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March 1984

LICENSED OPERATING REACTORS

STATUS SUMMARY REPORT
DATA AS OF 02-29-84

UNITED STATES NUCLEAR REGULATORY COMMISSION

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Manuscript Completed: April 1984
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OFFICE OF RESOURCE MANAGEMENT
U.S. NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

AUTHORIZATION AND CLEARANCE*

The U.S. Nuclear Regulatory Commission's Office of Management and Program Analysis publishes this monthly status report "as part of the reporting requirements in Section 50.36 of 10 CFR Part 50 under GAO Clearance Number B-180225, with an expiration date of September 30, 1981," as stated in the October 3, 1978 letter from John M. Lovelady, Assistant Director, General Government Division, U.S. General Accounting Office, to J.M. Felton, Director, Division of Rules and Records, U.S. Nuclear Regulatory Commission.

*Extended to April 30, 1985 by OMB Directive 3150-0011.

STATEMENT OF PURPOSE

The U.S. Nuclear Regulatory Commission's monthly LICENSED OPERATING REACTORS Status Summary Report provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Resource Management, from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor years of experience and non-power reactors in the United States.

The percentage computations, Items 20 through 24 in Section 2, the vendor capacity factors on page 1-7, and actual vs. potential energy production on Page 1-2 are computed using actual data for the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetic average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U.S. energy situation as a whole.

TABLE OF CONTENTS

G L O S S A R Y

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours, divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (NET MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an offnormal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.

HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilizations. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous minth, report the same outage date but change "Method of Shutting Down Reactor" to "4 (continuations)" and add a note: "Continued from previous month."
OUTAGE DURATION	The Total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Averagn Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The Total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period that the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calender month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)

UNIT AVAILABILITY FACTOR	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
- Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
- Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
- Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
- Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
- Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC GROSS and/or MDC NET have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

NOTE:

At the end of each statement in the Enforcement Summary for any given facility may be found numbers in parentheses. These numbers are related to the inspection, e.g., 8111 (the 11th inspection of the plant in 1981); and the severity level, e.g., 4 (severity level IV). Violations are ranked by severity levels from I through V with level I being the most serious. The severity level is used in the determination of any resulting enforcement action. Gray Book lists severity level by Arabic numbers corresponding to the Roman numerals. Details on the various severity levels and enforcement actions can be found in Appendix C to 10 CFR Part 2 published in the Federal Register of March 9, 1982 pages 9987 through 9995, and as corrected April 14, 1982.

INDEX TO OPERATING POWER REACTORS

SECTION 1

CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS

LICENSED POWER REACTORS				
	77	IN COMMERCIAL OPERATION	59,865	CAPACITY MWe (Net) --Based upon maximum dependable capacity; design elec. rating used if MDC not determined
(a)	2	IN POWER ASCENSION	2,267	
(b)	79	LICENSED TO OPERATE	62,132	TOTAL
(c)	3	LICENSED FOR FUEL LOADING AND LOW POWER TESTING		

	MDC NET
(a) MCGUIRE 2	1180
SAN ONOFRE 3	1087

(b) Excludes these plants licensed for operation which are shut down indefinitely

	DER
1. DRESDEN 1	200
2. HUMBOLDT BAY	65
3. TMI 2	906

(c)	DATE	DER
GRAND GULF 1	06/16/82	1250
LA SALLE 2	12/16/83	1078
WASH. NUC. 2	12/20/83	1103

POWER GENERATION

	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	28,561,699	30,126,759	58,688,458
2. NET ELECTRICAL (MWHE)	27,258,287	28,789,343	56,047,630
3. AVG. UNIT SERVICE FACTOR (%)	69.6	70.3	70.0
4. AVG. UNIT AVAILABILITY FACTOR (%)	69.6	70.3	70.0
5. AVG. UNIT CAPACITY FACTOR (MDC) (%)	65.8	65.5	65.6
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	64.1	63.8	64.0
7. FORCED OUTAGE RATE (%)	11.2	11.7	11.4

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
1.	ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	27,258,287 NET	65.4
2.	ENERGY NOT PRODUCED DUE TO SCHEDULED OUTAGES (NET)	7,743,760 MWHe	18.6
3.	ENERGY NOT PRODUCED DUE TO FORCED OUTAGES (NET)	4,801,049 MWHe	11.5
4.	ENERGY NOT PRODUCED FOR OTHER REASONS (NET)	1,862,944 MWHe	4.5
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		41,666,040 MWHe	100.0% TOTAL
5.	ENERGY NOT PRODUCED DUE TO NRC-REQUIRED OUTAGES	564,029 MWHe	
6.	ENERGY NOT PRODUCED DUE TO NRC RESTRICTED POWER LEVELS	..... MWHe	0 UNIT(S) WITH NRC RESTRICTION

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING REPORT PERIOD	58	5,512.5	10.3	4,801,049
2. SCHEDULED OUTAGES DURING REPORT PERIOD	23	10,785.6	20.1	7,743,760
TOTAL	81	16,298.1	30.4	12,544,809

MWHE LOST PRODUCTION = Down time X maximum dependable capacity net

MONTHLY HIGHLIGHTS

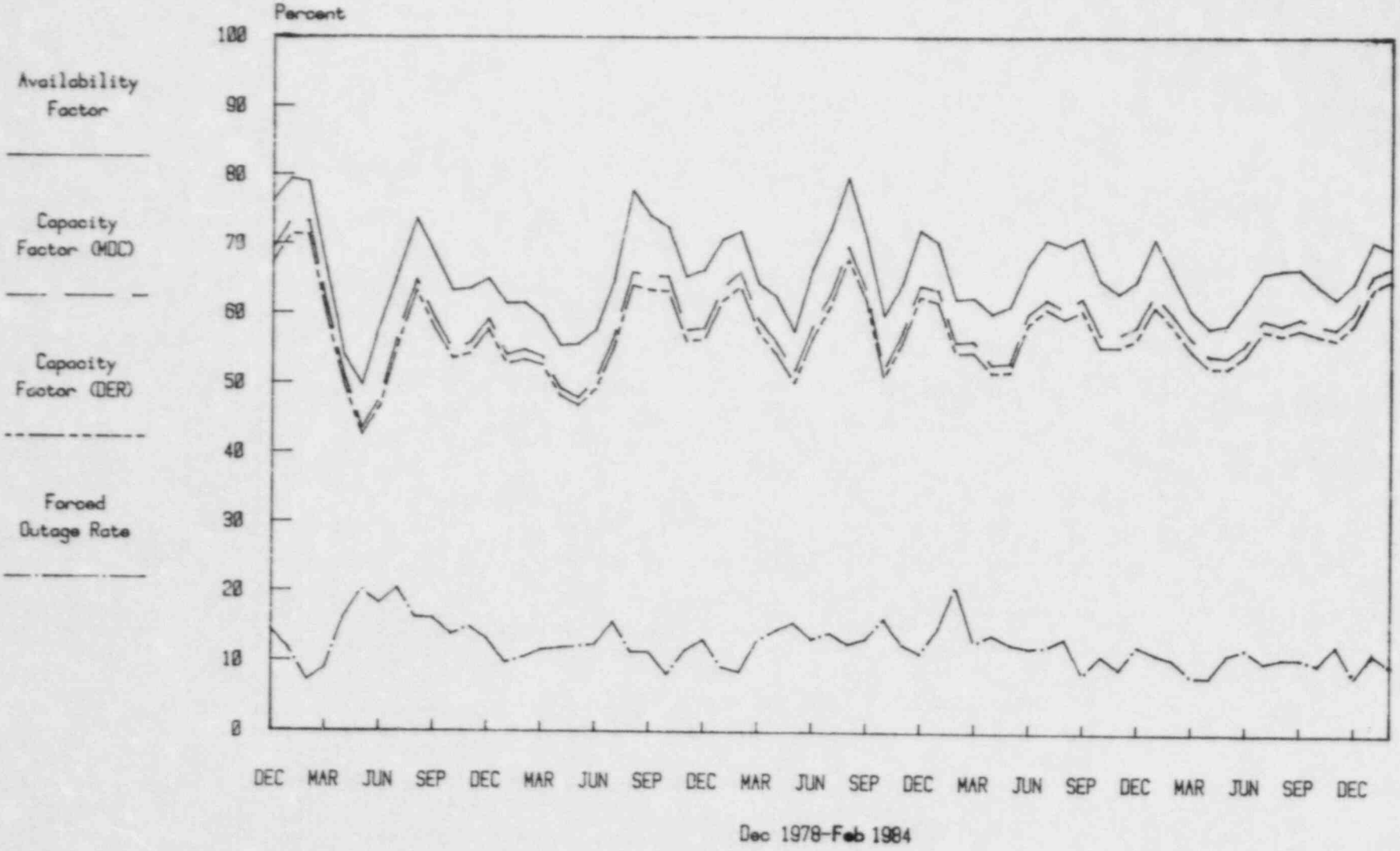
REASONS FOR SHUTDOWNS		NUMBER	HOURS LOST
	A - Equipment Failure	41	4,170.5
	B - Maintenance or Test	12	1,438.6
	C - Refueling	15	8,132.7
	D - Regulatory Restriction	2	723.7
	E - Operator Training & License Examination	0	0.0
	F - Administrative	0	0.0
	G - Operational Error	3	178.2
	H - Other	8	1,654.4
	TOTAL	81	16,298.1

DERATED		MDC (MWe Net)	POWER LIMIT (MWe Net)	TYPE
	FORT ST VRAIN	330	280	Self-imposed

SHUTDOWNS GREATER THAN 72 HRS EACH	UNIT	REASON	UNIT	REASON	UNIT	REASON	UNIT	REASON
	BIG ROCK POINT 1	A	BROWNS FERRY 1	A	BROWNS FERRY 2	H	BROWNS FERRY 3	C
	BRUNSWICK 1	A	DRESDEN 3	C	FARLEY 1	C	FORT ST VRAIN	C
	HATCH 1	A,H	HATCH 2	H	INDIAN POINT 2	A	LASALLE 1	A
	MCGUIRE 1	C	MILLSTONE 2	A	MONTICELLO	C	NORTH ANNA 1	A
	NORTH ANNA 2	G	OYSTER CREEK 1	C	PALISADES	C	PEACH BOTTOM 2	B
	PILGRIM 1	C	POINT BEACH 1	C	QUAD CITIES 2	C	ROBINSON 2	C
	SALEM 1	A	SALEM 2	A	SAN ONOFRE 1	B	SAN ONOFRE 2	B
	SEQUOYAH 1	C	ST LUCIE 1	C	SURRY 1	H	SUSQUEHANNA 1	A,H
	THREE MILE ISLAND 1	D	TURKEY POINT 3	A	TURKEY POINT 4	A	ZION 1	A

Unit Availability, Capacity, Forced Outage

Avg. Unit Percentage as of ~~02-29-84~~



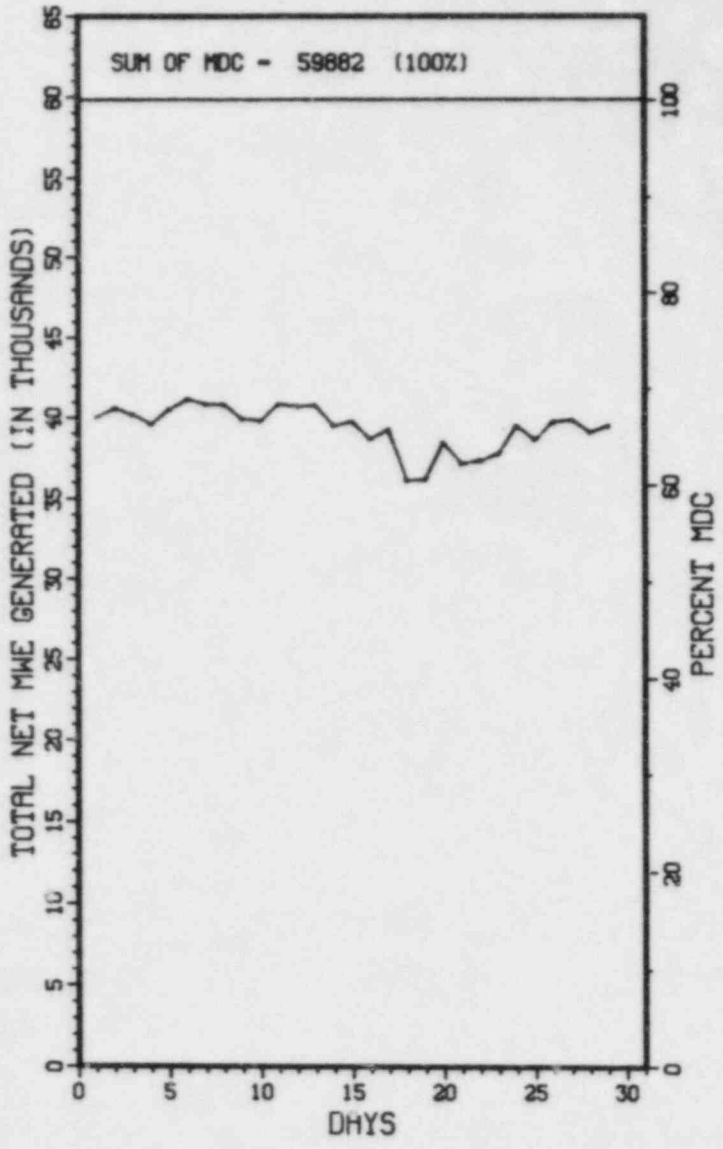
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MDC" is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the same units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MDC".

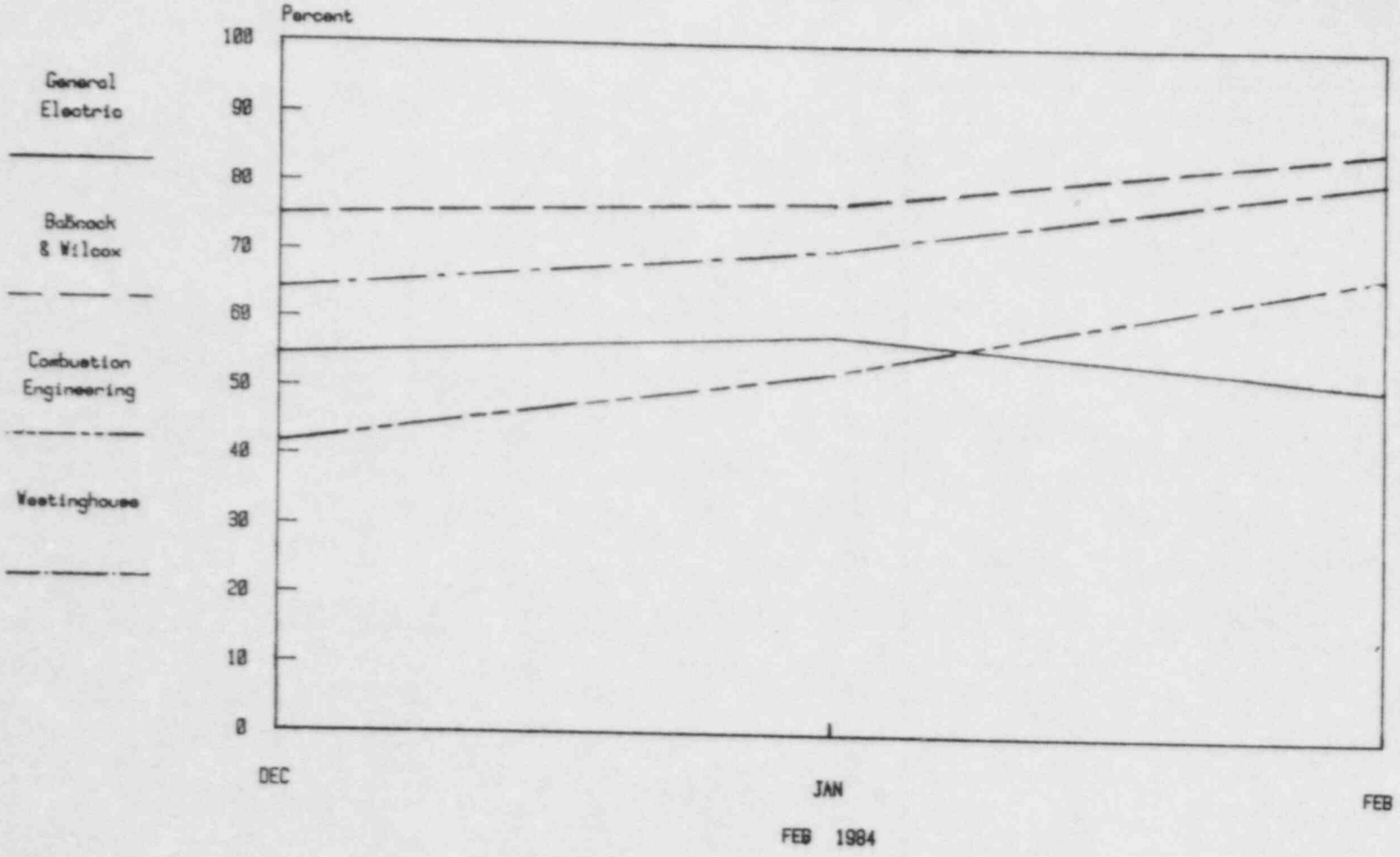
It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



FEBRUARY 1984

Vendor Average Capacity Factors

As Of 02-29-84



NOTE: This display of average capacity factors provides a general performance comparison of plants supplied by the four nuclear steam supply system vendors. One must be careful when drawing conclusions regarding the reasons for the performance levels indicated, since plant performance may be affected by unspecified factors such as: (1) various plant designs and models are included for each vendor; (2) turbine/generators and (3) different architect/engineers are also involved.

AVERAGE CAPACITY FACTORS BY VENDORS

GENERAL ELECTRIC	CFMDC		CFMDC		CFMDC		CFMDC	
	59.9	BROWNS FERRY 1	62.7	BROWNS FERRY 2	0.0	BROWNS FERRY 3	86.1	BRUNSWICK 1
	85.8	BRUNSWICK 2	88.3	COOPER STATION	91.1	DRESDEN 2	0.0	DRESDEN 3
	98.6	DUANE ARNOLD	93.8	FITZPATRICK	19.0	HATCH 1	0.0	HATCH 2
	28.2	LASALLE 1	99.0	MILLSTONE 1	4.3	MONTICELLO	88.0	NINE MILE POINT 1
	0.0	OYSTER CREEK 1	61.2	PEACH BOTTOM 2	93.1	PEACH BOTTOM 3	0.0	PILGRIM 1
	96.8	QUAD CITIES 1	10.1	QUAD CITIES 2	5.6	SUSQUEHANNA 1	103.0	VERMONT YANKEE 1

BABCOCK & WILCOX	CFMDC		CFMDC		CFMDC		CFMDC	
	91.3	ARKANSAS 1	91.7	CRYSTAL RIVER 3	97.9	DAVIS-BESSE 1	100.2	OCONEE 1
	99.3	OCONEE 2	97.3	OCONEE 3	90.0	RANCHO SECO 1	0.0	THREE MILE ISLAND 1

COMBUSTION ENGINEERING	CFMDC		CFMDC		CFMDC		CFMDC	
	59.9	ARKANSAS 2	102.6	CALVERT CLIFFS 1	98.0	CALVERT CLIFFS 2	99.7	FORT CALHOUN 1
	99.7	MAINE YANKEE	77.6	MILLSTONE 2	0.0	PALISADES	43.4	SAN ONOFRE 2
	0.0	ST LUCIE 1	100.1	ST LUCIE 2				

WESTINGHOUSE	CFMDC		CFMDC		CFMDC		CFMDC	
	98.1	BEAVER VALLEY 1	86.2	COOK 1	96.7	COOK 2	33.7	FARLEY 1
	102.3	FARLEY 2	99.3	GINNA	101.5	HADDAM NECK	37.3	INDIAN POINT 2
	68.6	INDIAN POINT 3	102.1	KEWAUNEE	74.9	MCGUIRE 1	67.3	NORTH ANNA 1
	73.9	NORTH ANNA 2	0.0	POINT BEACH 1	98.8	POINT BEACH 2	100.8	PRAIRIE ISLAND 1
	104.2	PRAIRIE ISLAND 2	0.0	ROBINSON 2	82.4	SALEM 1	0.0	SALEM 2
	0.0	SAN ONOFRE 1	60.1	SEQUOYAH 1	86.5	SEQUOYAH 2	84.9	SUMMER 1
	76.3	SURRY 1	93.3	SURRY 2	90.8	TROJAN	62.1	TURKEY POINT 3
	61.1	TURKEY POINT 4	96.8	YANKEE-ROWE 1	45.6	ZION 1	97.9	ZION 2

OTHER INFO

Units excluded are:
- BIG ROCK POINT
- DRESDEN 1
- FORT ST VRAIN
- HUMBOLDT BAY
- LACROSSE
- THREE MILE ISLAND 2

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the glossary. The vendor averages are computed by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production by Vendor in this Month}} \times 100\%$$

	GE BWRs	West PWRs	Comb PWRs	B&W PWRs	ALL PWRs
NET ELECTRICAL PRODUCTION.....	6,934,975	12,582,345	3,707,674	3,974,315	20,264,334
MDC NET.....	19,226	25,508	7,929	6,760	40,197
CFMDC.....	51.8	70.9	67.2	84.5	72.4

MEMORANDA

THE FOLLOWING UNITS USE WEIGHTED AVERAGES TO CALCULATE CAPACITY FACTORS:

ITEM 22

BIG ROCK POINT 1
CALVERT CLIFFS 1 & 2
FARLEY 1
FITZPATRICK
FORT CALHOUN 1
INDIAN POINT 2*
KEWAUNEE
OYSTER CREEK 1
POINT BEACH 1 & 2
THREE MILE ISLAND 1
TURKEY POINT 3 & 4

ITEM 22 & 23

GINNA
HADDAM NECK (CONNECTICUT YANKEE)
MAINE YANKEE
MILLSTONE 2
OCONEE 1, 2, & 3
YANKEE-ROWE 1

*COMPUTED SINCE 7/1/74, THE DATE OF COMPLETION OF A 100 DAY - 100% POWER OPERATION TEST.

