

Tennessee Valley Authority, Fold Office Box (2000, Solidy Diany, Terminage, 107376)

J. L. Wilson Vice President Sequoyah Nuclear Francisco

March 16, 1992

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Gentlemen:

In the Matter of Tennessee Valley Authority Docket Nos. 50-327 50-328

20.0

SEQUOYAH NUCLEAR PLANT (SQN) - FEBRUARY 1992 MONTHLY OPERATING REPORT

Enclosed is the February 1992 Monthly Operating Report as required by SQN Technical Specification 6.9.1.10.

If you have any questions oncerning this matter, please call M. A. Cooper at (615) 843-8924.

Sincerely,

L. Wilson

Enclosure cc: See page 2

GKOH!

U.S. Nuclear Regulatory Commission Page 2 March 16, 1992

cc (Enclosure):

INPO Records Center Institute of Nuclear Power Operations 1100 Circle 75 Parkway, Suite 1500 Atlanta, Georgia 30389

Mr. D. E. LaBarge, Project Manager U.S. Nuclear Regulatory Commission One White Flint, North 11555 Rockville Pike Rockville, Maryland 2085.

Mr. Ted Marston, Director Electric Power Research Institute P.O. Box 10412 Palo Alto, California 94304

NRC Resident Inspector Sequoyah Nuclear Plant 2600 Igou Ferry Road Soddy-Daisy, Tennessee 37379

Regional Administration U.S. Nuclear Regulatory Commission Office of Inspection and Enforcement Region II 101 Marietta Street, NW, Suite 2900 Atlanta, Georgia 30323

Mr. B. A. Wilson, Project Chief U.S. Nuclear Regulatory Commission Region II 101 Marietta Street, NW, Suite 2900 Atlanta, Georgia 30323

Mr. F. Yost, Director Research Services Utility Data Institute 1700 K Street, NW, Suite 400 Washington, D.C. 20006

# TENNESSEE VALLEY AUTHORITY

NUCLEAR POWER GROUP SEQUOLAH NUCLEAR PLANT

MONTHLY OPERATING REPORT

TO THE

NUCLEAR REGULATORY COMMISSION

FEBRUARY 1992

UNIT 1

DOCKET NUMBER 50-327

LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

### OPERATIONAL SUMMARY FEBRUARY 1992

### UNIT 1

Unit I generated 690,550 megawathours (MWh) (gross) electrical power during February with a capacity factor of 85.38 percent. Unit I was operating at 100 percent reactor power level on February 13 at 0113 Eastern standard time (EST), when a power level decrease to 70 percent power level was initiated to allow repair of the Number 3 heater drain tank Level Control Valve (LCV) LCV-6-106B; it appeared that the plug had separated from the stem. At 2348 (EST) on February 13, a further decrease in reactor power level was initiated as a result of chemistry requirements being exceeded on steam generator blow down because of a condenser tube leak. On February 14 at 0222 (EST), Unit I power level reduction was terminated at 30 percent reactor power.

On February 18 at 0026 (EST), a power level increase was initiated. Unit 1 was operating at 100 percent reactor power level at 1635 (EST) on February 21.

On February 27 at 1725 (EST), Unit 1 reactor power level was decreased to approximately 82 percent to allow work on Number 7 heater drain tank LCV-6-109A. On February 28 at 0812 (EST), an additional power level decrease to 80 percent was initiated to remove Condensate Booster Pump (CEF) 1A from service to repair a casing vent valve leak. On February 28 at 205 (2ST), reactor power level increase was initiated. Unit 1 was operating at 100 percent reactor power on February 29 at 0400 (EST), and continued to operate at 100 percent through the end of February.

### UNIT 2

Unit 2 generated 628,470 MWh (gross) electrical power during February with a capacity factor of 77.71 percent. Unit 2 was operating at approximately 96 percent reactor power level at the beginning of February and was in coastdown to the Unit 2 Cycle 5 refueling outage. On February 10 at 0528 (EST), with Unit 2 operating at 89 percent reactor power level, a reactor trip occurred when the turbine tripped as a result of low auto stop oil pressure. The unit had been operating with no abnormal indications before the trip. Unit 2 had received a reactor trip "P-9 Turbine Trip/Reactor Trip." The cause of the event was determined to be the failure of the autostop trip solenoid valve on the turbine trip block.

