

Docket No. 50-263

JAN 5 1977

Northern States Power Company
ATTN: Mr. L. O. Mayer, Manager
Nuclear Support Services
414 Nicollet Mall - Eighth Floor
Minneapolis, Minnesota 55401

Gentlemen:

RE: MONTICELLO UNIT NO. 1

We have noted a potential deficiency in the method being used to confirm valve operability during periodic testing of Boiling Water Reactor (BWR) safety-relief valves. This deficiency concerns the use of the safety-relief valve temperature indication as a positive method of confirmation that a safety-relief valve is open when manually actuated during surveillance testing.

We have found that an increased temperature indication may be obtained at the safety-relief valve exit with the safety-relief valve closed. This indicated temperature increase is the result of steam vented through the valve actuation mechanism during the surveillance test. In view of this finding, we have concluded that a temperature increase at the valve exit, by itself does not provide a positive means of verification that the safety-relief valve has opened.

We have reviewed your Technical Specifications and have determined that they do not provide adequate surveillance requirements to assure safety-relief valve operability. Therefore, we require that you eliminate this potential deficiency in your surveillance testing program. You should propose a change to your technical specifications that provides a positive means of verification that the safety-relief valve opens when tested as part of the Automatic Depressurization System. An example of an acceptable method would involve the manual actuation of each individual safety-relief valve at one or more specified pressures and observing a specified minimum compensating turbine bypass or throttle valve closure or change in measured steam flow rate.

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Please submit your proposed Technical Specification changes within 60 days of your receipt of this letter. Your submittal should include the basis for your proposal and in particular should address the adequacy of the method proposed to positively assess the opening of safety-relief valves.

Sincerely,

Original signed by
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

cc:
See next page

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