THE FOLLOWING UNITS USE THE DATE OF FIRST ELECTRICAL GENERATION INSTEAD OF COMMERCIAL OPERATION, FOR THEIR CUMULATIVE DATA:

ITEMS 20 THROUGH 24

COOK 1 & 2
BEAVER VALLEY 1
SAN ONOFRE 1

ITEM 24 ONLY

BIG ROCK POINT 1

E R R A T A

CORRECTIONS TO PREVIOUSLY REPORTED DATA

NOTE: THESE CHANGES ARE REFLECTED IN THE DATA CONTAINED IN THE CURRENT REPORT

Report Month - January 1984

	Vol. 8, No. 2	Revised
Gross Elec.	29,110,788	30,126,759
Net Elec.	27,824,839	28,789,343
Unit Serv.	70.0	70.3
Unit Avail.	70.0	70.3
Cap. Fac. (MDC)	65.5	65.5
Cap. Fac. (DER)	63.7	63.8
F. Outage Rate	11.9	11.7

The above monthly figures for January 1984 were revised.

SECTION 2

OPERATING POWER REACTORS

1. Docket: 50-313 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: K. L. MORTON (501) 964-3155

4. Licensed Thermal Power (MWt): 2568

5. Nameplate Rating (Gross MWe): 1003 X 0.9 = 903

6. Design Electrical Rating (Net MWe): 850

7. Maximum Dependable Capacity (Gross MWe): 883

8. Maximum Dependable Capacity (Net MWe): 836

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	80,635.0
13. Hours Reactor Critical	696.0	1,440.0	53,875.3
14. Rx Reserve Shtdwn Hrs	.0	.0	5,044.0
15. Hrs Generator On-Line	696.0	1,440.0	52,690.2
16. Unit Reserve Shtdwn Hrs	.0	.0	817.5
17. Gross Therm Ener (MWH)	1,648,365	3,549,633	125,469,230
18. Gross Elec Ener (MWH)	554,700	1,194,755	41,333,120
19. Net Elec Ener (MWH)	530,977	1,145,442	39,403,829
20. Unit Service Factor	100.0	100.0	65.3
21. Unit Avail Factor	100.0	100.0	66.4
22. Unit Cap Factor (MDC Net)	91.3	95.1	58.5
23. Unit Cap Factor (DER Net)	89.8	93.4	57.5
24. Unit Forced Outage Rate	.0	.0	16.2
25. Forced Outage Hours	.0	.0	10,178.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

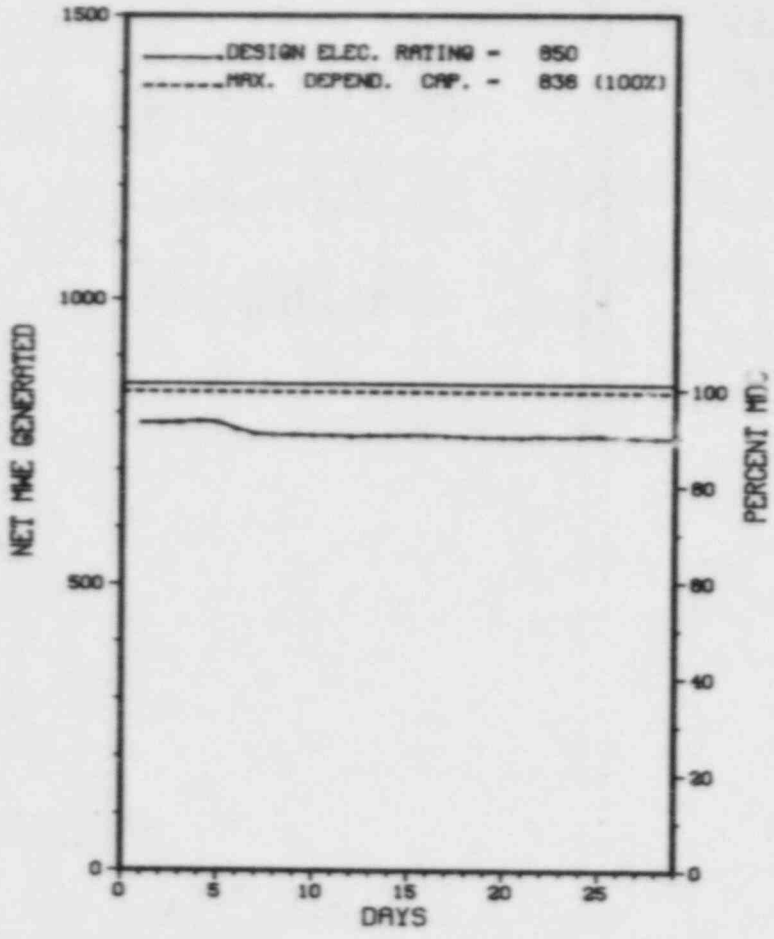
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ARKANSAS 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ARKANSAS 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

ARKANSAS 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

ARKANSAS 1 OPERATED WITH NO OUTAGES OR REDUCTIONS DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
		9-Other	(LER) File (NUREG-0161)

****
* ARKANSAS 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ARKANSAS

COUNTY.....POPE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...6 MI WNW OF
RUSSELLVILLE, AR

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 6, 1974

DATE ELEC ENER 1ST GENER...AUGUST 17, 1974

DATE COMMERCIAL OPERATE....DECEMBER 19, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DARDANELLE RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ARKANSAS POWER & LIGHT

CORPORATE ADDRESS.....NINTH & LOUISIANA STREETS
LITTLE ROCK, ARKANSAS 72203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....L. CALLAN

LICENSING PROJ MANAGER.....G. VISSING
DOCKET NUMBER.....50-313

LICENSE & DATE ISSUANCE....DPR-51, MAY 21, 1974

PUBLIC DOCUMENT ROOM.....ARKANSAS TECH UNIVERSITY
RUSSELLVILLE, ARKANSAS 72801

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED NOVEMBER 17-18, 1983 (83-30): ANNOUNCED INSPECTION OF THE SECURITY PLAN TO EXAMINE PROPOSED CHANGES. WITHIN THE 107 PROPOSED CHANGES INSPECTED, THOSE ITEMS IDENTIFIED AS REQUIRING A 10 CFR 50.90 CHANGE WILL BE SUBMITTED TO THE OFFICE OF NUCLEAR REACTOR REGULATION, NRC. SEVERAL OTHER ITEMS SUBMITTED AS 50.54(P) CHANGES WILL ALSO BE REFERRED TO OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS FOR GUIDANCE. THE REMAINING ITEMS ARE RECOGNIZED AS 50.54(P) CHANGES TO THE ANO SECURITY PLAN.

INSPECTION CONDUCTED DECEMBER 1-31, 1983, (83-34): ROUTINE, ANNOUNCED INSPECTION OF MAINTENANCE,SURVEILLANCE, OPERATIONAL SAFETY VERIFICATION, IE BULLETIN FOLLOWUP, AND FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS. WITHIN THE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED (USE OF A RADIATION MONITORING INSTRUMENT THAT WAS NOT CALIBRATED); AND ONE DEVIATION WAS IDENTIFIED (FAILURE TO MEET A COMMITMENT RELATIVE TO REACTOR TRIP BREAKER MAINTENANCE).

INSPECTION CONDUCTED DECEMBER 19-21, 1983, (83-36): ROUTINE, UNANNOUNCED INSPECTION OF THE QUALITY ASSURANCE PLANS, INSTRUCTIONS, AND PROCEDURES FOR A LOW-LEVEL RADIOACTIVE WASTE STORAGE FACILITY, AND THE FOLLOWING AREAS CONCERNING TRANSPORTATION ACTIVITIES AND LAND DISPOSAL OF RADIOACTIVE WASTE: MANAGEMENT CONTROLS; INDOCTRINATION AND TRAINING PROGRAMS; AUDIT PROGRAMS; PROCUREMENT AND SELECTION OF PACKAGES; PREPARATION OF PACKAGES FOR SHIPMENT; DELIVERY OF COMPLETED PACKAGES TO CARRIER; RECEIPT OF PACKAGES; RECORDS AND REPORTS; WASTE MANIFESTS; WASTE CLASSIFICATIONS; WASTE FORM AND CHARACTERIZATION; WASTE SHIPMENT LABELING; AND TRACKING OF WASTE SHIPMENTS. WITHIN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: DECEMBER 19-21, 1983

INSPECTION REPORT NO: 50-313/83-36

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-368 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: LINDY BRAMLETT (501) 964-3145
4. Licensed Thermal Power (MWt): 2815
5. Nameplate Rating (Gross MWe): 943
6. Design Electrical Rating (Net MWe): 912
7. Maximum Dependable Capacity (Gross MWe): 897
8. Maximum Dependable Capacity (Net MWe): 858
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	34,464.0
13. Hours Reactor Critical	696.0	823.6	22,496.3
14. Rx Reserve Shtdwn Hrs	.0	.0	1,430.1
15. Hrs Generator On-Line	688.8	688.8	21,639.1
16. Unit Reserve Shtdwn Hrs	.0	.0	75.0
17. Gross Therm Ener (MWH)	1,178,733	1,178,936	53,728,476
18. Gross Elec Ener (MWH)	381,460	381,460	17,398,411
19. Net Elec Ener (MWH)	357,446	357,446	16,563,786
20. Unit Service Factor	99.0	47.8	62.8
21. Unit Avail Factor	99.0	47.8	63.0
22. Unit Cap Factor (MDC Net)	59.9	28.9	56.0
23. Unit Cap Factor (DER Net)	56.3	27.2	52.7
24. Unit Forced Outage Rate	.0	.0	19.9
25. Forced Outage Hours	.0	.0	5,378.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

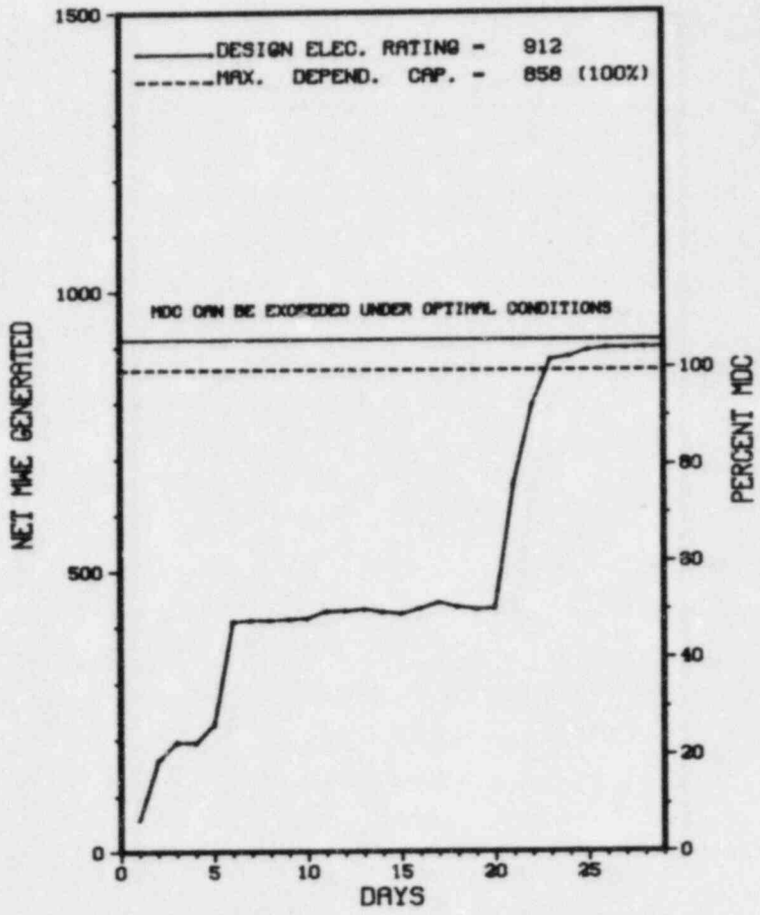
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ARKANSAS 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ARKANSAS 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

ARKANSAS 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
83-10	10/05/83	S	7.2	C	4		ZZ	ZZZZZZ	REFUELING AND MAINTENANCE OUTAGE CONCLUDES.

SUMMARY

ARKANSAS 2 RETURNED ONLINE FEBRUARY 1ST AND OPERATED ROUTINELY THE REMAINDER OF THE REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report (LER) File (NUREG-0161)
			9-Other	

****
* ARKANSAS 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ARKANSAS

COUNTY.....POPE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...6 MI WNW OF
RUSSELLVILLE, AR

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 5, 1978

DATE ELEC ENER 1ST GENER...DECEMBER 26, 1978

DATE COMMERCIAL OPERATE....MARCH 26, 1980

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....DARDANELLE RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ARKANSAS POWER & LIGHT

CORPORATE ADDRESS.....NINTH & LOUISIANA STREETS
LITTLE ROCK, ARKANSAS 72203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....L. CALLAN

LICENSING PROJ MANAGER.....R. LEE
DOCKET NUMBER.....50-368

LICENSE & DATE ISSUANCE....NPF-6, SEPTEMBER 1, 1978

PUBLIC DOCUMENT ROOM.....ARKANSAS TECH UNIVERSITY
RUSSELLVILLE, ARKANSAS 72801

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED NOVEMBER 17-18, 1983 (83-30): ANNOUNCED INSPECTION OF THE SECURITY PLAN TO EXAMINE PROPOSED CHANGES. WITHIN THE 107 PROPOSED CHANGES INSPECTED, THOSE ITEMS IDENTIFIED AS REQUIRING A 10 CFR 50.90 CHANGE WILL BE SUBMITTED TO THE OFFICE OF NUCLEAR REACTOR REGULATION, NRC. SEVERAL OTHER ITEMS SUBMITTED AS 50.54(P) CHANGES WILL ALSO BE REFERRED TO OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS FOR GUIDANCE. THE REMAINING ITEMS ARE RECOGNIZED AS 50.54(P) CHANGES TO THE ANO SECURITY PLAN.

INSPECTION CONDUCTED DECEMBER 1-31, 1983, (83-34): ROUTINE, ANNOUNCED INSPECTION OF MAINTENANCE,SURVEILLANCE, OPERATIONAL SAFETY VERIFICATION, IE BULLETIN FOLLOWUP, AND FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS. WITHIN THE AREAS INSPECTED, TWO VIOLATIONS WERE IDENTIFIED (USE OF TEST EQUIPMENT THAT DID NOT HAVE A CURRENT CALIBRATION) AND (FAILURE TO FOLLOW PRECEDURES FOR RECORDING OF MEASURING AND TEST EQUIPMENT).

INSPECTION CONDUCTED DECEMBER 19-21, 1983, (83-36): ROUTINE, UNANNOUNCED INSPECTION OF THE QUALITY ASSURANCE PLANS, INSTRUCTIONS, AND PROCEDURES FOR A LOW-LEVEL RADIOACTIVE WASTE STORAGE FACILITY, AND THE FOLLOWING AREAS CONCERNING TRANSPORTATION ACTIVITIES AND LAND DISPOSAL OF RADIOACTIVE WASTE: MANAGEMENT CONTROLS; INDOCTRINATION AND TRAINING PROGRAMS; AUDIT PROGRAMS; PROCUREMENT AND SELECTION OF PACKAGES; PREPARATION OF PACKAGES FOR SHIPMENT; DELIVERY OF COMPLETED PACKAGES TO CARRIER; RECEIPT OF PACKAGES; RECORDS AND REPORTS; WASTE MANIFESTS; WASTE CLASSIFICATIONS; WASTE FORM AND CHARACTERIZATION; WASTE SHIPMENT LABELING; AND TRACKING OF WASTE SHIPMENTS. WITHIN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

CONTRARY TO UNIT 2 TS 6.8.1, THE LICENSEE DID NOT FOLLOW THE REQUIREMENTS OF STEP 6.2.4 OF PROCEDURE 1000.14 IN THAT THE M AND TE NUMBER WAS NOT RECORDED FOR A VISICORDER USED TO PERFORM SURVEILLANCE ON RPS CHANNEL C.
(8334 5)

CONTRARY TO UNIT 2 TECHNICAL SPECIFICATION 6.8.1, THE LICENSEE DID NOT ADEQUATELY IMPLEMENT PROCEDURE 1000. 24, "CONTROL OF MAINTENANCE," IN THAT FOUR OUT OF 36 JOB ORDER FORMS REVIEWED DID NOT PROPERLY DOCUMENT POST MAINTENANCE TESTING AND INSPECTIONS.