Unit 2 was returned critical at 1329 (EST) on February 11 and entered Mode 1 at 2015 (EST). Unit 2 tied online on February 12 at 0219 (EST). On February 19 at 1547 (EST), Unit 2 reactor power level was at 87 percent. Unit 2 continued the coastdown to the Unit 2 Cycle 5 refueling outage. Unit 2 was operating at approximately 79 percent reactor power level at the end of February.

### POWER-OPERATED RELIEF VALVES (PORV) AND SAFETY VALVES SUMMARY

There were no challenges to PORVs or safety valves in February.

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327 UNIT No. One DATE: 03-05-92

COMPLETED BY: T. J. Hollomon TELEPHONE: (615) 843-7528

MONTH: FEBRUARY 1992

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1149	1.7	271
1148	18	622
1148	19	730
1148	20	732
1148	21	1037
1147	22	1138
1147	23	1142
1147	24	1142
1147	25	1131
1144	26	1141
1144	2.7	1103
1137	28	941
825	29	1140
303	30	N/A
283	31	N/A
287		

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328 UNIT No. Two DATE: 03-05-92

COMPLETED BY: T. J. Hollomon TELEPHONE: (615) 843-7528

MONTH: FEBRUARY 1992

X	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
	1082	17	935
	1075	18	962
	1069	19	972
	1060	20	966
	1052	21	967
	1045	22	955
	1034	23	946
	1021	24	938
	1019	25	933
	257	26	928
	-33	27	922
	210	28	907
	564	29	898
	769	30	N/A
	825	31	N/A
	919		

# OPERATING DATA REPORT

DOCKET NO. 50-327
DATE Mar. 5. 1992
TELEPHONE (615) 843-/528

Unit Name: Sequoyah Unit One Reporting Period: February 1992 Licensed Thermal Power (MWt): 3411.0 Nameplate Rating (Gross MWe): 1220.6 Design Electrical Rating (Net MWe): 1148. Maximum Dependable Capacity (Gross MWe): Maximum Dependable Capacity (Net MWe): 11 Maximum Dependable Capacity (Net MWe): 11 Maximum Dependable Capacity (Ratings (Item	0	Notes	rt, Give Reasons
9. Power Level To Which Restricted, If Any ( 10. Reasons for Restrictions, If Any: N			Cumulative
11. Hours in Reporting Period 12. Number of Hours Reactor Was Critical 13. Reactor Reserve Shutdown Hours 14. Hours Generator On-Line 15. Unit Reserve Shutdown Hours 16. Gross Thermal Energy Generated (MWH) 17. Gross Electrical Energy Generated (MWH) 18. Net Electrical Energy Generated (MWH) 19. Unit Service Factor 20. Unit Availability Factor 21. Unit Capacity Factor (Using MDC Net) 22. Unit Capacity Factor (Using DER Net) 23. Unit Forced Outage Rate 24. Shutdowns Scheduled Over Next 6 Months	100.0 100.0 84.7 82.8	1,440 1,440.0 0 1,440.0 0 4,533,376.7 1,565.188 1,507.633 100.0 100.0 93.3 91.2 0.0	93,505 48,394.0 0 47,311.1 0 154,145,910 52,232,684 50,072,367 50.5 50.6 47,7 46.6 40.7

<sup>25.</sup> If Shut Down At End Of Report Period, Estimated Date of Startup:

