

NSP

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

COPY

October 8, 1976

Mr D L Ziemann, Chief
Operating Reactors Branch No. 2
Division of Operating Reactors
U S Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr Ziemann:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Monticello Conformance to 10CFR50, Appendix J

Our submittals dated September 19, 1975, January 30, 1976, May 4, 1976, and May 5, 1976, outlined our program for implementing the provisions of 10CFR50, Appendix J at the Monticello Nuclear Generating Plant. Our submittals included a detailed description of the existing provisions for conducting containment leakage testing, a brief description of proposed modifications to permit testing in several areas to conform to Appendix J, and a request for exemption from Appendix J requirements in other areas.

We have been informed by your Staff that a number of items in our submittals require further discussion and resolution. These items include:

- a. Additional justification must be provided to exempt Core Spray and Low Pressure Coolant Injection testable check valves from Type C Tests.
- b. Additional justification must be provided to exempt Drywell Spray, Torus Spray, and RHR Recirculation motor operated valves from Type C Tests.
- c. An acceptance criterion for 10 psig airlock tests must be provided which relates the leakage rate at 10 psig to the expected leakage rate at P_a .
- d. The manner in which the Appendix J criteria were used for determining which isolation valves require Type C Tests must be explained.

We have been asked by the Staff to meet with them to discuss and resolve

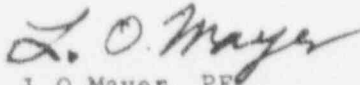
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these issues for Monticello. It is our understanding that there is widespread divergence of opinion on what is required under the current Appendix J. It may be premature to expect resolution and commitments to possible backfits on a case-by-case basis until guidance has been issued and there has been sufficient opportunity for licensee input. A meeting with your Staff to discuss our experience with containment leakage testing and the difficulties we have in satisfying certain Appendix J requirements would be very useful, however, if it is understood that we do not believe we can make any additional commitments at this time. We will make arrangements for such a meeting through our NRC Project Manager for a mutually convenient time later this month.

Because of the number of potentially generic issues that will be discussed, we have asked representatives of the Monticello Architect-Engineer and the Monticello Nuclear Steam Supply System vendor to attend also. Other interested utility representatives would also be welcome to attend.

Yours very truly,



L O Mayer, PE
Manager of Nuclear Support Services

LOM/DMM/deb

cc: J G Keppler
G Charnoff
MPCA
Attn: J W Ferman

50-263

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DESCRIPTION

Ltr re their 9-19-76, 1-30-76, 5-4-76 & 5-5-76 submittals.....furnishing info with regard to clarification of Appendix J Requirements.....

PLANT NAME: Monticello

ENCLOSURE

DONE

2. 10-13-76

SAFETY

FOR ACTION/INFORMATION

ENVIRO

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