(8401 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

POWER OPERATION

LAST IE SITE INSPECTION DATE: DECEMBER 19-21, 1983

INSPECTION REPORT NO: 50-368/83-36

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-334 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: J. L. HOLTZ (412) 643-1369
4. Licensed Thermal Power (MWt): 2660
5. Nameplate Rating (Gross MWe): 1026 X 0.9 = 923
6. Design Electrical Rating (Net MWe): 835
7. Maximum Dependable Capacity (Gross MWe): 860
8. Maximum Dependable Capacity (Net MWe): 810
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	68,664.0
13. Hours Reactor Critical	696.0	1,414.0	32,297.3
14. Rx Reserve Shtdwn Hrs	.0	.0	4,482.7
15. Hrs Generator On-Line	696.0	1,352.1	31,131.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,811,480	3,363,799	70,953,331
18. Gross Elec Ener (MWH)	587,700	1,091,500	22,520,440
19. Net Elec Ener (MWH)	553,128	1,041,605	20,930,403
20. Unit Service Factor	100.0	93.9	47.5
21. Unit Avail Factor	100.0	93.9	47.5
22. Unit Cap Factor (MDC Net)	98.1	89.3	41.1
23. Unit Cap Factor (DER Net)	95.2	86.6	39.9
24. Unit Forced Outage Rate	.0	6.1	30.2
25. Forced Outage Hours	.0	87.9	17,765.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

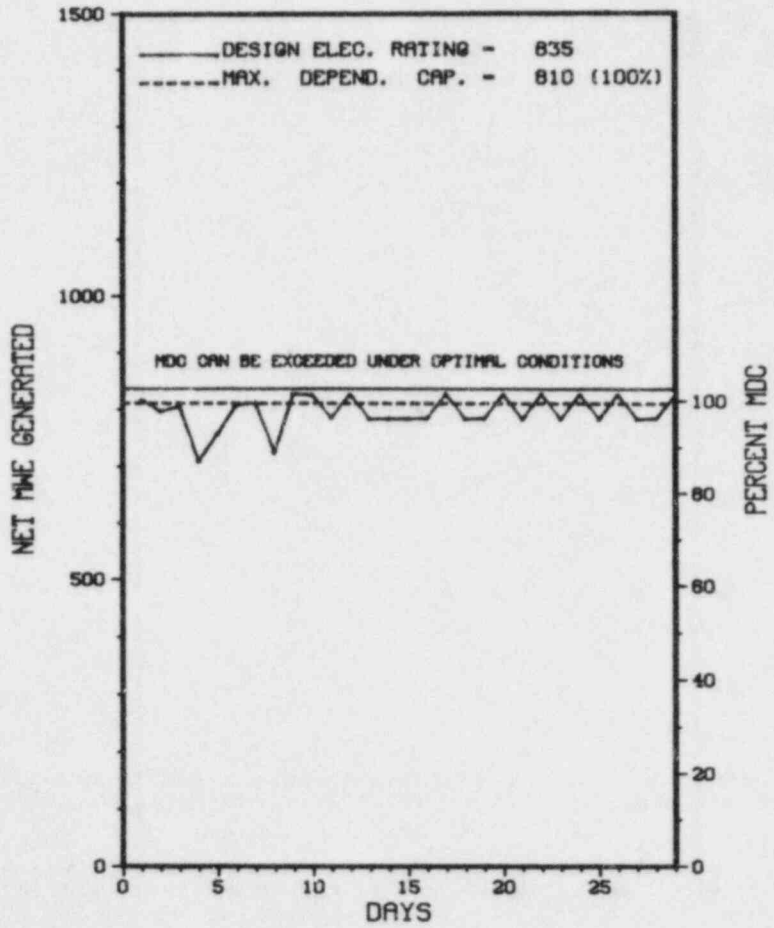
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BEAVER VALLEY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BEAVER VALLEY 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

BEAVER VALLEY 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

BEAVER VALLEY 1 OPERATED ROUTINELY WITH NO OUTAGES OR REDUCTIONS DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BEAVER VALLEY 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....BEAVER

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI E OF
E. LIVERPOOL, OH

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 10, 1976

DATE ELEC ENER 1ST GENER...JUNE 14, 1976

DATE COMMERCIAL OPERATE....OCTOBER 1, 1976

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....OHIO RIVER

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUQUESNE LIGHT

CORPORATE ADDRESS.....ONE OXFORD CENTRE, 301 GRANT STREET
PITTSBURGH, PENNSYLVANIA 15279

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....W. TROSKOSKI

LICENSING PROJ MANAGER.....P. TAM
DOCKET NUMBER.....50-334

LICENSE & DATE ISSUANCE....DPR-66, JULY 2, 1976

PUBLIC DOCUMENT ROOM.....B.F. JONES MEMORIAL LIBRARY
633 FRANKLIN AVENUE
ALIQUIPPA, PA 15001

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-155 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: SUE AMSTUTZ (616) 547-6537
4. Licensed Thermal Power (MWt): 240
5. Nameplate Rating (Gross MWe): 70.6 X 0.85 = 60
6. Design Electrical Rating (Net MWe): 72
7. Maximum Dependable Capacity (Gross MWe): 69
8. Maximum Dependable Capacity (Net MWe): 64
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	183,427.0
13. Hours Reactor Critical	435.1	1,179.1	128,889.5
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	432.8	1,176.8	126,469.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	83,473	227,577	23,713,468
18. Gross Elec Ener (MWH)	27,406	74,678	7,490,287
19. Net Elec Ener (MWH)	25,879	70,512	7,082,724
20. Unit Service Factor	62.2	81.7	68.9
21. Unit Avail Factor	62.2	81.7	68.9
22. Unit Cap Factor (MDC Net)	58.1	76.5	57.5*
23. Unit Cap Factor (DER Net)	51.6	68.0	53.6
24. Unit Forced Outage Rate	37.8	18.3	17.0
25. Forced Outage Hours	263.2	263.2	10,163.5

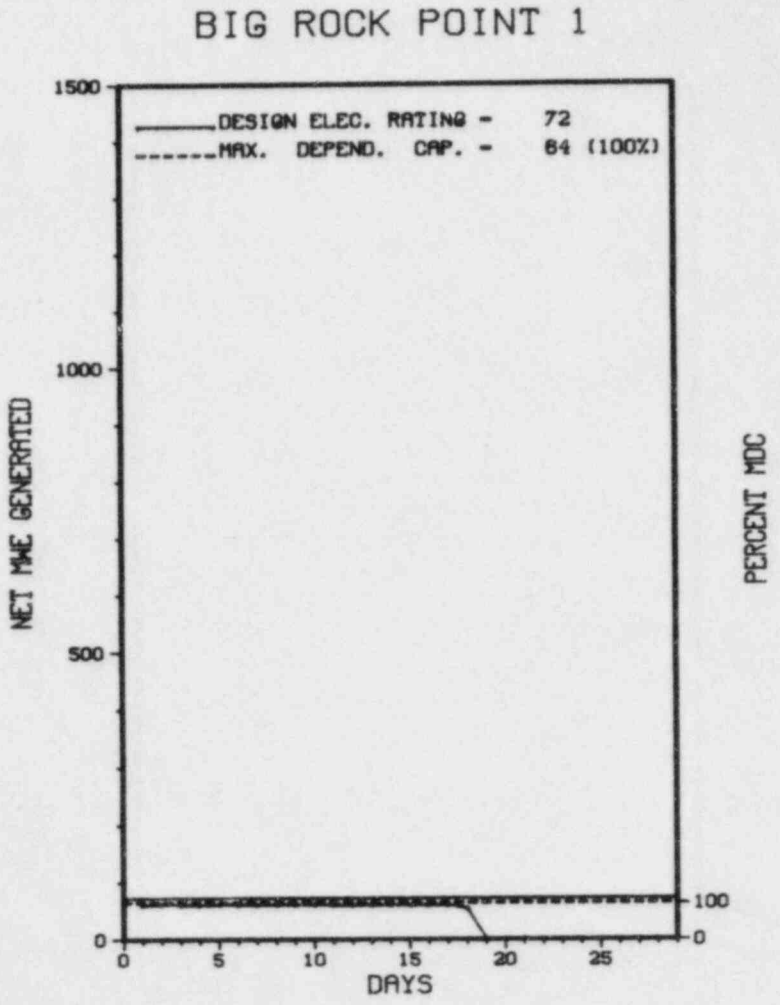
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: 03/17/84

****
* BIG ROCK POINT 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

BIG ROCK POINT 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-01	02/19/84	F	263.2	A	1		CB	VALVEX	STEAM LEAK IN THE PACKING OF A PRIMARY SYSTEM VALVE CAUSED THE INITIAL SHUTDOWN. SUBSEQUENT FAILURE OF THREE OUT OF FOUR REACTOR DEPRESSURIZATION VALVES REQUIRED ADDITIONAL REPAIRS AND TESTING PRIOR TO POWER ESCALATION.

SUMMARY

BIG ROCK POINT 1 SHUTDOWN ON FEBRUARY 19TH FOR A STEAM LEAK AND SUBSEQUENT FAILURE OF RDS VALVES.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BIG ROCK POI . . *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE..... ..MICHIGAN

COUNTY.....CHARLEVOIX

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...4 MI NE OF
CHARLEVOIX, MICH

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 27, 1962

DATE ELEC ENER 1ST GENER...DECEMBER 8, 1962

DATE COMMERCIAL OPERATE....MARCH 29, 1963

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONSUMERS POWER

CORPORATE ADDRESS.....212 WEST MICHIGAN AVENUE
JACKSON, MICHIGAN 49201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....G. WRIGHT

LICENSING PROJ MANAGER.....R. EMCH
DOCKET NUMBER.....50-155

LICENSE & DATE ISSUANCE....DPR-6, AUGUST 30, 1962

PUBLIC DOCUMENT ROOM.....CHARLEVOIX PUBLIC LIBRARY
107 CLINTON STREET
CHARLEVOIX, MICHIGAN 49720

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON NOVEMBER 19, - JANUARY 31, (83-19): ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTOR OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, OPERATIONAL SAFETY, MAINTENANCE, SURVEILLANCE, LERS, TMI ITEM REVIEW, FIRE PROTECTION, AND REGIONAL REQUESTS. THE INSPECTION INVOLVED A TOTAL OF 201 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

****
* BIG ROCK POINT 1 *
****

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT WAS SHUT DOWN ON 2/19/84 TO REPAIR LEAK ON RECIRCULATING PUMP SEAL COOLANT LINE, AND REMAINED SHUT DOWN THROUGH THE END OF THE MONTH.

LAST IE SITE INSPECTION DATE: JANUARY 1-31, 1984

INSPECTION REPORT NO: 84-01

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-259 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: TED THOM (205) 729-0834
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1065
7. Maximum Dependable Capacity (Gross MWe): 1098
8. Maximum Dependable Capacity (Net MWe): 1065
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	84,002.0
13. Hours Reactor Critical	480.8	1,208.5	51,014.3
14. Rx Reserve Shtdwn Hrs	215.1	215.1	5,999.7
15. Hrs Generator On-Line	453.1	1,140.3	49,857.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,363,322	2,988,964	141,546,643
18. Gross Elec Ener (MWH)	459,170	1,005,810	46,651,430
19. Net Elec Ener (MWH)	444,294	977,541	45,302,868
20. Unit Service Factor	65.1	79.2	59.4
21. Unit Avail Factor	65.1	79.2	59.4
22. Unit Cap Factor (MDC Net)	59.9	63.7	50.6
23. Unit Cap Factor (DER Net)	59.9	63.7	50.6
24. Unit Forced Outage Rate	34.9	19.0	23.7
25. Forced Outage Hours	242.9	267.9	15,492.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

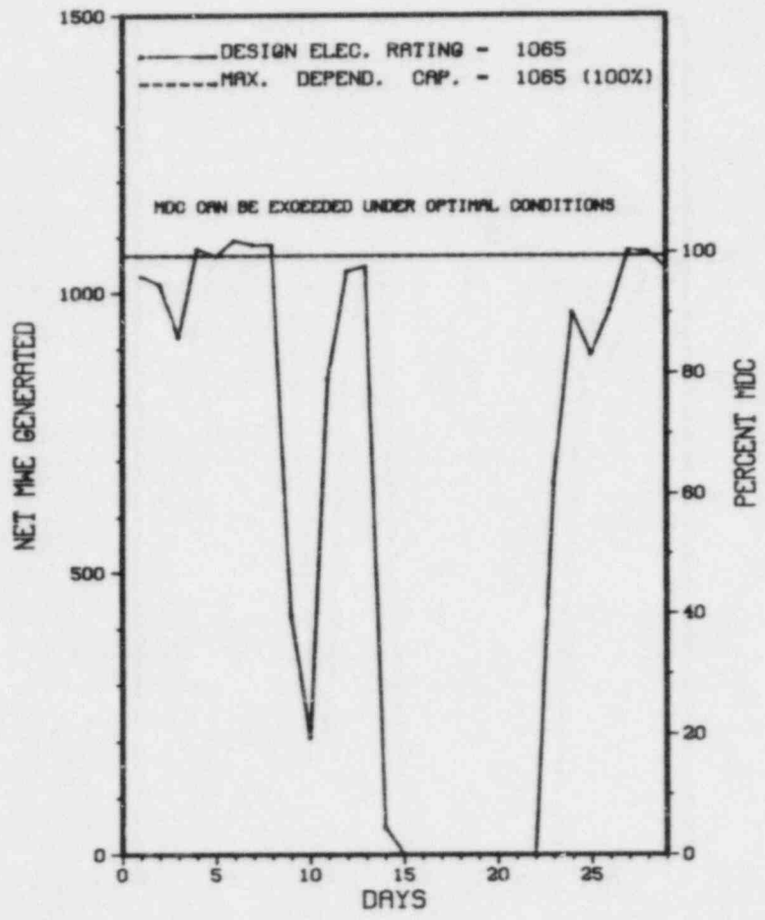
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BROWNS FERRY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BROWNS FERRY 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

BROWNS FERRY 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
270	02/02/84	S	0.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
271	02/09/84	F	26.2	B	3				REACTOR SCRAM DURING THE PERFORMANCE OF SI 4.1.A.10.
272	02/14/84	F	199.4	A	2				REACTOR SCRAM FOR MODIFICATIONS TO RHRSW AIR RELEASE VALVES.
273	02/22/84	F	16.4	A	3				REACTOR SCRAM (OFF LINE) WHEN THE HIGH PRESSURE TURBINE FIRST STAGE PRESSURE EXCEEDED 142 PSIG WITH THE TSV'S CLOSED.
274	02/29/84	F	0.9	H	3				REACTOR SCRAM DUE TO POSSIBLE BUMPING OF PANEL 25-6A.

SUMMARY

BROWNS FERRY 1 OPERATED WITH 1 REDUCTION AND 4 OUTAGES DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* BROWNS FERRY 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....LIMESTONE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI NW OF
DECATUR, ALA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...AUGUST 17, 1973

DATE ELEC ENER 1ST GENER...OCTOBER 15, 1973

DATE COMMERCIAL OPERATE....AUGUST 1, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....TENNESSEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....500A CHESTNUT STREET TOWER II
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. PAULK

LIC ING PROJ MANAGER.....R. CLARK
D ET NUMBER.....50-259

LICEN & DATE ISSUANCE....DPR-33, DECEMBER 20, 1973

PUBL DCUMENT ROOM.....ATHENS PUBLIC LIBRARY
SOUTH AND FORREST
ATHENS, ALABAMA 35611

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JANUARY 23-27 (84-04): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF STARTUP TESTING, NEW FUEL RECEIPT, PLANT TOURS, AND MSIV AIR STATUS. OF THE (4) FOUR AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS.

ENFORCEMENT SUMMARY

10 CFR 50.54(Q) REQUIRES THAT NUCLEAR POWER REACTOR LICENSEES FOLLOW AND MAINTAIN IN EFFECT EMERGENCY PLANS WHICH MEET THE REQUIREMENTS OF APPENDIX E TO 10 CFR PART 50 AND THE PLANNING STANDARDS OF 50.47(B). TECHNICAL SPECIFICATION 6.3(A)(8) REQUIRES THAT DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECKOFF LISTS COVERING RADIOLOGICAL EMERGENCY PLAN IMPLEMENTING PROCEDURES SHALL BE PREPARED, APPROVED AND ADHERED TO. SECTION (B)(10) OF 10 CFR 50.47 REQUIRES THAT THE LICENSEE'S EMERGENCY PLANS SHALL INCLUDE INFORMATION TO DEMONSTRATE COMPLIANCE WITH THE FOLLOWING: A RANGE OF PROTECTIVE ACTIONS HAVE BEEN DEVELOPED FOR THE PLUME EXPOSUREPATHWAY EPZ FOR EMERGENCY WORKERS AND THE PUBLIC. GUIDELINES FOR THE CHOICE OF PROTECTIVE ACTIONS DURING AN EMERGENCY, CONSISTENT WITH FEDERAL GUIDANCE, ARE DEVELOPED AND IN PLACE, AND PROTECTIVE ACTIONS FOR THE INGESTION EXPOSURE PATHWAY EPZ APPROPRIATE TO THE LOCALE HAVE BEEN DEVELOPED. THE FEDERAL GUIDANCE ON PROTECTIVE ACTIONS TO BE RECOMMENDED TO OFFSITE OFFICIALS FOR GENERAL EMERGENCIES IS ADDRESSED IN APPENDIX 1 OF NUREG-0654/FEMA-REP-1, REVISION 1, ENTITLED "CRITERIA FOR PREPARATION AND EVALUATION OF RADIOLOGICAL EMERGENCY PREPAREDNESS IN SUPPORT OF NUCLEAR POWER PLANTS." THIS GUIDANCE IS CLARIFIED BY IE INFORMATION NOTICE NO. 83-28: "CRITERIA FOR PROTECTIVE ACTION RECOMMENDATIONS FOR GENERAL EMERGENCIES". CONTRARY TO THE ABOVE,

ENFORCEMENT SUMMARY

THE LICENSEE HAS FAILED TO INCORPORATE THE ABOVE GUIDANCE IN THE IMPLEMENTING PROCEDURES IN THAT, UNTIL THE RESPONSE ORGANIZATION IS MOBILIZED (I.E. CECC AND MSECC), THERE IS NO MECHANISM OR REQUIREMENT FOR CONSIDERING PROTECTIVE ACTION RECOMMENDATIONS OTHER THAN FOR SHELTERING IN THE PLUME EPZ. 10 CFR 50.54(Q) REQUIRES THAT NUCLEAR POWER REACTOR LICENSEES FOLLOW AND MAINTAIN IN EFFECT EMEERGENCY PLANS WHICH MEET THE REQUIREMENTS OF APPENDIX E TO 10 CFR PART 50 AND THE PLANNING STANDARDS OF 50.47(B). SECTION IV.B OF APPENDIX E REQUIRES THAT A LICENSEE'S EMERGENCY PLANS SHALL INCLUDE INFORMATION TO DEMONSTRATE COMPLIANCE WITH THE FOLLOWING: THE MEANS TO BE USED FOR DETERMINING THE MAGNITUDE OF AND FOR CONTINUALLY ASSESSING THE IMPACT OF THE RELEASE OF RADIOACTIVE MATERIAL SHALL BE DESCRIBED, INCLUDING EMERGENCY ACTION LEVELS THAT ARE TO BE USED AS CRITERIA FOR NOTIFICATION AND PARTICIPATION OF LOCAL AND STATE AGENCIES, THE COMMISSION, AND OTHER FEDERAL AGENCIES, AND THE EMERGENCY ACTION LEVELS THAT ARE TO BE USED FOR DETERMINING WHEN AND WHAT TYPE OF PROTECTIVE MEASURES SHOULD BE CONSIDERED WITHIN AND OUTSIDE THE SITE BOUNDARY TO PROTECT HEALTH AND SAFETY. 10 CFR 50.47(B)(15) REQUIRES THAT THOSE WHO MAY BE CALLED ON TO ASSIST IN AN EMERGENCY BE PROVIDED RADIOLOGICAL EMERGENCRESPONSE TRAINING. SECTION 4.1.1 OF THE BROWNS FERRY NUCLEAR POWER PLANT EMERGENCY PLAN STATES IN PART THAT THE SITE EMERGENCY DIRECTOR HAS THE AUTHORITY TO RECOMMEND PROTECTIVE ACTIONS TO OFFSITE AUTHORITIES. SECTION 4.1 STATES IN PART THAT IN THE EVENT OF AN ACCIDENT, THE SHIFT ENGINEER IS INITIALLY THE SITE EMERGENCY DIRECTOR. CONTRARY TO THE ABOVE, SHIFT ENGINEERS, INITIALLY THE SITE EMERGENCY DIRECTORS, WERE INCAPABLE OF DETERMINING WHEN AND WHAT TYPE OF PROTECTIVE MEASURES SHOULD BE CONSIDERED OUTSIDE THE SITE BOUNDARY TO PROTECT HEALTH AND SAFETY.
(8340 4)

10 CFR 50, APPENDIX B, CRITERION VI (AS IMPLEMENTED BY TVA'S QA TOPICAL REPORT TVA TR 75-1 PARAGRAPH 17.2.6 AND OPERATIONAL QUALITY ASSURANCE MANUAL PART III SECTION 1.1) REQUIRES THAT MEASURES SHALL BE ESTABLISHED TO CONTROL THE ISSUANCE OF DOCUMENTS, INCLUDING CHANGES THERETO, WHICH PRESCRIBE ALL ACTIVITIES AFFECTING QUALITY. THESE MEASURES SHALL ASSURE THAT DOCUMENTS, INCLUDING CHANGES, ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED. CONTRARY TO THE ABOVE, SEVERAL EXAMPLES WERE NOTED WHERE THIS REQUIREMENT WAS NOT MET IN THAT THE MUSCLE SHOALS EMERGENCY CONTROL CENTER IMPLEMENTING PROCEDURES DOCUMENT (MSECC IPD) REVISIONS WERE NOT ADEQUATELY IMPLEMENTED. EXAMPLES ARE LISTED: (1) IP-4 (NOTIFICATION PROCEDURE) WAS DELETED OCTOBER, 1982; HOWEVER, IP-4 HAS NOT BEEN REMOVED FROM THE MSECC IPD AT THE FOLLOWING LCOATIONS: BROWNS FERRY TSC (TECHNICAL SUPPORT CENTER) AND BROWNS FERRY FILE ROOM. (2) IP-5 (TRAINING/DRILL REQUIREMENTS) WAS DELETED JUNE 1983; HOWEVER, IP-5 HAS NOT BEEN REMOVED FROM CONTROL COPIES OF MSECC IPD AT THE FOLLOWING LOCATIONS: MUSCLE SHOALS EMERGENCY CONTROL CENTER AND BROWNS FERRY FILE ROOM.
(8340 5)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERIA I & XVI AS IMPLEMENTED BY THE LICENSEES QA PROGRAM (TVA-TR75-1A) OQA IS NOT ENSURING EFFECTIVE EXECUTION OF THE QA PROGRAM IN THAT ALL CONDITIONS ADVERSE TO QUALITY HAVE NOT BEEN PROMPTLY CORRECTED. THE CURRENT COMPOSITE OPEN ITEM LIST REVIEW SUMMARY CONTAINS 1 OUTSTANDING ITEM FROM 1979, 16 OUTSTANDING ITEMS FROM 1981 AND 65 OUTSTANDING ITEMS FROM 1982. REGION II HAS ISSUED FOUR VIOLATIONS SINCE FEBRUARY 1981 FOR FAILURE TO TAKE PROMPT CORRECTIVE ACTION BY MECHANISMS DEFINED WITHIN THE QA PROGRAM. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION III AS IMPLEMENTED BY THE LICENSEE'S QA PROGRAM (TVA-TR75-1A), THE CONSTRUCTION ENGINEERING BRANCH HAS NOT ESTABLISHED MEASURES TO CONTROL DESIGN INTERFACES BETWEEN PARTICIPATING ORGANIZATIONS. THIS WAS IDENTIFIED IN THE LICENSEE'S AUDIT 83V-26 AND AGAIN IN AUDIT 83V-73. BOTH OF THESE AUDITS WERE CONDUCTED ON DIFFERENT VENDORS. CONTRARY TO BROWNS FERRY TECHNICAL SPECIFICATION (TS) 6.10.C, SEQUOYAH TS 6.5.2.10.C, THE LICENSEES ACCEPTED QA PROGRAM'S ENDORSEMENT OF REGULATORY GUIDE 1.144 AND ANSI N45.2.12 - 1974 OR 1977, PARAGRAPH 4.4.6 MULTIPLE EXAMPLES WERE IDENTIFIED OF FAILURE TO ISSUE AUDITS WITHIN REQUIRED TIMEFRAMES. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII AS IMPLEMENTED BY THE LICENSEES QA PROGRAM (TVA-TR75-1A), REGULATORY GUIDE 1.144, ANSI AND N45.2.12, PARAGRAPH 4.5.1, NUMEROUS EXAMPLES WERE IDENTIFIED WHERE THE AUDITED ORGANIZATION DID NOT RESPOND TO AUDITS FINDINGS WITHIN REQUIRED TIMEFRAMES. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION II AS IMPLEMENTED BY THE LICENSEES QA PROGRAM, REGULATORY GUIDE 1.146, AND ANSI N45.2.23, MEASURES HAD NOT BEEN ESTABLISHED TO REQUIRE VERIFICATION OF MINIMUM CREDITS NEEDED TO BE A LEAD AUDITOR.
(8353 4)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVII AS IMPLEMENTED BY THE LICENSEE'S QA PROGRAM, REGULATORY GUIDE 1.88, AND ANSI N45.2.9 RECORDS WERE NOT MAINTAINED TO DEMONSTRATE THAT ALL AUDITORS AND LEAD AUDITORS WERE QUALIFIED TO PERFORM SAFETY-RELATED QA AUDITS. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII IMPLEMENTED BY THE LICENSEE'S QA PROGRAM, REGULATORY GUIDE 1.144, AND ANSI N45.2.12, PROCEDURE DID NOT DELINEATE THAT PERSONS CONTACTED DURING THE AUDIT BE IDENTIFIED IN THE AUDIT REPORT.

