

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Project - Unit 2 DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 1 OF 0 2 PAGE (3)

TITLE (4) R.X. Scram During Feedwater Control System Adjustment

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)												
0	4	2	3	8	4	8	4	0	3	6	0	0	0	5	1	8	8	4	0	5	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) 2	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0, 0, 5	20.406(a)(1)(i)	50.38(e)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	50.38(e)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	50.72(2)ii
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12) NAME: L.D. Kassakatis, Plant Compliance Engineer TELEPHONE NUMBER: 510 931 7711 AREA CODE: 251011

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2201

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS
A				N					

SUPPLEMENTAL REPORT EXPECTED (14) YES (if yes, complete EXPECTED SUBMISSION DATE) NO EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

During Initial Power Operation test equipment was installed to facilitate troubleshooting of the Feedwater Control System. The test equipment was installed with a bypass switch positioned improperly and this immediately injected a test signal into the Feedwater Control Circuit.

The Control Room Operator noted the change in level indication and ordered the test equipment removed but the changes injected into the Feedwater Control System had already caused the Startup Level Control Valve (RFW-FCV-10) to start cycling. As RFW-FCV-10 started cycling open and injecting cold feedwater into the RPV, an Intermediate Range Channel F Upscale Trip initiated a Subchannel-B Half Scram. This caused a Full Scram because a Subchannel-A Half Scram already existed due to surveillance testing which had been in process the entire time.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Washington Nuclear Project - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 9 7 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0 3 6	— 0 0 0	2	OF	0 2	

TEXT (If more space is required, use additional NRC Form 368A's) (17)

- (a) Power Level 5%
- (b) Plant Mode - 2
- (c) During Initial Power Operations

During Initial Power Operation the Startup Level Control Valve (RfW-FCV-10) was exhibiting hunting problems. While installing test equipment to facilitate troubleshooting a Full Scram occurred. The scram was caused by the combination of troubleshooting and surveillance testing in progress at the same time. A Subchannel-A Half Scram was inserted to support the surveillance testing and the upset introduced into the Feedwater Control System was of sufficient magnitude to cause an IRM Upscale Trip on Channel F and in turn a Subchannel-B Half Scram.

The test equipment being used was a step signal generator and the intent was to introduce simulated small RPV-Level change and monitor the response of the Feedwater Control System and its output to RfW-FCV-10 in an effort to determine the source of instability. The signal generator is equipped with a bypass switch that when left in the ON position prevents the introduction of a test signal into the actual control circuitry. When the signal generator was plugged in the bypass switch was inadvertently left in the OFF position and this immediately introduced a step change in RPV-Level into the Feedwater Control System.

The simulated level change was observed by the Control Room Operator and he ordered the test equipment removed. The test equipment was removed immediately but RfW-FCV-10 had already started cycling from closed to open. As RfW-FCV-10 cycled open cold feedwater was injected into the RPV and this caused the IRM upscale trip.

After the scram the plant was kept in a shutdown condition for a two day outage.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397
May 18, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

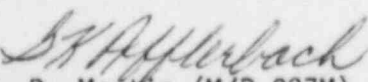
Subject: NUCLEAR PROJECT NO. 2
LICENSEE EVENT REPORT NO. 84-036

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-036 for WNP-2 Plant. This report is submitted in response to the report requirements of Technical Specification Section 6.9.1.7 and discusses the item of noncompliance, corrective action taken, and action taken to preclude recurrence.

This is the follow-up report to the verbal notification given at 1328 hours on April 23, 1984.

Very truly yours,


J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:mm

Enclosure:

Licensee Event Report No. 84-036

cc: Mr. John B. Martin, Administrator
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