

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 1	PAGE (3) 1 OF 0 2
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TITLE (4)
REACTOR TRIP BREAKER SERIAL NO. 18 UNDERVOLTAGE DEVICE ANOMALY

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 4	2 3	8 4	8 4	0 2 5	0 0	0 5	2 3	8 4			0 5 0 0 0

OPERATING MODE (9) 1

POWER LEVEL (10) 1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	Informational Report
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: J. G. HAYNES, STATION MANAGER

TELEPHONE NUMBER: 7 1 4 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This report is submitted to provide information concerning operation of Reactor Trip Breakers (RTB's) on their undervoltage (UV) trip devices. (As in the past, the breakers continue to function acceptably using the shunt trip device.) Although this occurrence was determined to be not reportable under the Unit 2 Technical Specifications or 10 CFR 50.73, we are submitting this report to inform you of the circumstances involved and corrective actions taken.

As discussed in Licensee Event Report (LER) 2-83-153, Revision 1, RTB Serial No. 256A4002-656-18 was returned to service as RTB #3 on February 12, 1984. However, this RTB was removed from service on March 23, 1984 and replaced with spare breaker RTB Serial No. AKN 6325600002. RTB Serial No. 256A4002-656-18 was designated for use as a spare in the nonsafety-related cross-tie position (RTB #9) only. This allowed continued evaluation under installed conditions.

On April 23, 1983, with Unit 2 in Mode 1 at 100% power and surveillance testing in progress, the UV trip device for RTB Serial No. 256A4002-656-18 exhibited procedurally unacceptable response times. The RTB was tagged as not acceptable for use in any RTB position.

The health and safety of plant personnel or the public were not affected by this occurrence.

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PDR ADOCK 05000361
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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQ. NUMBER	REV. NUMBER			
		8 4	- 0 2 5	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This report is submitted to provide information concerning operation of Reactor Trip Breakers (RTB's) (EIS Component Code 52) on their undervoltage (UV) trip devices. (As in the past, the breakers continue to function acceptably using the shunt trip device.) Although this occurrence was determined to be not reportable under the Unit 2 Technical Specifications or 10 CFR 50.73, we are submitting this report to inform you of the circumstances involved and corrective actions taken.

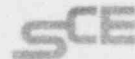
As discussed in Licensee Event Report (LER) 2-83-153, Revision 1, RTB Serial No. 256A4002-656-18, which had been previously designated as a spare, was returned to service as RTB #3 on February 12, 1984. It was returned to service based on enhanced trip times as a result of installation of metal oxide varistors (MOV's) (EIS Component Code 70) in the RTB cubicles and the fact that no other problems had been encountered since its designation as a spare. However, this RTB was removed from service on March 23, 1984 and replaced with spare breaker RTB Serial No. AKN6325600002. The decision to remove RTB Serial No. 256A4002-656-18 from service in safety-related breaker positions was based on an evaluation of trending data and previous anomalies exhibited by the breaker. In order to continue to investigate RTB performance, it was designated for use as a spare for the nonsafety-related cross-tie position (RTB #9) and maintained in the normal surveillance and maintenance interval.

On April 23, 1983, with Unit 2 in Mode 1 at 100% power, surveillance testing in accordance with S023-II-11.161, "Reactor Breaker Undervoltage Response Time Testing," prior to the scheduled RTB maintenance, was in progress. During this surveillance, the UV trip device for RTB Serial No. 256A4002-656-18 exhibited procedurally unacceptable response times. The response times (in order) were 58 msec, 114 msec and 94 msec. S023-II-11.161 contains an acceptance criterion of 82 msec, which was developed from baseline testing and consideration of the Combustion Engineering (CE) guideline of 100 msec. The shunt trip feature operated properly. No Action Statements were entered since the RTB was serving as a designated spare. The Nonconformance Report (NCR) documenting this condition, rejects the RTB for further use in any RTB position.

RTB Serial No. 256A4002-656-18 as well as RTB Serial No. 256A4002-656-26 (see LER 2-83-153, Revision 1) will be returned to the vendor (General Electric) for refurbishment. This refurbishment will include replacement of bearings. Upon completion of the refurbishment, these breakers are expected to be returned and maintained in the normal surveillance and maintenance interval. The performance of these breakers will be closely monitored. Based on the results of surveillance and maintenance performed on the breakers, their return to service will be considered.

Public health and safety were not affected since the breaker was designated for use as a spare for the nonsafety-related cross-tie position.

Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

TELEPHONE
(714) 492-7700

May 23, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
Informational Report
Licensee Event Report No. 84-025
San Onofre Nuclear Generating Station, Unit 2

Reference: Letter, J. G. Haynes (SCE) to J. B. Martin (NRC),
"Informational Report Licensee Event Report No. 83-153,
Revision 1," dated March 7, 1984

This submittal provides an informational Licensee Event Report (LER) for an occurrence involving the Plant Protection System (PPS). The health and safety of plant personnel or the public were not affected by this occurrence.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER 84-025

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

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