ENFORCEMENT SUMMARY

(8353 5)

CONTRARY TO TECHNICAL SPECIFICATION 4.6.H.6 THE LICENSEE DID NOT FUNCTIONALLY TEST AN ADDI NAL SAMPLE OF 10% OF THE BERGEN-PATERSON HYDRAULIC SNUBBERS FOR EACH OF TWO SNUBBERS WHICH FAILED THE FUNCTIONAL TES.
(8356 4)

TECHNICAL SPECIFICATION 6.3.A.10 REQUIRES THAT DETAILED WRITTEN PROCEDURES BE PREPARED, APPROVED, AND ADHERED TO AS RELATED TO FIRE PROTECTION PROCEDURES. MECHANICAL MAINTENANCE INSTRUCTION (MMI) 122 IMPLEMENTS A HIGH PRESSURE FIRE PROTECTION SYSTEM FLUSH AND STRAINER INSPECTION PROGRAM TO ASSURE SYSTEM OPERABILITY. CONTRARY TO THE ABOVE, THE REQUIREMENT WAS NOT MET IN THAT PROCEDURAL STEPS FOR THE FIXED SPRAY SYSTEM FLUSH WERE NOT FOLLOWED DURING OBSERVATION BY THE INSPECTOR ON DECEMBER 7, 1983, IN THAT NO PROCEDURE WAS AVAILABLE AT THE WORK SITE TO ASSURE PROCEDURE COMPLIANCE. ADDITIONALLY, MMI 122, PART 1 (FIXED SPRAY FLUSH AND STRAINER INSPECTION), WAS INADEQUATE AND COULD NOT BE PERFORMED AS WRITTEN. THE PROCEDURE LISTED ISOLATION VALVES NOT ACTUALLY INSTALLED IN THE SYSTEM. THE PROCEDURE ALSO REQUIRED ACTUATION OF THE DELUGE VALVE THAT, IF ACTUATED, WOULD SPRAY PLANT EQUIPMENT. MMI 122 HAS BEEN CONDUCTED MONTHLY SINCE APRIL 1981, WITH NO APPARENT PROBLEMS NOTED BY PLANT PERSONNEL. THE PART 1 DATA SHEET FOR MMI 122, ADDITIONALLY, DOES NOT INDICATE COMPLETION OF THE QUARTERLY CLEANING OF SYSTEM STRAINERS AS REQUIRED BY MMI 122. TECHNICAL SPECIFICATION 6.3.A.7 REQUIRES THAT DETAILED WRITTEN PROCEDURES RELATED TO HEALTH PHYSICS AND RADIATION CONTROL PRACTICES BE FOLLOWED. RADIATION CONTROL INSTRUCTION 9 REQUIRES THAT SPECIAL WORK PERMIT (SWP) INSTRUCTIONS BE FOLLOWED. CONTRARY TO THE ABOVE, THE REQUIREMENT WAS NOT MET IN THAT ON DECEMBER 13, 1983, TWO OPERATORS WERE OBSERVED IN THE UNIT 1 N.W. QUADRANT, 519 LEVEL, TO NOT BE IN COMPLIANCE WITH THE REQUIREMENTS OF SWP 1-1-18285 WHICH FORBADE ENTRY INTO THE HIGHLY CONTAMINATED N.W. QUADRANT WITHOUT A SPECIAL SWP. A SPECIAL SWP WAS NOT AVAILABLE FOR ENTRY. TECHNICAL SPECIFICATION 6.3.D.2 REQUIRES EACH HIGH RADIATION AREA IN WHICH THE INTENSITY OF THE RADIATION IS GREATER THAN 1,000 MREM/HR. SHALL BE PROVIDED WITH LOCKED DOORS TO PREVENT UNAUTHORIZED ENTRY. CONTRARY TO THE ABOVE, THE REQUIREMENT WAS NOT MET IN THAT ON DECEMBER 19, 1983, THE 1-B REACTOR CLEANUP PUMP ROOM WAS NOT LOCKED AND WAS NOT POSITIVELY CONTROLLED TO PREVENT UNAUTHORIZED ENTRY. THIS ROOM IS POSTED AS A HIGH RADIATION AREA. A RADIOLOGICAL SURVEY WAS CONDUCTED ON DECEMBER 20, 1983, AND THE GENERAL AREA READING WAS 1,200 MREM/HR.

(8357 4)

FAILURE TO CONTROL ACCESS TO FUEL-HANDLING AREA AS COMMITTED TO IN APPROVED PHYSICAL SECURITY PLAN.
(8359 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

INTERGRANULAR STRESS CORROSION CRACKING (IGSCC) PROBLEMS IN RHR, CORE SPRAY AND OTHER PIPING HAS NECESSITATED WELD OVERLAY REPAIRS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

OTHER ITEMS

ROUTINE POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: JANUARY 23-27, 1984 +

INSPECTION REPORT NO: 50-259/84-04 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-260 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: TED THOM (205) 729-0834

4. Licensed Thermal Power (MWt): 3293

5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152

6. Design Electrical Rating (Net MWe): 1065

7. Maximum Dependable Capacity (Gross MWe): 1098

8. Maximum Dependable Capacity (Net MWe): 1065

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	78,913.0
13. Hours Reactor Critical	494.6	1,149.8	51,113.7
14. Rx Reserve Shtdwn Hrs	201.4	290.2	14,190.5
15. Hrs Generator On-Line	465.0	1,105.2	49,598.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,426,819	3,459,852	143,604,897
18. Gross Elec Ener (MWH)	478,980	1,139,280	47,736,568
19. Net Elec Ener (MWH)	464,527	1,106,575	46,365,178
20. Unit Service Factor	66.8	76.8	62.9
21. Unit Avail Factor	66.8	76.8	62.9
22. Unit Cap Factor (MDC Net)	62.7	72.2	55.2
23. Unit Cap Factor (DER Net)	62.7	72.2	55.2
24. Unit Forced Outage Rate	33.2	17.5	24.6
25. Forced Outage Hours	231.0	233.8	16,288.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

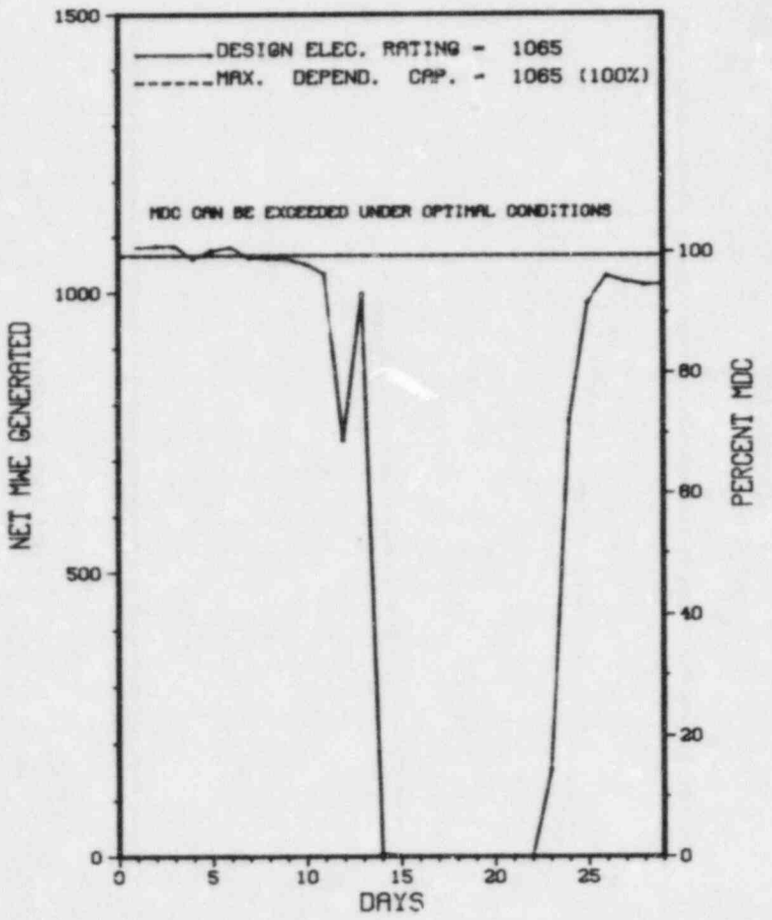
JULY, 1984 REFUEL

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BROWNS FERRY 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BROWNS FERRY 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

BROWNS FERRY 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
289	02/11/84	F	0.0	B	5				DERATED FOR "A" & "B" RECIRCULATION PUMP MG SET REPLACEMENT.
290	02/14/84	F	191.8	H	2				RX SCRAM FOR MODIFICATIONS ON RHRSW AIR RELEASE VALVES AND TO PLACE MAIN TRANSFORMER IN SERVICE AND TAKE SPARE TRANSFORMER OUT OF SERVICE.
291	02/22/84	F	39.2	H	3				RX SCRAM DUE TO HIGH CONTROL ROD WORTH.

SUMMARY

BROWNS FERRY 2 OPERATED WITH 2 OUTAGES AND 1 REDUCTION DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

F A C I L I T Y D A T A

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....LIMESTONE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI NW OF
DECATUR, ALA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JULY 20, 1974

DATE ELEC ENER 1ST GENER...AUGUST 28, 1974

DATE COMMERCIAL OPERATE....MARCH 1, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....TENNESSEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....500A CHESTNUT STREET TOWER II
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
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NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. PAULK

LICENSING PROJ MANAGER.....R. CLARK
DOCKET NUMBER.....50-260

LICENSE & DATE ISSUANCE....DPR-52, AUGUST 2, 1974

PUBLIC DOCUMENT ROOM.....ATHENS PUBLIC LIBRARY
SOUTH AND FORREST
ATHENS, ALABAMA 35611

I N S P E C T I O N S T A T U S

INSPECTION SUMMARY

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(8340 5)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERIA I & XVI AS IMPLEMENTED BY THE LICENSEES QA PROGRAM (TVA-TR75-1A) OQA IS NOT ENSURING EFFECTIVE EXECUTION OF THE QA PROGRAM IN THAT ALL CONDITIONS ADVERSE TO QUALITY HAVE NOT BEEN PROMPTLY CORRECTED. THE CURRENT COMPOSITE OPEN ITEM LIST REVIEW SUMMARY CONTAINS 1 OUTSTANDING ITEM FROM 1979, 16 OUTSTANDING ITEMS FROM 1981 AND 65 OUTSTANDING ITEMS FROM 1982. REGION II HAS ISSUED FOUR VIOLATIONS SINCE FEBRUARY 1981 FOR FAILURE TO TAKE PROMPT CORRECTIVE ACTION BY MECHANISMS DEFINED WITHIN THE QA PROGRAM. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION III AS IMPLEMENTED BY THE LICENSEE'S QA PROGRAM (TVA-TR75-1A), THE CONSTRUCTION ENGINEERING BRANCH HAS NOT ESTABLISHED MEASURES TO CONTROL DESIGN INTERFACES BETWEEN PARTICIPATING ORGANIZATIONS. THIS WAS IDENTIFIED IN THE LICENSEE'S AUDIT 83V-26 AND AGAIN IN AUDIT 83V-73. BOTH OF THESE AUDITS WERE CONDUCTED ON DIFFERENT VENDORS. CONTRARY TO BROWNS FERRY TECHNICAL SPECIFICATION (TS) 6.10.C, SEQUOYAH TS 6.5.2.10.C, THE LICENSEES ACCEPTED QA PROGRAM'S ENDORSEMENT OF REGULATORY GUIDE 1.144 AND ANSI N45.2.12 - 1974 OR 1977, PARAGRAPH 4.4.6 MULTIPLE EXAMPLES WERE IDENTIFIED OF FAILURE TO ISSUE AUDITS WITHIN REQUIRED TIMEFRAMES. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII AS IMPLEMENTED BY THE LICENSEES QA PROGRAM (TVA-TR75-1A), REGULATORY GUIDE 1.144, ANSI AND N45.2.12, PARAGRAPH 4.5.1, NUMEROUS EXAMPLES WERE IDENTIFIED WHERE THE AUDITED ORGANIZATION DID NOT RESPOND TO AUDITS FINDINGS WITHIN REQUIRED TIMEFRAMES. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION II AS IMPLEMENTED BY THE LICENSEES QA PROGRAM, REGULATORY GUIDE 1.146, AND ANSI N45.2.23, MEASURES HAD NOT BEEN ESTABLISHED TO REQUIRE VERIFICATION OF MINIMUM CREDITS NEEDED TO BE A LEAD AUDITOR.
(8353 4)

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ENFORCEMENT SUMMARY

(8353 5)

TECHNICAL SPECIFICATION 6.3.A.10 REQUIRES THAT DETAILED WRITTEN PROCEDURES BE PREPARED, APPROVED, AND ADHERED TO AS RELATED TO FIRE PROTECTION PROCEDURES. MECHANICAL MAINTENANCE INSTRUCTION (MMI) 122 IMPLEMENTS A HIGH PRESSURE FIRE PROTECTION SYSTEM FLUSH AND STRAINER INSPECTION PROGRAM TO ASSURE SYSTEM OPERABILITY. CONTRARY TO THE ABOVE, THE REQUIREMENT WAS NOT MET IN THAT PROCEDURAL STEPS FOR THE FIXED SPRAY SYSTEM FLUSH WERE NOT FOLLOWED DURING OBSERVATION BY THE INSPECTOR ON DECEMBER 7, 1983, IN THAT NO PROCEDURE WAS AVAILABLE AT THE WORK SITE TO ASSURE PROCEDURE COMPLIANCE. ADDITIONALLY, MMI 122, PART 1 (FIXED SPRAY FLUSH AND STRAINER INSPECTION), WAS INADEQUATE AND COULD NOT BE PERFORMED AS WRITTEN. THE PROCEDURE LISTED ISOLATION VALVES NOT ACTUALLY INSTALLED IN THE SYSTEM. THE PROCEDURE ALSO REQUIRED ACTUATION OF THE DELUGE VALVE THAT, IF ACTUATED, WOULD SPRAY PLANT EQUIPMENT. MMI 122 HAS BEEN CONDUCTED MONTHLY SINCE APRIL 1981, WITH NO APPARENT PROBLEMS NOTED BY PLANT PERSONNEL. THE PART 1 DATA SHEET FOR MMI 122, ADDITIONALLY, DOES NOT INDICATE COMPLETION OF THE QUARTERLY CLEANING OF SYSTEM STRAINERS AS REQUIRED BY MMI 122.

(8357 4)

FAILURE TO CONTROL ACCESS TO FUEL-HANDLING AREA AS COMMITTED TO IN APPROVED PHYSICAL SECURITY PLAN.

(8359 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JANUARY 23-27, 1984 +

INSPECTION REPORT NO: 50-260/84-04 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-296 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: TED THOM (205) 729-0834
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1065
7. Maximum Dependable Capacity (Gross MWe): 1098
8. Maximum Dependable Capacity (Net MWe): 1065
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	61,368.0
13. Hours Reactor Critical	.0	.0	43,088.6
14. Rx Reserve Shtdwn Hrs	.0	.0	3,878.1
15. Hrs Generator On-Line	.0	.0	42,194.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	126,285,520
18. Gross Elec Ener (MWH)	0	0	41,597,620
19. Net Elec Ener (MWH)	0	0	40,376,156
20. Unit Service Factor	.0	.0	68.8
21. Unit Avail Factor	.0	.0	68.8
22. Unit Cap Factor (MDC Net)	.0	.0	61.8
23. Unit Cap Factor (DER Net)	.0	.0	61.8
24. Unit Forced Outage Rate	.0	.0	10.8
25. Forced Outage Hours	.0	.0	5,091.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

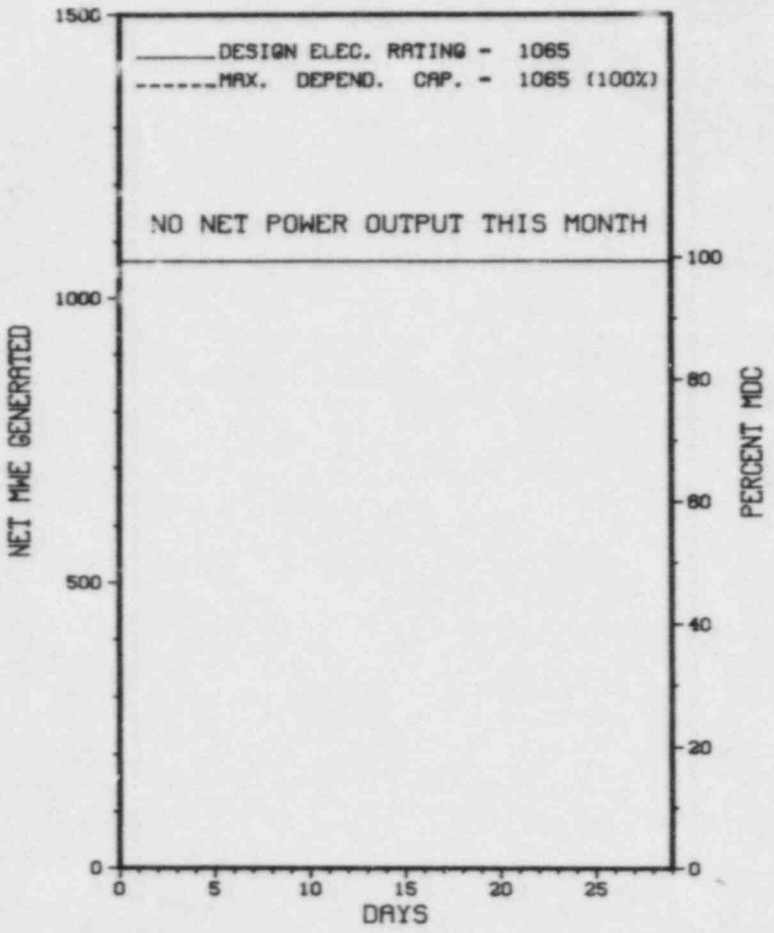
NONE

27. If Currently Shutdown Estimated Startup Date: 08/15/84

****
* BROWNS FERRY 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BROWNS FERRY 3



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

BROWNS FERRY 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
140	09/07/83	S	696.0	C	4				EOC-5 REFUEL OUTAGE CONTINUES.

