BY SAFETY ANALYSIS (T.S. 6.7.B.1.i). NO SIMILAR _____
0 6 | EVENTS. NO EFFECT ON THE HEALTH AND SAFETY OF THE PUBLIC. MARGIN OF SAFETY FOR _____
0 7 | BOLTING REDUCED. _____

0 8 | _____ (9)

0 9 | SYSTEM CODE M | D | 11 | CAUSE CODE R | 12 | CAUSE SUBCODE B | 13 | COMPONENT CODE X | X | X | X | X | X | X | 14 | COMP. FL. CODE 2 | 15 | VALVE SUBCODE 2 | 16 |

17 | LER RD REPORT NUMBER 7 | 8 | 21 | EVENT YEAR 7 | 8 | 22 | SEQUENTIAL REPORT NO. 0 | 0 | 1 | 24 | OCCURRENCE CODE 0 | 1 | 26 | REPORT TYPE T | 30 | REVISION NO. 0 | 32 |

ACTION TAKEN X | 18 | G | 19 | EFFECT ON PLANT 2 | 20 | SHUTDOWN METHOD 2 | 21 | HOURS 0 | 0 | 0 | 0 | 0 | 22 | ATTACHMENT SUBMITTED N | 23 | NRD-4 FORM SUB N | 24 | PRIME COMP SUPPLIER X | 25 | COMPONENT MANUFACTURER N | 2 | 7 | 5 | 26 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | CAUSE APPARENTLY DUE TO ERROR BY CASK SUPPLIER. NOTIFIED CASK SUPPLIER OF PROBLEM _____
1 1 | AND RETURNED CASK. NUCLEAR ENGINEERING CO., INC., CASK MODEL B3-1. NRC CERTIFICATE _____
1 2 | OF COMPLIANCE 6058. CASK HANDLING PROCEDURE WILL BE REVISED TO INCLUDE VERIFICA - _____
1 3 | TION OF PROPER BOLT SIZE. _____

1 4 | _____ (9)

1 5 | FACILITY STATUS E | 28 | N POWER 1 | 0 | 0 | 29 | OTHER STATUS NA | 30 | METHOD OF DISCOVERY C | 31 | DISCOVERY DESCRIPTION ENGINEER OBSERVATION | 32 |

1 6 | ACTIVITY CONTENT 2 | 33 | AMOUNT OF ACTIVITY 2 | 34 | LOCATION OF RELEASE NA | 36 |

1 7 | PERSONNEL EXPOSURES NUMBER 0 | 0 | 0 | 37 | TYPE 2 | 38 | DESCRIPTION NA | 39 |

1 8 | PERSONNEL INJURIES NUMBER 0 | 0 | 0 | 40 | DESCRIPTION NA | 41 |

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE 2 | 42 | DESCRIPTION NA | 43 |

2 0 | PUBLICITY ISSUED NA | 44 | DESCRIPTION NA | 45 |