# OPERATING DATA REPORT

DOCKET NO. 50-328

DATE Mar. 5, 1992

COMPLETED BY I. J. Hollomon

TELEPHONE (615) 843-7528

		Notes						
. Unit Name: Sequoyah Unit Two	Increa							
. Reporting Period: February 1992								
. Licensed Thermal Power (MWt): 3411.0								
. Nameplate Rating (Gross MWe): 1220.6								
. Design Electrical Rating (Net MWe): 114								
. Maximum Dependable Capacity (Gross MWe):								
. Maximum Dependable Capacity (Net MWe): _								
. If Changes Occur in Capacity Ratings (Item Numbers 3 Through 7) Since Last Report, Give Reaso								
. Power Level To Which Restricted, If Any O. Reasons For Restrictions, If Any:		Α						
	This Month	Yr-to-Date	Cumulative					
1. Hours in Reporting Period	This Month							
		Yr-to-Date	Cumulative 85,465 50,416					
2. Number of Hours Reactor Was Critical	696	1,440	85,465					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours	696 664.0	1,440	85,465 50,416 0					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 1. Hours Generator On-Line	696 664.0	1,440 1,408.0 0	85,465 50,416					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 1. Hours Generator On-Line 5. Unit Reserve Shutdown Hours	696 664.0 0 651.2	1,440 1,408.0 0 1,395.2	85,465 50,416 0 49,438,4					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 1. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 5. Gross Thermal Energy Generated (MWH)	696 664.0 0 651.2	1,440 1,408.0 0 1,395.2	85,465 50,416 0 49,438.4					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 4. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 6. Gross Thermal Energy Generated (MWH) 7. Gross Electrical Energy Generated (MWH) 8. Net Electrical Energy Generated (MWH)	696 664.0 0 651,2 0.0 1,840,863.3	1,440 1,408.0 0 1,395.2 0 4,367,657.4	85,465 50,416 0 49,438,4 0 155,294,970					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 4. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 6. Gross Thermal Energy Generated (MWH) 7. Gross Electrical Energy Generated (MWH) 8. Net Electrical Energy Generated (MWH) 9. Unit Service Factor	696 664.0 0 651.2 0.0 1.840.863.3 628,470	1,440 1,408.0 0 1,395.2 0 4,367,657.4 1,491,156 1,436,861 96.9	85,465 50,416 0 49,438,4 0 155,294,970 52,649,447 50,381,825					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 4. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 6. Gross Thermal Energy Generated (MWH) 7. Gross Electrical Energy Generated (MWH) 8. Net Electrical Energy Generated (MWH) 9. Unit Service Factor 1. Unit Availability Factor	696 664.0 0 651.2 0.0 1.840.863.3 628.470 603,339	1,440 1,408.0 0 1,395.2 0 4,367,657.4 1,491.156	85,465 50,416 0 49,438,4 0 155,294,970 52,649,447					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 4. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 6. Gross Thermal Energy Generated (MWH) 7. Gross Electrical Energy Generated (MWH) 8. Net Electrical Energy Generated (MWH) 9. Unit Service Factor 1. Unit Capacity Factor (Using MDC Net)	696 664.0 0 651,2 0.0 1,840,863.3 628,470 603,339 93.6 93.6	1,440 1,408.0 0 1,395.2 0 4,367,657.4 1,491,156 1,436,861 96.9 96.9 96.9	85,465 50,416 0 49,438,4 0 155,294,970 52,649,447 50,381,825 57,8					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 4. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 6. Gross Thermal Energy Generated (MWH) 7. Gross Electrical Energy Generated (MWH) 8. Net Electrical Energy Generated (MWH) 9. Unit Service Factor 1. Unit Availability Factor 1. Unit Capacity Factor (Using MDC Net) 2. Unit Capacity Factor (Using DER Net)	696 664.0 0 651,2 0.0 1,840,863.3 628,470 603,339 93.6 93.6 77.3 75.5	1,440 1,408.0 0 1,395.2 0 4,367,657.4 1,491,156 1,436,861 96.9 96.9 88.9 88.9	85,465 50,416 0 49,438,4 0 155,294,970 52,649,447 50,381,825 57,8					
2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 4. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 6. Gross Thermal Energy Generated (MWH) 7. Gross Electrical Energy Generated (MWH) 8. Net Electrical Energy Generated (MWH) 9. Unit Service Factor 0. Unit Availability Factor 1. Unit Capacity Factor (Using MDC Net)	696 664.0 0 651.2 0.0 1.840.863.3 628.470 603.339 93.6 93.6 77.3 75.5 6.4	1,440 1,408.0 0 1,395.2 0 4,367,657.4 1,491,156 1,436,861 96.9 96.9 96.9 88.9 88.9	85,465 50,416 0 49,438,4 0 155,294,970 52,649,447 50,381,825 57,8 57,8					
1. Hours in Reporting Period 2. Number of Hours Reactor Was Critical 3. Reactor Reserve Shutdown Hours 4. Hours Generator On-Line 5. Unit Reserve Shutdown Hours 6. Gross Thermal Energy Generated (MWH) 7. Gross Electrical Energy Generated (MWH) 8. Net Electrical Energy Generated (MWH) 9. Unit Service Factor 0. Unit Availability Factor 1. Unit Capacity Factor (Using MDC Net) 2. Unit Capacity Factor (Using DER Net)	696 664.0 0 651,2 0.0 1,840,863.3 628,470 603,339 93.6 93.6 77.3 75.5	1,440 1,408.0 0 1,395.2 0 4,367,657.4 1,491,156 1,436,861 96.9 96.9 88.9 88.9	85,465 50,416 49,438 155,294,976 52,649,447 50,381,825 57					