SUMMARY

BROWNS FERRY 3 REMAINS SHUTDOWN IN A CONTINUING REFUELING OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....LIMESTONE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI NW OF
DECATUR, ALA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...AUGUST 8, 1976

DATE ELEC ENER 1ST GENER...SEPTEMBER 12, 1976

DATE COMMERCIAL OPERATE....MARCH 1, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....TENNESSEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....500A CHESTNUT STREET TOWER II
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. PAULK

LICENSING PROJ MANAGER.....R. CLARK
DOCKET NUMBER.....50-296

LICENSE & DATE ISSUANCE....DPR-68, AUGUST 18, 1976

PUBLIC DOCUMENT ROOM.....ATHENS PUBLIC LIBRARY
SOUTH AND FORREST
ATHENS, ALABAMA 35611

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JANUARY 23-27 (84-04): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF STARTUP TESTING, NEW FUEL RECEIPT, PLANT TOURS, AND MSIV REPAIR STATUS. OF THE (4) FOUR AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS.

ENFORCEMENT SUMMARY

10 CFR 50.54(Q) REQUIRES THAT NUCLEAR POWER REACTOR LICENSEES FOLLOW AND MAINTAIN IN EFFECT EMERGENCY PLANS WHICH MEET THE REQUIREMENTS OF APPENDIX E TO 10 CFR PART 50 AND THE PLANNING STANDARDS OF 50.47(B). TECHNICAL SPECIFICATION 6.3(A)(8) REQUIRES THAT DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECKOFF LISTS COVERING RADIOLOGICAL EMERGENCY PLAN IMPLEMENTING PROCEDURES SHALL BE PREPARED, APPROVED AND ADHERED TO. SECTION (B)(10) OF 10 CFR 50.47 REQUIRES THAT THE LICENSEE'S EMERGENCY PLANS SHALL INCLUDE INFORMATION TO DEMONSTRATE COMPLIANCE WITH THE FOLLOWING: A RANGE OF PROTECTIVE ACTIONS HAVE BEEN DEVELOPED FOR THE PLUME EXPOSUREPATHWAY EPZ FOR EMERGENCY WORKERS AND THE PUBLIC. GUIDELINES FOR THE CHOICE OF PROTECTIVE ACTIONS DURING AN EMERGENCY, CONSISTENT WITH FEDERAL GUIDANCE, ARE DEVELOPED AND IN PLACE, AND PROTECTIVE ACTIONS FOR THE INGESTION EXPOSURE PATHWAY EPZ APPROPRIATE TO THE LOCALE HAVE BEEN DEVELOPED. THE FEDERAL GUIDANCE ON PROTECTIVE ACTIONS TO BE RECOMMENDED TO OFFSITE OFFICIALS FOR GENERAL EMERGENCIES IS ADDRESSED IN APPENDIX 1 OF NUREG-0654/FEMA-REP-1, REVISION 1, ENTITLED "CRITERIA FOR PREPARATION AND EVALUATION OF RADIOLOGICAL EMERGENCY PREPAREDNESS IN SUPPORT OF NUCLEAR POWER PLANTS." THIS GUIDANCE IS CLARIFIED BY IE INFORMATION NOTICE NO. 83-28: "CRITERIA FOR PROTECTIVE ACTION RECOMMENDATIONS FOR GENERAL EMERGENCIES". CONTRARY TO THE ABOVE,

ENFORCEMENT SUMMARY

THE LICENSEE HAS FAILED TO INCORPORATE THE ABOVE GUIDANCE IN THE IMPLEMENTING PROCEDURES IN THAT, UNTIL THE RESPONSE ORGANIZATION IS MOBILIZED (I.E. CECC AND MSECC), THERE IS NO MECHANISM OR REQUIREMENT FOR CONSIDERING PROTECTIVE ACTION RECOMMENDATIONS OTHER THAN FOR SHELTERING IN THE PLUME EPZ. 10 CFR 50.54(Q) REQUIRES THAT NUCLEAR POWER REACTOR LICENSEES FOLLOW AND MAINTAIN IN EFFECT EMEERGENCY PLANS WHICH MEET THE REQUIREMENTS OF APPENDIX E TO 10 CFR PART 50 AND THE PLANNING STANDARDS OF 50.47(B). SECTION IV.B OF APPENDIX E REQUIRES THAT A LICENSEE'S EMERGENCY PLANS SHALL INCLUDE INFORMATION TO DEMONSTRATE COMPLIANCE WITH THE FOLLOWING: THE MEANS TO BE USED FOR DETERMINING THE MAGNITUDE OF AND FOR CONTINUALLY ASSESSING THE IMPACT OF THE RELEASE OF RADIOACTIVE MATERIAL SHALL BE DESCRIBED, INCLUDING EMERGENCY ACTION LEVELS THAT ARE TO BE USED AS CRITERIA FOR NOTIFICATION AND PARTICIPATION OF LOCAL AND STATE AGENCIES, THE COMMISSION, AND OTHER FEDERAL AGENCIES, AND THE EMERGENCY ACTION LEVELS THAT ARE TO BE USED FOR DETERMINING WHEN AND WHAT TYPE OF PROTECTIVE MEASURES SHOULD BE CONSIDERED WITHIN AND OUTSIDE THE SITE BOUNDARY TO PROTECT HEALTH AND SAFETY. 10 CFR 50.47(B)(15) REQUIRES THAT THOSE WHO MAY BE CALLED ON TO ASSIST IN AN EMERGENCY BE PROVIDED RADIOLOGICAL EMERGENCRESPONSE TRAINING. SECTION 4.1.1 OF THE BROWNS FERRY NUCLEAR POWER PLANT EMERGENCY PLAN STATES IN PART THAT THE SITE EMERGENCY DIRECTOR HAS THE AUTHORITY TO RECOMMEND PROTECTIVE ACTIONS TO OFFSITE AUTHORITIES. SECTION 4.1 STATES IN PART THAT IN THE EVENT OF AN ACCIDENT, THE SHIFT ENGINEER IS INITIALLY THE SITE EMERGENCY DIRECTOR. CONTRARY TO THE ABOVE, SHIFT ENGINEERS, INITIALLY THE SITE EMERGENCY DIRECTORS, WERE INCAPABLE OF DETERMINING WHEN AND WHAT TYPE OF PROTECTIVE MEASURES SHOULD BE CONSIDERED OUTSIDE THE SITE BOUNDARY TO PROTECT HEALTH AND SAFETY.
(8340 4)

10 CFR 50, APPENDIX B, CRITERION VI (AS IMPLEMENTED BY TVA'S QA TOPICAL REPORT TVA TR 75-1 PARAGRAPH 17.2.6 AND OPERATIONAL QUALITY ASSURANCE MANUAL PART III SECTION 1.1) REQUIRES THAT MEASURES SHALL BE ESTABLISHED TO CONTROL THE ISSUANCE OF DOCUMENTS, INCLUDING CHANGES THERETO, WHICH PRESCRIBE ALL ACTIVITIES AFFECTING QUALITY. THESE MEASURES SHALL ASSURE THAT DOCUMENTS, INCLUDING CHANGES, ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED. CONTRARY TO THE ABOVE, SEVERAL EXAMPLES WERE NOTED WHERE THIS REQUIREMENT WAS NOT MET IN THAT THE MUSCLE SHOALS EMERGENCY CONTROL CENTER IMPLEMENTING PROCEDURES DOCUMENT (MSECC IPD) REVISIONS WERE NOT ADEQUATELY IMPLEMENTED. EXAMPLES ARE LISTED: (1) IP-4 (NOTIFICATION PROCEDURE) WAS DELETED OCTOBER, 1982; HOWEVER, IP-4 HAS NOT BEEN REMOVED FROM THE MSECC IPD AT THE FOLLOWING LCOATIONS: BROWNS FERRY TSC (TECHNICAL SUPPORT CENTER) AND BROWNS FERRY FILE ROOM. (2) IP-5 (TRAINING/DRILL REQUIREMENTS) WAS DELETED JUNE 1983; HOWEVER, IP-5 HAS NOT BEEN REMOVED FROM CONTROL COPIES OF MSECC IPD AT THE FOLLOWING LOCATIONS: MUSCLE SHOALS EMERGENCY CONTROL CENTER AND BROWNS FERRY FILE ROOM.
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(8353 4)

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ENFORCEMENT SUMMARY

(8353 5)

TECHNICAL SPECIFICATION 6.3.A.10 REQUIRES THAT DETAILED WRITTEN PROCEDURES BE PREPARED, APPROVED, AND ADHERED TO AS RELATED TO FIRE PROTECTION PROCEDURES. MECHANICAL MAINTENANCE INSTRUCTION (MMI) 122 IMPLEMENTS A HIGH PRESSURE FIRE PROTECTION SYSTEM FLUSH AND STRAINER INSPECTION PROGRAM TO ASSURE SYSTEM OPERABILITY. CONTRARY TO THE ABOVE, THE REQUIREMENT WAS NOT MET IN THAT PROCEDURAL STEPS FOR THE FIXED SPRAY SYSTEM FLUSH WERE NOT FOLLOWED DURING OBSERVATION BY THE INSPECTOR ON DECEMBER 7, 1983, IN THAT NO PROCEDURE WAS AVAILABLE AT THE WORK SITE TO ASSURE PROCEDURE COMPLIANCE. ADDITIONALLY, MMI 122, PART 1 (FIXED SPRAY FLUSH AND STRAINER INSPECTION), WAS INADEQUATE AND COULD NOT BE PERFORMED AS WRITTEN. THE PROCEDURE LISTED ISOLATION VALVES NOT ACTUALLY INSTALLED IN THE SYSTEM. THE PROCEDURE ALSO REQUIRED ACTUATION OF THE DELUGE VALVE THAT, IF ACTUATED, WOULD SPRAY PLANT EQUIPMENT. MMI 122 HAS BEEN CONDUCTED MONTHLY SINCE APRIL 1981, WITH NO APPARENT PROBLEMS NOTED BY PLANT PERSONNEL. THE PART 1 DATA SHEET FOR MMI 122, ADDITIONALLY, DOES NOT INDICATE COMPLETION OF THE QUARTERLY CLEANING OF SYSTEM STRAINERS AS REQUIRED BY MMI 122.

(8357 4)

FAILURE TO CONTROL ACCESS TO FUEL-HANDLING AREA AS COMMITTED TO IN APPROVED PHYSICAL SECURITY PLAN.

(8359 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN 9/6/83 TO PERFORM IGSCC INSPECTION.

LAST IE SITE INSPECTION DATE: JANUARY 23-27, 1984 +

INSPECTION REPORT NO: 50-296/84-04 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-325 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: FRANCES HARRISON (919) 457-9521

4. Licensed Thermal Power (MWt): 2436

5. Nameplate Rating (Gross MWe): 963 X 0.9 = 867

6. Design Electrical Rating (Net MWe): 821

7. Maximum Dependable Capacity (Gross MWe): 815

8. Maximum Dependable Capacity (Net MWe): 790

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	60,961.0
13. Hours Reactor Critical	641.1	1,362.8	37,760.8
14. Rx Reserve Shtdwn Hrs	.0	.0	1,647.1
15. Hrs Generator On-Line	614.3	1,336.0	35,424.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,447,183	3,172,514	71,599,800
18. Gross Elec Ener (MWH)	486,205	1,066,181	23,613,229
19. Net Elec Ener (MWH)	473,376	1,038,422	22,652,253
20. Unit Service Factor	88.3	92.8	58.1
21. Unit Avail Factor	88.3	92.8	58.1
22. Unit Cap Factor (MDC Net)	86.1	91.3	47.0
23. Unit Cap Factor (DER Net)	82.8	87.8	45.3
24. Unit Forced Outage Rate	11.7	7.2	20.4
25. Forced Outage Hours	81.7	104.0	9,023.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

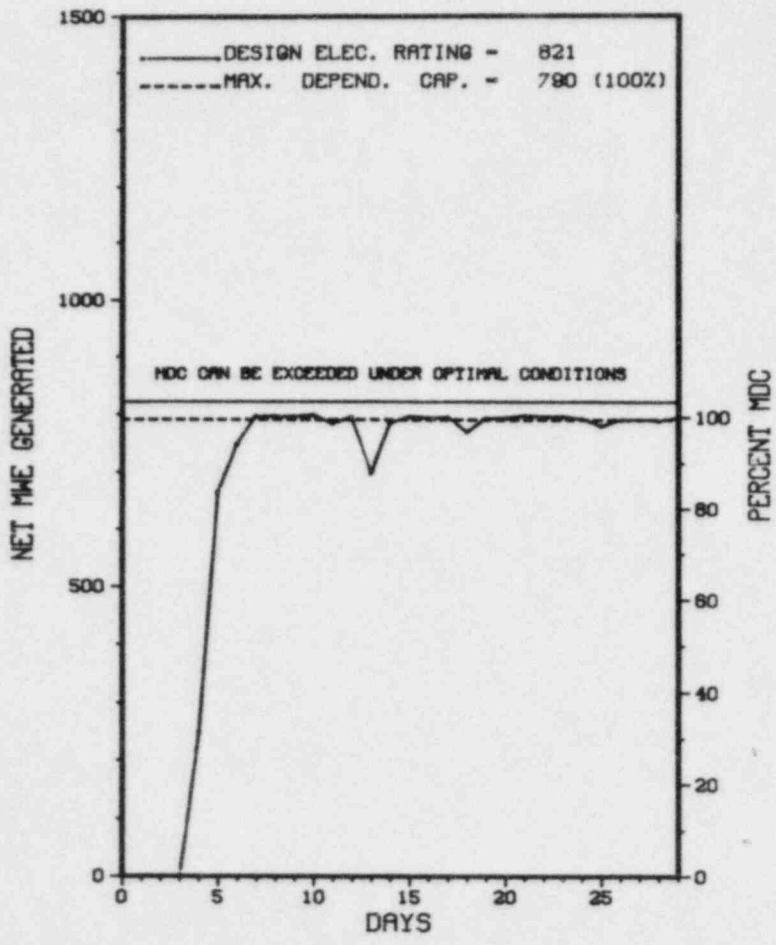
REPLACING SEALS ON RECIRC PUMP - 03/03/84- 1 WK.

27. If Currently Shutdown Estima ; Startup Date: N/A

****
* BRUNSWICK 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BRUNSWICK 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* BRUNSWICK 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-006	01/31/84	F	81.7	A	4		XX	VESSEL	UNIT SEPARATED FROM GRID TO FIND SOURCE OF DRYWELL FLOOR AND EQUIPMENT DRAIN LEAKAGE AND REPAIR.
84-009	02/13/84	F	0.0	B	5		CB	PUMPXX	REMOVED 1A RS FEEDPUMP FROM SERVICE TO ALLOW I&C TO CHECK OUT MGU.
84-010	02/18/84	F	0.0	A	5				REDUCED POWER DUE TO HI D/F ACROSS RADWASTE AND APRM UPSCALE.
84-011	02/25/84	S	0.0	B	5				CONTROL ROD OPERABILITY COMP. 100% ROD PATTERN.

****
* SUMMARY *
****

BRUNSWICK 1 RETURNED ONLINE FEBRUARY 4TH FROM A REPAIR OUTAGE AND OPERATED ROUTINELY THE REMAINDER OF THE MONTH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BRUNSWICK 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....BRUNSWICK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...3 MI N OF
SOUTHPORT, NC

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...OCTOBER 8, 1976

DATE ELEC ENER 1ST GENER...DECEMBLR 4, 1976

DATE COMMERCIAL OPERATE....MARCH 18, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CAPE FEAR RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CAROLINA POWER & LIGHT

CORPORATE ADDRESS.....P. O. BOX 1551
RALEIGH, NORTH CAROLINA 27602

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BROWN & ROOT

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. MYERS

LICENSING PROJ MANAGER.....S. MACKAY
DOCKET NUMBER... 50-325

LICENSE & DATE ISSUANCE....DPR-71, NOVEMBER 12, 1976

PUBLIC DOCUMENT ROOM.....SOUTHPORT-BRUNSWICK COUNTY LIBRARY
108 W. MOORE STREET
SOUTHPORT, NORTH CAROLINA 28461

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 15, 1983-JANUARY 15, 1984 (83-42): THIS ROUTINE SAFETY INSPECTION INVOLVED 69 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE, MAINTENANCE, OPERATIONAL SAFETY VERIFICATION, ESF SYSTEM WALKDOWN, IN-OFFICE LICENSEE EVENT REPORTS REVIEW, INDEPENDENT INSPECTION, AND PLANT TRANSIENTS. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 3.7.7.2 FOR BRUNSWICK, UNIT 1, REQUIRES THAT THE DELUGE SYSTEM FOR THE STANDBY GAS TREATMENT SYSTEM (SBGTS) TRAINS 1A AND 1B, BE OPERABLE WHENEVER THE SBGTS SYSTEM IS REQUIRED TO BE OPERABLE. ACTION STATEMENT "A" OF TS 3.7.7.2 REQUIRES THAT, WITH THE DELUGE SYSTEM INOPERABLE, A CONTINUOUS FIRE WATCH WITH BACKUP SUPPRESSION EQUIPMENT BE ESTABLISHED WITHIN ONE HOUR. CONTRARY TO THE ABOVE, THE UNIT 1 DELUGE SYSTEM FOR THE SBGTS TRAINS 1A AND 1B WAS RENDERED INOPERABLE WHEN VALVE 1-WW-V207 WAS SHUT DURING THE PERIOD OF FEBRUARY 11 TO MARCH 13, 1983 AND A CONTINUOUS FIRE WATCH WAS NOT ESTABLISHED. DURING THIS TIME THE PLANT WAS IN A CONDITION WHICH REQUIRED THE SBGTS TO BE OPERABLE. IN A RESPONSE DATED MAY 24, 1982 DESCRIBING CORRECTIVE ACTION TAKEN WITH RESPECT TO A NOTICE OF VIOLATION DATED APRIL 2, 1982, THE LICENSEE STATED THAT TABLES I AND IA OF VOLUME XI OF THE LICENSEE'S PLANT OPERATING MANUAL HAD BEEN REVISED TO ASSURE THAT ALL Q-LIST EQUIPMENT WAS CORRECTLY IDENTIFIED ON BOTH TABLES. THESE TABLES ARE USED BY LICENSEE PERSONNEL AS A REFERENCE TO DETERMINE IF A PLANT INSTRUMENT IS A Q-ITEM (I.E., SAFETY-RELATED). CONTRARY TO THE ABOVE, ON MAY 24, 1982 AND AS OF NOVEMBER 1982, NUMEROUS DISCREPANCIES EXISTED BETWEEN TABLES I AND IA OF VOLUME XI OF THE LICENSEE'S PLANT OPERATING MANUAL AND CONSEQUENTLY THE TABLES DID NOT CORRECTLY IDENTIFY ALL Q-LIST

ENFORCEMENT SUMMARY

EQUIPMENT. FOR EXAMPLE, TABLE IA DID NOT IDENTIFY PRESSURE SWITCH AS Q-LIST EQUIPMENT WHILE TABLE I DID. THUS, THE LICENSEE'S RESPONSE TO THE NRC OF MAY 24, 1982 CONTAINED A MATERIAL FALSE STATEMENT WITHIN THE MEANING OF SECTION 186 OF THE ATOMIC ENERGY ACT OF 1954, AS AMENDED. THE LICENSEE'S STATEMENT WAS FALSE IN THAT TABLES I AND IA DID NOT CORRECTLY IDENTIFY ALL Q-LIST EQUIPMENT. THE STATEMENT IS MATERIAL FOR THE NRC WOULD HAV.. TAKEN FURTHER REGULATORY ACTION TO CORRECT THE TABLES HAD IT KNOW THEY WERE INCORRECT.
(8311 3)

