



Commonwealth Edison  
1400 Opus Place  
Downers Grove, Illinois 60515

March 17, 1992

Dr. Thomas E. Murley, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Attn: Document Control Desk

Subject: Quad Cities Station Unit 2  
Supplemental Information Relating to  
Application for Amendment to  
Facility Operating Licenses DPR-29 and DPR-30,  
Appendix A, Technical Specifications  
NRC Docket No. 50-265

- References: (1) R. Stols to T.E. Murley letter  
dated June 10, 1991
- (2) R. Stols to T.E. Murley letter  
dated June 28, 1991
- (3) J.L. Schrage to T.E. Murley  
letter dated November 15, 1991
- (4) Teleconference on March 17, 1992  
between NRR (L. Olshan) and  
CECo (J. Schrage)

Dear Dr. Murley:

In References (1) and (2), Commonwealth Edison described a proposed modification and associated proposed Technical Specifications for the High Pressure Coolant Injection (HPCI) system. The modification added new vacuum breakers to the HPCI system. These vacuum breakers created a testable primary containment air boundary, and established a water seal between primary containment and the HPCI turbine exhaust check valve (2301-45).

Reference (3) provided additional information pertaining to revised testing requirements for the 2301-45 check valve subsequent to installation of the new vacuum breakers. During the referenced teleconference, Commonwealth Edison and NRR representatives discussed these revised requirements with respect to the criteria of Section XI of the ASME Code. The purpose of this letter is to provide information clarifying the revised testing requirements of the 2301-45 check valve.

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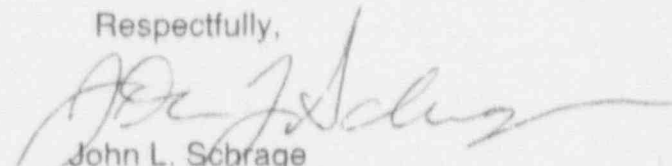
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The revised testing requirements provide verification that the HPCI turbine exhaust line does not experience water leaks, and thus is capable of maintaining an effective primary containment air boundary (via the torus water inventory). Commonwealth Edison (CECo) will test the 2301-45 check valve during inservice inspection and testing according to Section XI of the ASME Code per 10 CFR 50.55 (g). In addition, CECo will test the 2301-45 check valve utilizing Appendix J methodologies (without adding the leakage to the total containment leakage) until the revised testing requirements are incorporated into the Inservice Testing Program.

If there are any comments or questions, please contact me at 708-515-7283.

Respectfully,



John L. Schrage  
Nuclear Licensing Administrator

cc: A. Bart Davis - Regional Administrator, Region III  
F.A. Maura - Inspector, Region III  
L.N. Olshan - Project Manager, Quad Cities  
T.E. Taylor - Senior Resident Inspector, Quad Cities