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

#### UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: February 1992

DOCKET NO:\_ DATE: 03/10/92 COMPLETED BY: T. J. Hollomen

TELEPHONE: (615) 843-7528

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report No.	System Code <sup>4</sup>	Compenent Code <sup>5</sup>	Cause and Corrective * Action to Prevent Recurrence
1	2/13/92	1	N/A	A/B	5	N/A	N/A	N/A	On 2/13/92 at 0113 (EST), a power level reduction to 70 percent was initiated to allow repair of the Number 3 heater drain tank level control valve (LCV) LCV-6-1058. It appeared the plug had separated.
									Corrective maintenance was performed on LCV-6-106B and the valve was returned to service. At 234B (EST) on 2/13/92, a further decrease to 30 percent reactor power level was initiated as a result of chemistry requirements being exceeded on steam generator blowdown because of a condenser tube leak. The condenser tube leak was corrected. On 2/18/92 at 0526 (EST), reactor power increase was initiated.
2	2/27/92	1	N/A	В	5	N/A	N/A	N/A	On 2/27/92 at 1725 (EST), reactor power level was reduced to 82 percent to perform corrective maintenance on No. 7 heater drain tank LCV-6-109A. On 2/28/92, an additional decrease to 80 percent was initiated to repair a casing vent valve leak on the 1A condensate booster pump. At 2055 (EST) on 2/28/92, reactor power level increase was initiated.

1 F: Forced

2 Reason:

S: Scheduled

A-Equipment failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restruction

E-Operator Training and License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Continuation of Existing Outage

5-Reduction

9-Other

<sup>4</sup>Exhibit G-Instructions for Preparation of Data

Entry sheets for Licensee Event Report (LER) File

(NUREG-1022)

5Exhibit I-Same Source

### UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: February 1992

DOCKET NO: 50-323 UNIT NAME: DATE:

COMPLETED BY: T. J. Hollomon TELEPHONE: (615) 843-7528

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reacter <sup>3</sup>	Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective * Action to Prevent Recurrence
									Unit 2 was in coastdown to U2C5 refueling outage at the beginning of February.
	2/10/92	f	44.9	A	3	328/92001	TG	PSV	On 2/10/92 at 0528 (EST), Unit 2 reactor tripped when the turbine tripped as a result of low auto stop oil pressure. The cause of the event was the failure of the autostop trip solenoid valve on the turbine trip block.  Unit 2 was online again on 2/12/92 at 0219 (EST) and began increasing power. Unit 2 reached approximately 87 percent reactor power on 2/19/92 at 1547 (EST), and resumed coastdown to U2CS refueling outage.

1F: Forced

2 Reason:

S: Scheduled

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

N-Regulatory Restruction

E-Operator Training and License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Continuation of Existing Outage

5-Reduction

9-Other

<sup>4</sup>Exhibit G-Instructions for Preparation of Data Entry sheets for Licensee

Event Report (LER) File

(NUREG-1022)

SExhibit I-Same Source