TECHNICAL SPECIFICATION 3.7.8 FOR BRUNSWICK, UNITS 1 AND 2 REQUIRES THAT ALL FIRE BARRIER PENETRATIONS, FIRE DOORS AND FIRE DAMPERS, IN FIRE ZONE BOUNDARIES PROTECTING SAFETY-RELATED AREAS, SHALL BE FUNCTIONAL. TECHNICAL SPECIFICATION 3.7.8 ACTION STATEMENT "A" REQUIRES THAT, WITH ONE OR MORE OF THE FIRE BARRIER PENETRATIONS NON-FUNCTIONAL, WITHIN ONE HOUR A CONTINUOUS FIRE WATCH MUST BE ESTABLISHED ON AT LEAST ONE SIDE OF THE AFFECTED PENETRATION OR VERIFY THE OPERABILITY OF FIRE DETECTORS ON AT LEAST ONE SIDE OF THE NON-FUNCTIONAL FIRE BARRIER AND ESTABLISH AN HOURLY FIRE WATCH PATROL. CONTRARY TO THE ABOVE, DURING THE PERIOD OF FEBRUARY 13, TO APRIL 5, 1983, FIRE BARRIER PENETRATIONS PROTECTING SAFETY-RELATED AREAS IN UNITS 1 AND 2 WERE NON-FUNCTIONAL AND THE ASSOCIATED FIRE DETECTORS WERE INOPERABLE WITHOUT CONTINUOUS FIRE WATCH. TECHNICAL SPECIFICATION 6.8.1.F FOR UNITS 1 AND 2 REQUIRES THAT WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING FIRE PROTECTION PROGRAM IMPLEMENTATION. CONTRARY TO THE ABOVE, PROCEDURES COVERING THE FIRE PROTECTION PROGRAM FOR UNITS 1 AND 2 WERE NOT ADEQUATELY IMPLEMENTED AS DEMONSTRATED BY THE FOLLOWING EXAMPLES: (A) FIRE PROTECTION SURVEILLANCE PROCEDURE PT-35.7 WAS INADEQUATELY IMPLEMENTED ON FEBRUARY 12, 20, AND 26 AND MARCH 7, 1983 IN THAT THE POSITION VALVE WW-V207 WAS NOT PROPERLY IDENTIFIED. THE VALVE WAS SHUT. THE POSITION WAS RECORDED AS BEING LOCKED OPEN. (B) FIRE PROTECTION SURVEILLANCE PROCEDURE PT-35.1 WAS INADEQUATELY IMPLEMENTED ON FEBRUARY 14, 21, AND 28 AND MARCH 7, 1983 IN THAT VALVE WW-V207 WAS NOT PROPERLY VERIFIED AS LOCKED OPEN. (C) FIRE PROTECTION PROCEDURES PT-35.16 AND PT-35.18 WERE NOT BEING ADEQUATELY IMPLEMENTED IN THAT SURVEILLANCE TO ENSURE THE FUNCTIONAL STATUS OF FIRE BARRIER PENETRATIONS WERE NOT BEING PERFORMED IN ACCORDANCE WITH THE ACCEPTANCE CRITERIA SPECIFIED IN THESE TESTS. TECHNICAL SPECIFICATION 6.9.1.8.6 FOR UNIT 1 REQUIRES THE REPORTING WITHIN 24 HOURS BY TELEPHONE AND CONFIRMATION BY TELEGRAPH, MAILGRAM, OR FACSIMILE TRANSMISSION TO THE DIRECTOR OF THE REGIONAL OFFICE OR HIS DESIGNEE NO LATER THAN THE FIRST WORKING DAY FOLLOWING OPERATION OF THE UNIT OR AFFECTED SYSTEM WHEN ANY PARAMETER OR OPERATION SUBJECT TO AN LCO IS LESS CONSERVATIVE THAN THE LEAST CONSERVATIVE ASPECT OF THE LCO ESTABLISHED IN THE TECHNICAL SPECIFICATION. CONTRARY TO THE ABOVE, THE LCO VIOLATION DESCRIBED IN ITEM B ABOVE WAS A REPORTABLE EVENT WHICH WAS NOT REPORTED TO THE NRC REGION II WITHIN 24 HOURS. TECHNICAL SPECIFICATION 6.9.2 FOR UNIT 1 REQIRES A SPECIAL REPORT TO BE ISSUED WITHIN 30 DAYS AFTER A FIRE BARRIER PENETRATION HAS BEEN INOPERABLE FOR 7 DAYS. CONTRARY TO THE ABOVE, IN FOUR INSTANCES, ONCE ON JANUARY 26, TWICE ON FEBRUARY 19, AND ONCE ON MARCH 12, 1983, FIRE BARRIER PENETRATIONS WERE INOPERABLE FOR MORE THAN 7 DAYS AND THE REQUIRED SPECIAL REPORTS WERE NOT SUBMITTED.
(8311 4)

TECHNICAL SPECIFICATION 6.8.1.A, REQUIRES WRITTEN PROCEDURES BE IMPLEMENTED FOR ACTIVITIES REFERENCED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER, 1972. THE GUIDE ITEM, H.2.B 30, REFERENCES PROCEDURES FOR PROCESS RADIATION MONITORING CALIBRATION. CONTRARY TO THE ABOVE, THE STEAM JET AIR EJECTOR RADIATION MONITOR CALIBRATION WAS NOT IMPLEMENTED, IN THAT STEP VII.B OF THE PROCEDURE PT-70, WAS NOT CORRECTLY PERFORMED. TECHNICAL SPECIFICATION 6.8.1.A, REQUIRES WRITTEN PROCEDURES BE ESTABLISHED FOR ACTIVITIES REFERENCED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972. THE GUIDE ITEM B.9, REFERENCES PROCEDURES FOR SHUTDOWN. CONTRARY TO THE ABOVE, GP-01, WHICH IS USED FOR SHUTDOWN, WAS INADEQUATELY ESTABLISHED, IN THAT A PRECAUTION WAS NOT PROVIDED TO VERIFY THAT THE IRM'S ARE NOT TRIPPED PRIOR TO CHANGING THE MODE SWITCH FROM "STARTUP" MODE.
(8341 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: DECEMBER 15, 1983 - JANUARY 15, 1984 +

INSPECTION REPORT NO: 50-325/83-42 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-324 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: FRANCES HARRISON (919) 457-9521

4. Licensed Thermal Power (MWt): 2436

5. Nameplate Rating (Gross MWe): 963 X 0.9 = 867

6. Design Electrical Rating (Net MWe): 821

7. Maximum Dependable Capacity (Gross MWe): 815

8. Maximum Dependable Capacity (Net MWe): 790

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	72,985.0
13. Hours Reactor Critical	671.6	1,338.4	46,065.7
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	662.1	1,301.0	43,086.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,468,832	2,744,288	81,321,002
18. Gross Elec Ener (MWH)	487,062	910,369	27,020,067
19. Net Elec Ener (MWH)	471,735	881,155	25,908,773
20. Unit Service Factor	95.1	90.3	59.0
21. Unit Avail Factor	95.1	90.3	59.0
22. Unit Cap Factor (MDC Net)	85.8	77.5	44.9
23. Unit Cap Factor (DER Net)	82.6	74.5	43.2
24. Unit Forced Outage Rate	4.9	2.7	17.6
25. Forced Outage Hours	33.9	35.5	9,638.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

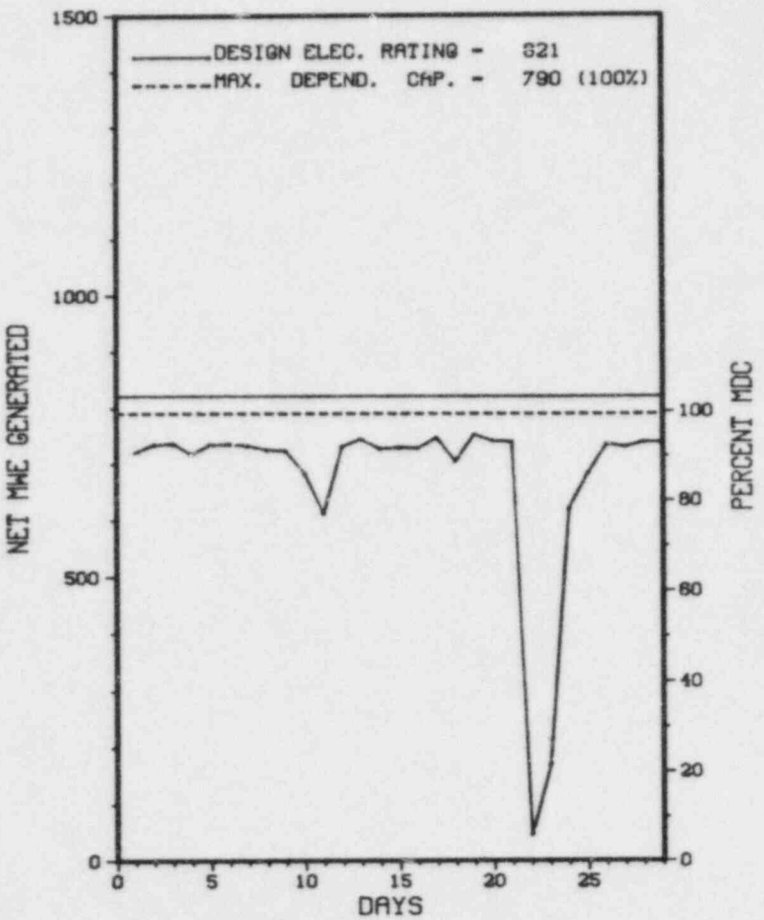
OFF-GAS/CONDENSER RETUBING 3/10 - 10/15/84.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BRUNSWICK 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BRUNSWICK 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

BRUNSWICK 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-008	02/10/84	S	0.0	B	5				REDUCED POWER FOR SCRAM TESTING.
84-011	02/18/84	S	0.0	B	5				REDUCTION FOR ROD IMPROVEMENT.
84-012	02/18/84	S	0.0	B	5				REDUCED POWER FOR DEBRIS FILTER BACKWASH AS PER OP-29.
84-013	02/17/84	F	0.0	A	5				REDUCED POWER IN ORDER TO ESTABLISH A STABLE CONDENSER VACUUM GREATER THAN 26 INCHES.
84-016	02/22/84	F	33.9	A	3		CD	VALVOP	RX SCRAM- STEAM LINE "D" OUTBOARD MSIV SHUTTING DURING PERFOMRING OF PT A22.
84-018	02/24/84	F	0.0	A	5		CH	PUMPXX	LOST 2B REACTOR FEEDPUMP.

SUMMARY

BRUNSWICK 2 OPERATED WITH 5 REDUCTIONS AND 1 OUTAGE DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
		9-Other	(LER) File (NUREG-0161)

****
* BRUNSWICK 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....BRUNSWICK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...3 MI N OF
SOUTHPORT, NC

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MARCH 20, 1975

DATE ELEC ENER 1ST GENER...APRIL 29, 1975

DATE COMMERCIAL OPERATE....NOVEMBER 3, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CAPE FEAR RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CAROLINA POWER & LIGHT

CORPORATE ADDRESS.....411 FAYETTEVILLE STREET
RALEIGH, NORTH CAROLINA 27602

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BROWN & ROOT

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. MYERS

LICENSING PROJ MANAGER.....S. MACKAY
DOCKET NUMBER.....50-324

LICENSE & DATE ISSUANCE....DPR-62, DECEMBER 27, 1974

PUBLIC DOCUMENT ROOM.....SOUTHPORT-BRUNSWICK COUNTY LIBRARY
108 W. MOORE STREET
SOUTHPORT, NORTH CAROLINA 28461

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 15, 1983-JANUARY 15, 1984 (83-42): THIS ROUTINE SAFETY INSPECTION INVOLVED 68 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE, MAINTENANCE, OPERATIONAL SAFETY VERIFICATION, ESF SYSTEM WALKDOWN, IN-OFFICE LICENSEE EVENT REPORTS REVIEW, INDEPENDENT INSPECTION, AND PLANT TRANSIENTS. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 3.7.7.2 FOR BRUNSWICK, UNIT 1, REQUIRES THAT THE DELUGE SYSTEM FOR THE STANDBY GAS TREATMENT SYSTEM (SBGTS) TRAINS 1A AND 1B, BE OPERABLE WHENEVER THE SBGTS SYSTEM IS REQUIRED TO BE OPERABLE. ACTION STATEMENT "A" OF TS 3.7.7.2 REQUIRES THAT, WITH THE DELUGE SYSTEM INOPERABLE, A CONTINUOUS FIRE WATCH WITH BACKUP SUPPRESSION EQUIPMENT BE ESTABLISHED WITHIN ONE HOUR. CONTRARY TO THE ABOVE, THE UNIT 1 DELUGE SYSTEM FOR THE SBGTS TRAINS 1A AND 1B WAS RENDERED INOPERABLE WHEN VALVE 1-WW-V207 WAS SHUT DURING THE PERIOD OF FEBRUARY 11 TO MARCH 13, 1983 AND A CONTINUOUS FIRE WATCH WAS NOT ESTABLISHED. DURING THIS TIME THE PLANT WAS IN A CONDITION WHICH REQUIRED THE SBGTS TO BE OPERABLE.
(8311 3)

IN A RESPONSE DATED MAY 24, 1982 DESCRIBING CORRECTIVE ACTION TAKEN WITH RESPECT TO A NOTICE OF VIOLATION DATED APRIL 2, 1982, THE LICENSEE STATED THAT TABLES I AND IA OF VOLUME XI OF THE LICENSEE'S PLANT OPERATING MANUAL HAD BEEN REVISED TO ASSURE THAT ALL Q-LIS. EQUIPMENT WAS CORRECTLY IDENTIFIED ON BOTH TABLES. THESE TABLES ARE USED BY LICENSEE PERSONNEL AS A REFERENCE TO DETERMINE

ENFORCEMENT SUMMARY

IF A PLANT INSTRUMENT IS A Q-ITEM (I.E., SAFETY-RELATED). CONTRARY TO THE ABOVE, ON MAY 24, 1982 AND AS OF NOVEMBER 1982, NUMEROUS DISCREPANCIES EXISTED BETWEEN TABLES I AND IA OF VOLUME XI OF THE LICENSEE'S PLANT OPERATING MANUAL AND CONSEQUENTLY THE TABLES DID NOT CORRECTLY IDENTIFY ALL Q-LIST EQUIPMENT. FOR EXAMPLE, TABLE IA DID NOT IDENTIFY PRESSURE SWITCH AS Q-LIST EQUIPMENT WHILE TABLE I DID. THUS, THE LICENSEE'S RESPONSE TO THE NRC OF MAY 24, 1982 CONTAINED A MATERIAL FALSE STATEMENT WITHIN THE MEANING OF SECTION 186 OF THE ATOMIC ENERGY ACT OF 1954, AS AMENDED. THE LICENSEE'S STATEMENT WAS FALSE IN THAT TABLES I AND IA DID NOT CORRECTLY IDENTIFY ALL Q-LIST EQUIPMENT. THE STATEMENT IS MATERIAL FOR THE NRC WOULD HAVE TAKEN FURTHER REGULATORY ACTION TO CORRECT THE TABLES HAD IT KNOW THEY WERE INCORRECT. TECHNICAL SPECIFICATION 3.7.8 FOR BRUNSWICK, UNITS 1 AND 2 REQUIRES THAT ALL FIRE BARRIER PENETRATIONS, FIRE DOORS AND FIRE DAMPERS, IN FIRE ZONE BOUNDARIES PROTECTING SAFETY-RELATED AREAS, SHALL BE FUNCTIONAL. TECHNICAL SPECIFICATION 3.7.8 ACTION STATEMENT "A" REQUIRES THAT, WITH ONE OR MORE OF THE FIRE BARRIER PENETRATIONS NON-FUNCTIONAL, WITHIN ONE HOUR A CONTINUOUS FIRE WATCH MUST BE ESTABLISHED ON AT LEAST ONE SIDE OF THE AFFECTED PENETRATION OR VERIFY THE OPERABILITY OF FIRE DETECTORS ON AT LEAST ONE SIDE OF THE NON-FUNCTIONAL FIRE BARRIER AND ESTABLISH AN HOURLY FIRE WATCH PATROL. CONTRARY TO THE ABOVE, DURING THE PERIOD OF FEBRUARY 13, TO APRIL 5, 1983, FIRE BARRIER PENETRATIONS PROTECTING SAFETY-RELATED AREAS IN UNITS 1 AND 2 WERE NON-FUNCTIONAL AND THE ASSOCIATED FIRE DETECTORS WERE INOPERABLE WITHOUT CONTINUOUS FIRE WATCH. TECHNICAL SPECIFICATION 6.8.1.F FOR UNITS 1 AND 2 REQUIRES THAT WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING FIRE PROTECTION PROGRAM IMPLEMENTATION. CONTRARY TO THE ABOVE, PROCEDURES COVERING THE FIRE PROTECTION PROGRAM FOR UNITS 1 AND 2 WERE NOT ADEQUATELY IMPLEMENTED AS DEMONSTRATED BY THE FOLLOWING EXAMPLES: (A) FIRE PROTECTION SURVEILLANCE PROCEDURE PT-35.7 WAS INADEQUATELY IMPLEMENTED ON FEBRUARY 12, 20, AND 26 AND MARCH 7, 1983 IN THAT THE POSITION VALVE WW-V207 WAS NOT PROPERLY IDENTIFIED. THE VALVE WAS SHUT. THE POSITION WAS RECORDED AS BEING LOCKED OPEN. (B) FIRE PROTECTION SURVEILLANCE PROCEDURE PT-35.1 WAS INADEQUATELY IMPLEMENTED ON FEBRUARY 14, 21, AND 28 AND MARCH 7, 1983 IN THAT VALVE WW-V207 WAS NOT PROPERLY VERIFIED AS LOCKED OPEN. (C) FIRE PROTECTION PROCEDURES PT-35.16 AND PT-35.18 WERE NOT BEING ADEQUATELY IMPLEMENTED IN THAT SURVEILLANCE TO ENSURE THE FUNCTIONAL STATUS OF FIRE BARRIER PENETRATIONS WERE NOT BEING PERFORMED IN ACCORDANCE WITH THE ACCEPTANCE CRITERIA SPECIFIED IN THESE TESTS. TECHNICAL SPECIFICATION 6.9.1.8.6 FOR UNIT 1 REQUIRES THE REPORTING WITHIN 24 HOURS BY TELEPHONE AND CONFIRMATION BY TELEGRAPH, MAILGRAM, OR FACSIMILE TRANSMISSION TO THE DIRECTOR OF THE REGIONAL OFFICE OR HIS DESIGNEE NO LATER THAN THE FIRST WORKING DAY FOLLOWING OPERATION OF THE UNIT OR AFFECTED SYSTEM WHEN ANY PARAMETER OR OPERATION SUBJECT TO AN LCO IS LESS CONSERVATIVE THAN THE LEAST CONSERVATIVE ASPECT OF THE LCO ESTABLISHED IN THE TECHNICAL SPECIFICATION. CONTRARY TO THE ABOVE, THE LCO VIOLATION DESCRIBED IN ITEM B ABOVE WAS A REPORTABLE EVENT WHICH WAS NOT REPORTED TO THE NRC REGION II WITHIN 24 HOURS. TECHNICAL SPECIFICATION 6.9.2 FOR UNIT 1 REQUIRES A SPECIAL REPORT TO BE ISSUED WITHIN 30 DAYS AFTER A FIRE BARRIER PENETRATION HAS BEEN INOPERABLE FOR 7 DAYS. CONTRARY TO THE ABOVE, IN FOUR INSTANCES, ONCE ON JANUARY 26, TWICE ON FEBRUARY 19, AND ONCE ON MARCH 12, 1983, FIRE BARRIER PENETRATIONS WERE INOPERABLE FOR MORE THAN 7 DAYS AND THE REQUIRED SPECIAL REPORTS WERE NOT SUBMITTED.
(8311 4)

TECHNICAL SPECIFICATION 6.8.1.A, REQUIRES WRITTEN PROCEDURES BE IMPLEMENTED FOR ACTIVITIES REFERENCED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER, 1972. THE GUIDE ITEM, H.2.B.(2), REFERENCES PROCEDURES FOR CALIBRATION OF CONTAINMENT ISOLATION TESTS. CONTRARY TO THE ABOVE, THE CALIBRATION PROCEDURE FOR THE REACTOR WATER CLEANUP SYSTEM ISOLATION ON HIGH AREA TEMPERATURE (PT 2.1.15PC), WAS NOT PROPERLY IMPLEMENTED IN THAT THE PROCEDURE FAILED TO TEST THE SYSTEM'S SENSOR. THIS RESULTED IN THE SURVEILLANCE FREQUENCY OF THE "A" TRIP SYSTEM OF THE RWCU SYSTEMS ISOLATION EXCEEDING ITS TECHNICAL SPECIFICATION ALLOWANCE OF 18 MONTHS.
(8342 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NONE.

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: DECEMBER 15, 1983 - JANUARY 15, 1984 +

INSPECTION REPORT NO: 50-324/83-42 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-317 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: EVELYN BEWLEY (301) 787-5365

4. Licensed Thermal Power (MWt): 2700

5. Nameplate Rating (Gross MWe): 1020 X 0.9 = 918

6. Design Electrical Rating (Net MWe): 845

7. Maximum Dependable Capacity (Gross MWe): 860

8. Maximum Dependable Capacity (Net MWe): 825

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	77,269.0
13. Hours Reactor Critical	669.0	1,395.5	61,362.4
14. Rx Reserve Shtdwn Hrs	.0	.0	1,887.9
15. Hrs Generator On-Line	669.0	1,389.6	60,135.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,771,548	3,669,547	147,811,[illegible]
18. Gross Elec Ener (MWH)	613,922	1,262,669	48,070,154
19. Net Elec Ener (MWH)	588,893	1,211,084	46,446,050
20. Unit Service Factor	96.1	96.5	77.8
21. Unit Avail Factor	96.1	96.5	77.8
22. Unit Cap Factor (MDC Net)	102.6	101.9	73.8*
23. Unit Cap Factor (DER Net)	100.1	99.5	71.1
24. Unit Forced Outage Rate	3.9	3.5	7.4
25. Forced Outage Hours	27.0	50.4	4,713.2

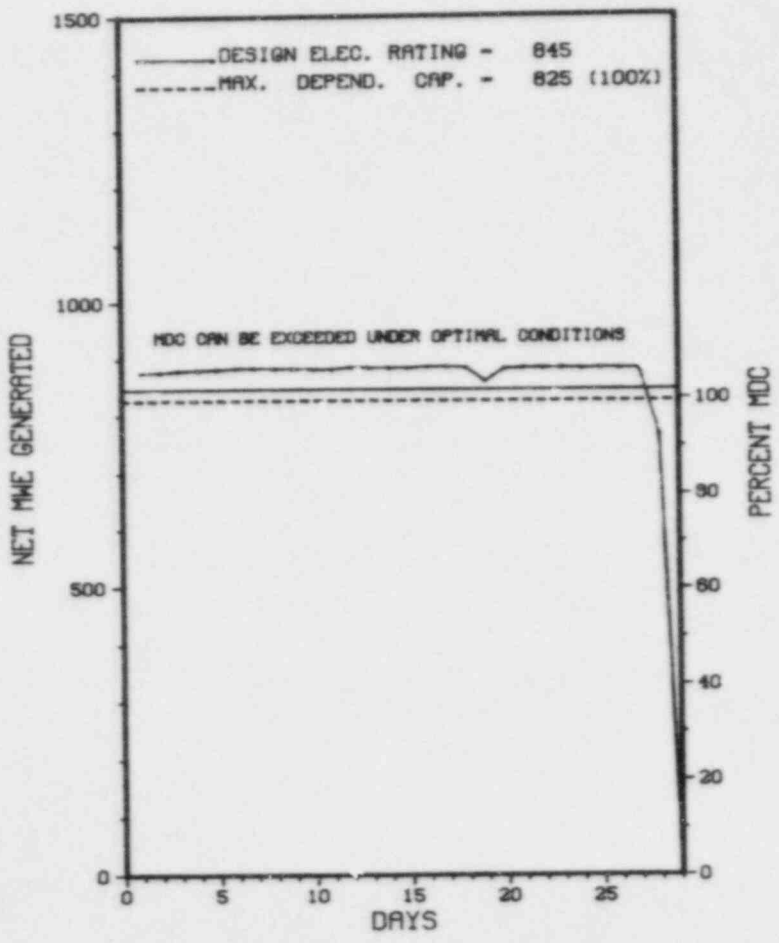
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: 03/15/84

****
* CALVERT CLIFFS 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* CALVERT CLIFFS 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-02	02/28/84	F	27.0	A	1		CB	PUMPXX	DUE TO LOSS OF TWO CHARGING PUMPS AND REPAIR LEAKING PRESSURIZER SAFETY VALVE.

****
* SUMMARY *
****

CALVERT CLIFFS 1 OPERATED WITH 1 OUTAGE DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* CALVERT CLIFFS 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MARYLAND

COUNTY.....CALVERT

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI S OF
ANNAPOLIS, MD

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 7, 1974

DATE ELEC ENER 1ST GENER...JANUARY 3, 1975

DATE COMMERCIAL OPERATE....MAY 8, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHESAPEAKE BAY

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....BALTIMORE GAS & ELEC

CORPORATE ADDRESS.....P.O. BOX 1475
BALTIMORE, MARYLAND 21203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....R. ARCHITZEL

LICENSING PROJ MANAGER.....D. JAFFE
DOCKET NUMBER.....50-317

LICENSE & DATE ISSUANCE....DPR-53, JULY 31, 1974

PUBLIC DOCUMENT ROOM.....CALVERT COUNTY LIBRARY
FOURTH STREET
PRINCE FREDERICK, MARYLAND 20678

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* CALVERT CLIFFS 1 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-318 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: EVELYN BEWLEY (310) 787-5365

4. Licensed Thermal Power (MWt): 2700

5. Nameplate Rating (Gross MWe): 1012 X 0.9 = 911

6. Design Electrical Rating (Net MWe): 845

7. Maximum Dependable Capacity (Gross MWe): 860

8. Maximum Dependable Capacity (Net MWe): 825

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	60,624.0
13. Hours Reactor Critical	696.0	1,440.0	51,367.8
14. Rx Reserve Shtdwn Hrs	.0	.0	958.1
15. Hrs Generator On-Line	696.0	1,440.0	50,555.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,787,897	3,778,858	125,620,551
18. Gross Elec Ener (MWH)	587,882	1,245,302	41,314,588
19. Net Elec Ener (MWH)	562,435	1,192,525	39,396,287
20. Unit Service Factor	100.0	100.0	83.4
21. Unit Avail Factor	100.0	100.0	83.4
22. Unit Cap Factor (MDC Net)	98.0	100.4	79.3*
23. Unit Cap Factor (DER Net)	95.6	98.0	76.9
24. Unit Forced Outage Rate	.0	.0	5.7
25. Forced Outage Hours	.0	.0	3,045.2

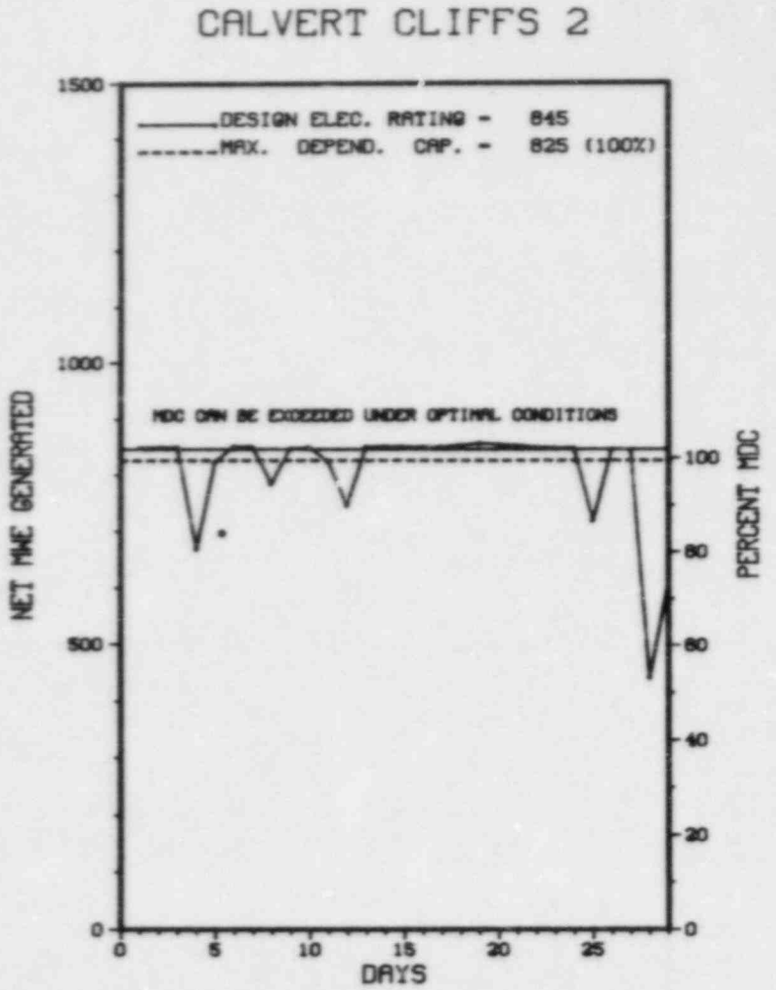
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

REFUELING & UNIT GENERAL INSPECTION - 04/84 - 10 WKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* CALVERT CLIFFS 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* CALVERT CLIFFS 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-01	02/28/84	F	0.0	B	5		CB	PUMPXX	ISOLATION OF AN INSTRUMENT LINE OF 22A REACTOR COOLANT PUMP SEAL. NO. 2 UNIT EXPERIENCED LOAD REDUCTION AT VARIOUS LOADS DUE TO MOISTURE SEPARATOR REHEATER TUBE LEAKS.

****
* SUMMARY *
****

CALVERT CLIFFS 2 OPERATED WITH 1 REDUCTION FOR MAINTENANCE DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* CALVERT CLIFFS 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MARYLAND

COUNTY.....CALVERT

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI S OF
ANNAPOLIS, MD

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 30, 1976

DATE ELEC ENER 1ST GENER...DECEMBER 7, 1976

DATE COMMERCIAL OPERATE....APRIL 1, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHESAPEAKE BAY

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....BALTIMORE GAS & ELEC

CORPORATE ADDRESS.....P.O. BOX 1475
BALTIMORE, MARYLAND 21203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....R. ARCHITZEL

LICENSING PROJ MANAGER.....D. JAFFE
DOCKET NUMBER.....50-318

LICENSE & DATE ISSUANCE....DPR-69, NOVEMBER 30, 1976

PUBLIC DOCUMENT ROOM.....CALVERT COUNTY LIBRARY
FOURTH STREET
PRINCE FREDERICK, MARYLAND 20678

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

INSPECTION STATUS - (CONTINUED)

****
* CALVERT CLIFFS 2 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

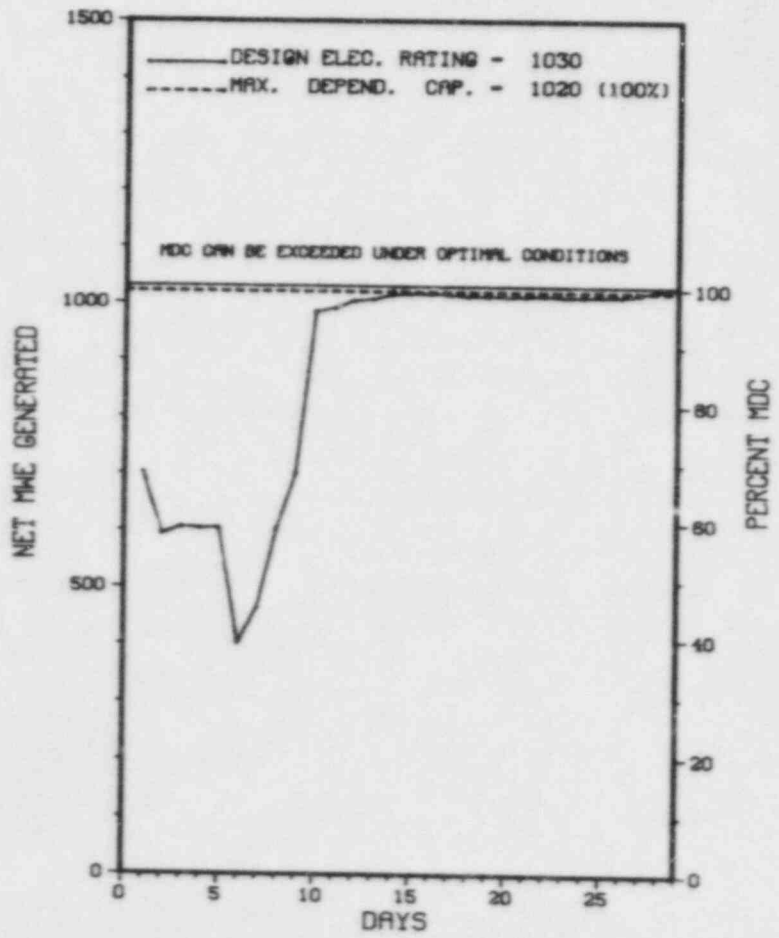
1. Docket: 50-315 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: W. T. GILLETT (616) 465-5901
4. Licensed Thermal Power (MWt): 3250
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1030
7. Maximum Dependable Capacity (Gross MWe): 1056
8. Maximum Dependable Capacity (Net MWe): 1020
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	80,328.0
13. Hours Reactor Critical	696.0	1,263.1	58,881.3
14. Rx Reserve Shtdwn Hrs	.0	.0	463.0
15. Hrs Generator On-Line	689.0	1,248.9	57,592.6
16. Unit Reserve Shtdwn Hrs	.0	.0	321.0
17. Gross Therm Ener (MWH)	1,940,152	3,683,162	167,764,776
18. Gross Elec Ener (MWH)	637,460	1,212,690	55,138,980
19. Net Elec Ener (MWH)	612,157	1,166,630	53,046,970
20. Unit Service Factor	99.0	86.7	73.6
21. Unit Avail Factor	99.0	86.7	73.6
22. Unit Cap Factor (MDC Net)	86.2	79.4	66.5
23. Unit Cap Factor (DER Net)	85.4	78.7	63.6
24. Unit Forced Outage Rate	1.0	13.3	7.9
25. Forced Outage Hours	7.0	191.1	4,271.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

COOK 1

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

Report Period FEB 1984 UNIT SHUTDOWNS / REDUCTIONS

****
* COOK 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
212	02/01/84	F	0.8	A	5		CH	PUMPXX	A POWER INCREASE FROM 79% TO 99% WAS IN PROGRESS WHEN AT 93% THE WEST MAIN FEED PUMP, MFP, DEVELOPED A VIBRATION PROBLEM. A POWER REDUCTION TO 59% WAS STARTED IMMEDIATELY TO REMOVE THE WEST MFP FROM SERVICE.
213	02/06/84	F	7.9	A	1		CH	HTEXCH	WHILE THE WEST MFP WAS STILL OUT OF SERVICE, THE EAST MFP TURBINE CONDENSER DEVELOPED A CONDENSER TUBE LEAK. TO EFFECT REPAIRS, THE UNIT HAD TO BE REMOVED FROM SERVICE. THE REACTOR WAS MAINTAINED CRITICAL AT 6% POWER. ONE LEAKING TUBE WAS IDENTIFIED AND PLUGGED.

****
* SUMMARY *
****

COOK 1 OPERATED WITH 1 REDUCTION AND 1 OUTAGE DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* COOK 1 *
****

F A C I L I T Y D A T A

FACILITY DESCRIPTION

LOCATION
STATE.....MICHIGAN

COUNTY.....BERRIEN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI S OF
BENTON HARBOR, MI

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JANUARY 18, 1975

DATE ELEC ENER 1ST GENER...FEBRUARY 10, 1975

DATE COMMERCIAL OPERATE....AUGUST 27, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....INDIANA & MICHIGAN ELECTRIC

CORPORATE ADDRESS.....1 RIVERSIDE PLAZA
COLUMBUS, OHIO 43216

CONTRACTOR
ARCHITECT/ENGINEER.....AMERICAN ELEC. POWER SERVICE CORP.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....AMERICAN ELEC. POWER SERVICE CORP.

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....E. SWANSON

LICENSING PROJ MANAGER.....D. WIGGINTON
DOCKET NUMBER.....50-315

LICENSE & DATE ISSUANCE....DPR-58, OCTOBER 25, 1974

PUBLIC DOCUMENT ROOM.....MAUDE PRESTON PALENSKE MEMORIAL LIBRARY
500 MARKET STREET
ST. JOSEPH, MICHIGAN 49085

I N S P E C T I O N S T A T U S

INSPECTION SUMMARY

INSPECTION ON DECEMBER 11, THROUGH JANUARY 20, (83-21): ROUTINE UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTOR OF LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS; OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; LICENSEE EVENT REPORTS; AND OPERATING ACTIVITIES. THE INSPECTION INVOLVED A TOTAL OF 180 INSPECTOR-HOURS BY TWO NRC INSPECTORS INCLUDING 34 INSPECTOR-HOURS OFF-SHIFTS. OF THE SIX AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE REMAINING AREA (FAILURE TO SUBMIT A SPECIAL REPORT).

INSPECTION ON JANUARY 23-27, (84-01): ROUTINE, ANNOUNCED INSPECTION OF THE DONALD C. COOK NUCLEAR PLANT EMERGENCY PREPAREDNESS EXERCISE INVOLVING OBSERVATIONS BY SIX NRC REPRESENTATIVES OF KEY FUNCTIONS AND LOCATIONS DURING THE EXERCISE. THE INSPECTION INVOLVED 116 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS AND THREE CONSULTANTS. NO ITEMS OF NONCOMPLIANCE, DEFICIENCIES, OR DEVIATIONS WERE IDENTIFIED.

MANAGEMENT MEETING ON JANUARY 31, (84-03): THE PROGRESS AND STATUS OF THE LICENSEE'S REGULATORY PERFORMANCE IMPROVEMENT PROGRAM. THE MEETING INVOLVED A TOTAL OF 16 MAN-HOURS BY 8 NRC PERSONNEL.

INSPECTION ON FEBRUARY 7, (84-04): REVIEW OF DOCUMENTATION AND WALK-DOWNS PERTAINING TO EQUIPMENT REFERRED TO IN INFORMATION LETTER 81-14 DATED FEBRUARY 10, 1981. THE INSPECTION UTILIZED A TOTAL OF 16 HRS ONSITE BY TWO NRC INSPECTORS. BASED UPON THIS REVIEW AND THE ASSOCIATED WALK-DOWNS OF THE AREAS AFFECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.5.2.7.E REQUIRES THE NUCLEAR SAFETY AND DESIGN REVIEW COMMITTEE (NSDRC) TO REVIEW "VIOLATIONS OF CODES, REGULATIONS, ORDERS,TECHNICAL SPECIFICATIONS, LICENSE REQUIREMENTS, OR INTERNAL PROCEDURES OR INSTRUCTIONS HAVING NUCLEAR SAFETY SIGNIFICANCE." CONTRARY TO THE ABOVE, THE REVIEWS PERFORMED BY THE NSDRC DID NOT INCLUDE ALL NRC INSPECTION FINDINGS, QA AUDIT FINDINGS, AND SIGNIFICANT NON-REPORTABLE CONDITION REPORTS WHICH INVOLVED VIOLATIONS OF TECHNICAL SPECIFICATIONS OR INTERNAL PROCEDURES HAVING NUCLEAR SAFETY SIGNIFICANCE. EXAMPLES OF NRC INSPECTION FINDINGS NOT REVIEWED BY THE NSDRC INCLUDED: (A) NONCOMPLIANCE IDENTIFIED IN NRC INSPECTION REPORT 315/81-08; 316/81-12 REGARDING ADHERENCE TO PROCEDURES GOVERNING QA DOCUMENTATION OF MATERIAL; (B) NONCOMPLIANCE REGARDING VIOLATION OF THE LIMITING CONDITION FOR OPERATION ON SPRAY ADDITIVE TANKS IDENTIFIED IN NRC INSPECTION REPORT 315/81-05; 316/81-05; (C) THREE SECURITY RELATED VIOLATIONS IDENTIFIED IN NRC INSPECTION REPORT 315/81-04; 316/81-04; AND (D) VIOLATIONS DESCRIBED IN PARAGRAPHS 5 AND 7 OF NRC INSPECTION REPORT 315/81-09, 316/81-13 REGARDING THE PERFORMANCE OF SURVEILLANCES AND ADHERENCE TO PROCEDURES.

10 CFR 50, APPENDIX B, CRITERION II, REQUIRES REGULAR REVIEWS OF THE STATUS AND ADEQUACY OF THE QUALITY ASSURANCE PROGRAM. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTION 1.2.2, REQUIRES THAT THE NSDRC REGULARLY ASSESS THE SCOPE, IMPLEMENTATION, AND EFFECTIVENESS OF THE QA PROGRAM TO COMPLY WITH 10 CFR 50, APPENDIX B, CRITERIA. ANSI N18.7-1976, PARAGRAPH 4.5, STATES IN PART, "PERIODIC REVIEW OF THE AUDIT PROGRAM SHALL BE PERFORMED BY THE INDEPENDENT REVIEW BODY OR BY A MANAGEMENT REPRESENTATIVE AT LEAST SEMIANNUALLY TO ASSURE THAT AUDITS ARE BEING ACCOMPLISHED IN ACCORDANCE WITH REQUIREMENTS OF TECHNICAL SPECIFICATIONS AND OF THIS STANDARD." CONTRARY TO THE ABOVE, THE NSDRC DID NOT PERIODICALLY REVIEW THE AUDIT PROGRAM, AUDIT REPORTS, AND AUDIT RESULTS IN ORDER TO ASSESS THE SCOPE, IMPLEMENTATION, AND EFFECTIVENESS OF THE QA PROGRAM. TECHNICAL SPECIFICATION 6.5.2.7.I REQUIRES THE NSDRC TO REVIEW THE "REPORTS AND MEETING MINUTES OF THE PNSRC." CONTRARY TO THE ABOVE, THE PLANT NUCLEAR SAFETY REVIEW COMMITTEE (PNSRC) REPORTS AND MINUTES WERE NOT REVIEWED BY THE NSDRC. TECHNICAL SPECIFICATION 6.5.2.8.A REQUIRES AN AUDIT UNDER THE COGNIZANCE OF THE NSDRC OF "THE CONFORMANCE OF FACILITY OPERATION TO PROVISIONS CONTAINED WITHIN THE TECHNICAL SPECIFICATION AND APPLICABLE LICENSE CONDITIONS AT LEAST ONCE PER 12 MONTHS." CONTRARY TO THE ABOVE, NSDRC AUDITS FAILED TO EXAMINE THE TECHNICAL SPECIFICATION LIMITING CONDITIONS FOR OPERATION OR THE NSDRC ACTIVITIES. 10 CFR 50, APPENDIX B, CRITERION XVIII, REQUIRES AUDITS TO DETERMINE THE EFFECTIVENESS OF THE LICENSEE'S QA PROGRAM. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTION 1.7.4.18, REQUIRES THAT THE NSDRC CONDUCT PERIODIC AUDITS OF PLANT OPERATIONS PURSUANT TO ESTABLISHED CRITERIA. ANSI N45.2.12-1977, TO WHICH THE LICENSEE IS COMMITTED, REQUIRES IN PARAGRAPH 4.4.4 THAT EACH AUDIT REPORT INCLUDE A SUMMARY OF RESULTS WHICH INCLUDES "AN EVALUATION STATEMENT REGARDING THE EFFECTIVENESS OF THE QUALITY ASSURANCE PROGRAM ELEMENTS WHICH WERE AUDITED." CONTRARY TO THE ABOVE, MOST OF THE NSDRC AUDIT REPORTS FOR AUDITS CONDUCTED OVER THE LAST SEVERAL YEARS CONTAINED NO EVALUATION STATEMENTS. 10 CFR 50, APPENDIX B, CRITERION V, REQUIRES THAT ACTIVITIES AFFECTING QUALITY BE ACCOMPLISHED IN ACCORDANCE WITH DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES. CRITERION XVI REQUIRES THAT MEASURES BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY ARE PROMPTLY IDENTIFIED AND CORRECTED. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTIONS 1.7.4.5 AND 1.7.4.16, REQUIRES THAT ACTIVITIES AFFECTING QUALITY BE CONDUCTED AS DIRECTED BY APPROPRIATE INSTRUCTIONS, PROCEDURES, OR DRAWINGS AND THAT CONDITIONS ADVERSE TO QUALITY BE REPORTED TO MANAGEMENT. ANSI N45.2.12-1977, PARAGRAPH 4.5.1, REQUIRES THAT "THE RESPONSE SHALL CLEARLY STATE THE CORRECTIVE ACTION TAKEN OR PLANNED TO PREVENT RECURRENCE. IN THE EVENT THAT CORRECTIVE ACTION CANNOT BE COMPLETED WITHIN THIRTY DAYS, THE AUDITED ORGANIZATION'S RESPONSE SHALL INCLUDE A SCHEDULED DATE FOR THE CORRECTIVE ACTION." CONTRARY TO THE ABOVE, THE LICENSEE'S NSDRC AUDIT PROCEDURE WAS INADEQUATE IN THAT IT ALLOWED A RESPONSE TIME OF 45 DAYS AFTER A CORRECTIVE ACTION REQUEST WAS ISSUED AND THE AUDIT COVER SHEET WAS INADEQUATE IN THAT IT ALLOWED 46 DAYS, RATHER THAN THE REQUIRED 30 DAYS. ADDITIONALLY, THERE WERE NUMEROUS EXAMPLES OF LATE RESPONSES TO CARS. (AUDIT 74 ON STAFF PERFORMANCE, QUALIFICATIONS, AND TRAINING, JULY 1981, CONTAINED FOUR CARS, NONE OF WHICH WERE RESPONDED TO IN LESS THAN 70 DAYS.) TECHNICAL SPECIFICATION 6.5.2.8.D REQUIRES AUDITS OF THE "PERFORMANCE OF ACTIVITIES REQUIRED BY THE QUALITY ASSURANCE PROGRAM TO MEET THE CRITERIA OF APPENDIX B OF 10 CFR 50..." CONTRARY TO THE ABOVE, NSDRC AUDITS WERE INSUFFICIENT AND FAILED TO EXAMINE ANY OF THE CORPORATE OFFICE ACTIVITIES RELATED TO APPENDIX B REQUIREMENTS. 10 CFR 50, APPENDIX B, CRITERION III, REQUIRES THAT DESIGN CONTROL MEASURES SHALL PROVIDE FOR DESIGN VERIFICATION, PROVIDE FOR VERIFYING OR CHECKING THE ADEQUACY OF THE DESIGN, AND THAT DESIGN CHANGES SHALL BE SUBJECT TO DESIGN CONTROL MEASURES COMMENSURATE WITH THOSE APPLIED TO THE ORIGINAL DESIGN. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTION 1.7.4.3, REQUIRES THAT MODIFICATIONS TO THE PLANT ARE CONTROLLED TO ASSURE COMPLIANCE WITH THE EXISTING DESIGN. ANSI N45.2.11-1974, TO WHICH THE LICENSEE IS COMMITTED, REQUIRES THAT DESIGN CONTROL MEASURES BE ESTABLISHED, THAT DESIGN VERIFICATIONS BE CLEARLY DOCUMENTED WITH THE IDENTIFICATION OF THE VEHICLE CLEARLY INDICATED, AND THAT DOCUMENTATION OF THE VERIFICATION RESULTS BE AUDITABLE AGAINST THE VERIFICATION METHODS IDENTIFIED BY THE RESPONSIBLE DESIGN ORGANIZATION. THE STANDARD DEFINES SUPERVISORY TECHNICAL REVIEW AS NOT SATISFYING THE DESIGN

ENFORCEMENT SUMMARY

VERIFICATION REQUIREMENT. CONTRARY TO THE ABOVE: (A) THE LICENSEE'S PROCEDURES FOR DESIGN CONTROL WERE INADEQUATE AS EVIDENCED BY: MECHANICAL ENGINEERING DESIGN PROCEDURE NO. 10, "DESIGN CONTROL", REVISION 1, REFERENCED G-P-25 WHICH ALLOWED DESIGN VERIFICATION BY THE DISCIPLINE ENGINEER'S SECTION MANAGER FOR AEPSC PERFORMED MECHANICAL DESIGN ACTIVITIES, ELECTRICAL GENERATION - ELECTRICAL ENGINEERING PROCEDURE MANUAL, "RFC PROCEDURE", DATED MAY 22, 1981, REFERENCED GP-25 WHICH ALLOWED DESIGN VERIFICATION BY THE DISCIPLINE ENGINEERS SECTION MANAGER FOR AEPSC PERFORMED ELECTRICAL DESIGN ACTIVITIES, GP-25, "DESIGN CHANGES", REVISIONS 0 AND 1, ALLOWED DESIGN VERIFICATION BY THE DISCIPLINE ENGINEER'S SECTION MANAGER FOR ALL AEPSC PERFORMED DESIGN ACTIVITIES, AND (B) DESIGN VERIFICATIONS FOR THE FOLLOWING EMERGENCY REQUESTS FOR CHANGES (RFCS) WERE INADEQUATE: RFC 12-1803 HAD NO DOCUMENTED DESIGN VERIFICATION, AND RFC 02-1885 CONTAINED A SKETCH THAT HAD BEEN INITIALED "OK", WHICH WAS THE EXTENT OF DESIGN VERIFICATION. (8318 4)

TS 6.5.2.10.C REQUIRES THAT AUDIT REPORTS BE FORWARDED TO THE SENIOR EXECUTIVE VICE PRESIDENT WITHIN 30 DAYS AFTER COMPLETION OF THE AUDIT. ANSI N.45.2.12-1977, PARAGRAPH 4.4, REQUIRES THE AUDIT REPORT TO BE ISSUED WITHIN THIRTY DAYS AFTER THE POST-AUDIT CONFERENCE. THE NSDRC PROCEDURE REQUIRED THAT "THE COMPLETED AUDIT CHECKLIST (THE LICENSEE'S DETAILED REPORT), OTHER AUDIT DOCUMENTATION VERIFYING COMPLIANCE, AND COPIES OF ALL ISSUED CARS...BE DISTRIBUTED WITHIN 30 DAYS AFTER COMPLETION OF THE AUDIT." CONTRARY TO THE ABOVE, AUDIT REPORTS EXAMINED FOR THE LAST FOUR AND ONE-HALF YEARS REVEALED THAT THE AUDIT REPORTS HAD SELDOM BEEN ISSUED OR FORWARDED TO THE SENIOR EXECUTIVE VICE PRESIDENT WITHIN 30 DAYS AFTER COMPLETION OF THE AUDIT. TECHNICAL SPECIFICATION 6.5.2.7.A REQUIRES THAT THE NSDRC REVIEW SAFETY EVALUATIONS FOR CHANGES TO EQUIPMENT OR SYSTEMS. GP-25 "DESIGN CHANGES", REVISION 0, TEMPORARY CHANGE NO. 3 TO GP-25, REVISION 0, AND PMI-5040, "DESIGN CHANGES", REVISION 4, DESCRIBED THE PROCESSING OF RFCS, INCLUDING THE PREPARATION OF A SAFETY EVALUATION WHICH WAS SUBSEQUENTLY TO BE REVIEWED BY THE NSDRC IN ACCORDANCE WITH TS 6.5.2.7.A. CONTRARY TO THE ABOVE, SAFETY EVALUATIONS WERE REVIEWED AT THE SUBCOMMITTEE LEVEL WITHOUT REVIEW AND APPROVAL BY THE FULL NSDRC.
(8318 5)

UNITS 1 AND 2 TECHNICAL SPECIFICATION 3.11.2.5 SPECIFIES LIMITS ON EXPLOSIVE GAS MIXTURES IN THE WASTE GAS HOLDUP SYSTEM. ACTION ITEM B. SPECIFIES: "WITH THE CONCENTRATION OF OXYGEN IN THE WASTE GAS HOLDUP SYSTEM OR TANK GREATER THAN 4% BY VOLUME AND GREATER THAN 4% HYDROGEN BY VOLUME WITHOUT DELAY SUSPEND ALL ADDITIONS OF WASTE GASES TO THE SYSTEM OR TANK AND REDUCE THE CONCENTRATION OF OXYGEN TO LESS THAN OR EQUAL TO 3% OR THE CONCENTRATION OF HYDROGEN TO LESS THAN OR EQUAL TO 4% WITHIN 96 HOURS IN THE SYSTEM OR TANK." CONTRARY TO THE ABOVE, ON NOVEMBER 3, 1983 AT 1400 HOURS, SAMPLES OF THE NO. 3 GAS DECAY TANK SHOWED GREATER THAN 4% BY VOLUME OF BOTH OXYGEN AND HYDROGEN, AND THE TANK WWAS NOT ISOLATED UNTIL 0615 HOURS ON NOVEMBER 4, 1983.

UNIT 1 TECHNICAL SPECIFICATION 3.6.5.6 REQUIRES TWO INDEPENDENT CONTAINMENT AIR RECIRCULATION SYSTEMS BE OPERABLE IN MODES 1, 2, 3, AND 4. TECHNICAL SPECIFICATION 3.0.4 REQUIRES THAT ENTRY INTO AN OPERATIONAL MODE OR OTHER SPECIFIED APPLICABILITY CONDITION SHALL NOT BE MADE UNLESS THE CONDITIONS OF THE LIMITING CONDITION FOR OPERATION ARE MET WITHOUT RELIANCE ON PROVISIONS CONTAINED IN THE ACTION STATEMENT UNLESS OTHERWISE EXCEPTED. CONTRARY TOTHE ABOVE, ON MARCH 2, 1982 AT 0447 HOURS, UNIT 1 ENTERED MODE 4 (FROM MODE 5) WITH ONE OF TWO CONTAINMENT AIR RECIRCULATION SYSTEMS INOPERABLE. THE SYSTEM WAS INOPERABLE BECAUSE A CONTROL SWITCH WAS IN THE STOP POSITION WHICH WOULD HAVE PREVENTED AUTOMATIC ACTUATION OF THE SYSTEM. 10 CFR 50 APPENDIX B, CRITERION XVI "CORRECTIVE ACTION" STATES IN PART: "...MEASURES SHALL ASSURE THAT THE CAUSE OF THE CONDITION IS DETERMINED AND CORRECTIVE ACTION TAKEN TO PRECLUDE REPETITION...." AEPSC GENERAL PROCEDURE 2.2; NONCONFORMANCE/NONCOMPLIANCE AND CORRECTIVE ACTION, SECTION 4.2.1 STATES: "CONDITIONS ADVERSE TO QUALITY (SUCH AS FAILURES, MALFUNCTIONS, DEFICIENCIES, DEVIATIONS, DEFECTIVE MATERIAL AND EQUIPMENT, AND NONCONFORMANCES) SHALL BE PROMPTLY IDENTIFIED, AS SOON AS PRACTICAL, BY MEASURES WHICH ARE ESTABLISHED AND DOCUMENTED." SECTION 4.2.4 STATES: "MEASURES SHALL BE ESTABLISHED TO INCLUDE FOLLOWUP ACTIONS FOR CORRECTIVE ACTION THAT CANNOT BE IMMEDIATELY COMPLETED TO ASSURE; TIMELY RESOLUTION, AND/OR COMPLETION OF THE PLANNED CORRECTIVE ACTION ACTIVITY." PMI-7030, CONDITION REPORTS, STATES IN PART: "IT IS A REQUIREMENT FOR THE SAFE AND EFFICIENT OPERATION OF THE PLANT THAT AS SOMETHING IS KNOWN OR SUSPECTED TO BE NONCOMPLIANT...OR INCONSISTENT WITH THE DESIGN OF PHYSICAL COMPONENTS, SYSTEMS AND STRUCTURES, THAT THE CONDITION BE PROMPTLY IDENTIFIED, AND RESOLVED THROUGH THE USE OF A CONDITION REPORT." CONTRARY TO THE ABOVE, IN A LETTER DATED DECEMBER 7, 1981, THE LICENSEE STATED THAT THE ROOT CAUSE FOR THE DISCREPANCY BETWEEN THE CONTAINMENT VENTILATION ISOLATION SYSTEM CONSTRUCTION AND THE WAY IT WAS FUNCTIONALLY UNDERSTOOD TO WORK ON BOTH UNITS (AS REFLECTED IN THE TECHNICAL SPECIFICATIONS AND CORRESPONDENCE TO THE NRC) WAS DUE TO THE USE OF INCORRECT FUNCTIONAL LOGIC DIAGRAMS. NEITHER A NONCONFORMANCE REPORT OR A CONDITION REPORT WAS ISSUED. THE LOGIC DIAGRAMS HAVE NOT BEEN CORRECTED AND ARE STILL IN USE. NONE OF THE LICENSEE'S CORRECTIVE ACTION SYSTEMS WERE EFFECTIVE IN ENSURING THE TIMELY RESOLUTION OF THE DISCREPANT DRAWINGS. TECHNICAL SPECIFICATION 6.5.2.7.E

ENFORCEMENT SUMMARY

REQUIRES THE NUCLEAR SAFETY AND DESIGN REVIEW COMMITTEE (NSDRC) TO REVIEW "VIOLATIONS OF CODES, REGULATIONS, ORDERS,TECHNICAL SPECIFICATIONS, LICENSE REQUIREMENTS, OR INTERNAL PROCEDURES OR INSTRUCTIONS HAVING NUCLEAR SAFETY SIGNIFICANCE." CONTRARY TO THE ABOVE, THE REVIEWS PERFORMED BY THE NSDRC DID NOT INCLUDE ALL NRC INSPECTION FINDINGS, QA AUDIT FINDINGS, AND SIGNIFICANT NON-REPORTABLE CONDITION REPORTS WHICH INVOLVED VIOLATIONS OF TECHNICAL SPECIFICATIONS OR INTERNAL PROCEDURES HAVING NUCLEAR SAFETY SIGNIFICANCE. EXAMPLES OF NRC INSPECTION FINDINGS NOT REVIEWED BY THE NSDRC INCLUDED: (A) NONCOMPLIANCE IDENTIFIED IN NRC INSPECTION REPORT 315/81-08; 316/81-12 REGARDING ADHERENCE TO PROCEDURES GOVERNING QA DOCUMENTATION OF MATERIAL; (B) NONCOMPLIANCE REGARDING VIOLATION OF THE LIMITING CONDITION FOR OPERATION ON SPRAY ADDITIVE TANKS IDENTIFIED IN NRC INSPECTION REPORT 315/81-05; 316/81-05; (C) THREE SECURITY RELATED VIOLATIONS IDENTIFIED IN NRC INSPECTION REPORT 315/81-04; 316/81-04; AND (D) VIOLATIONS DESCRIBED IN PARAGRAPHS 5 AND 7 OF NRC INSPECTION REPORT 315/81-09; 316/81-13 REGARDING THE PERFORMANCE OF SURVEILLANCES AND ADHERENCE TO PROCEDURES.

10 CFR 50, APPENDIX B, CRITERION II, REQUIRES REGULAR REVIEWS OF THE STATUS AND ADEQUACY OF THE QUALITY ASSURANCE PROGRAM. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTION 1.2.2, REQUIRES THAT THE NSDRC REGULARLY ASSESS THE SCOPE, IMPLEMENTATION, AND EFFECTIVENESS OF THE QA PROGRAM TO COMPLY WITH 10 CFR 50, APPENDIX B, CRITERIA. ANSI N18.7-1976, PARAGRAPH 4.5, STATES IN PART, "PERIODIC REVIEW OF THE AUDIT PROGRAM SHALL BE PERFORMED BY THE INDEPENDENT REVIEW BODY OR BY A MANAGEMENT REPRESENTATIVE AT LEAST SEMIANNUALLY TO ASSURE THAT AUDITS ARE BEING ACCOMPLISHED IN ACCORDANCE WITH REQUIREMENTS OF TECHNICAL SPECIFICATIONS AND OF THIS STANDARD." CONTRARY TO THE ABOVE, THE NSDRC DID NOT PERIODICALLY REVIEW THE AUDIT PROGRAM, AUDIT REPORTS, AND AUDIT RESULTS IN ORDER TO ASSESS THE SCOPE, IMPLEMENTATION, AND EFFECTIVENESS OF THE QA PROGRAM. TECHNICAL SPECIFICATION 6.5.2.7.I REQUIRES THE NSDRC TO REVIEW THE "REPORTS AND MEETING MINUTES OF THE PNSRC." CONTRARY TO THE ABOVE, THE PLANT NUCLEAR SAFETY REVIEW COMMITTEE (PNSRC) REPORTS AND MINUTES WERE NOT REVIEWED BY THE NSDRC. TECHNICAL SPECIFICATION 6.5.2.8.A REQUIRES AN AUDIT UNDER THE COGNIZANCE OF THE NSDRC OF "THE CONFORMANCE OF FACILITY OPERATION TO PROVISIONS CONTAINED WITHIN THE TECHNICAL SPECIFICATION AND APPLICABLE LICENSE CONDITIONS AT LEAST ONCE PER 12 MONTHS." CONTRARY TO THE ABOVE, NSDRC AUDITS FAILED TO EXAMINE THE TECHNICAL SPECIFICATION LIMITING CONDITIONS FOR OPERATION OR THE NSDRC ACTIVITIES. 10 CFR 50, APPENDIX B, CRITERION XVIII, REQUIRES AUDITS TO DETERMINE THE EFFECTIVENESS OF THE LICENSEE'S QA PROGRAM. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTION 1.7.4.18, REQUIRES THAT THE NSDRC CONDUCT PERIODIC AUDITS OF PLANT OPERATIONS PURSUANT TO ESTABLISHED CRITERIA. ANSI N45.2.12-1977, TO WHICH THE LICENSEE IS COMMITTED, REQUIRES IN PARAGRAPH 4.4.4 THAT EACH AUDIT REPORT INCLUDE A SUMMARY OF RESULTS WHICH INCLUDES "AN EVALUATION STATEMENT REGARDING THE EFFECTIVENESS OF THE QUALITY ASSURANCE PROGRAM ELEMENTS WHICH WERE AUDITED." CONTRARY TO THE ABOVE, MOST OF THE NSDRC AUDIT REPORTS FOR AUDITS CONDUCTED OVER THE LAST SEVERAL YEARS CONTAINED NO EVALUATION STATEMENTS. 10 CFR 50, APPENDIX B, CRITERION V, REQUIRES THAT ACTIVITIES AFFECTING QUALITY BE ACCOMPLISHED IN ACCORDANCE WITH DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES. CRITERION XVI REQUIRES THAT MEASURES BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY ARE PROMPTLY IDENTIFIED AND CORRECTED. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTIONS 1.7.4.5 AND 1.7.4.16, REQUIRES THAT ACTIVITIES AFFECTING QUALITY BE CONDUCTED AS DIRECTED BY APPROPRIATE INSTRUCTIONS, PROCEDURES, OR DRAWINGS AND THAT CONDITIONS ADVERSE TO QUALITY BE REPORTED TO MANAGEMENT. ANSI N45.2.12-1977, PARAGRAPH 4.5.1, REQUIRES THAT "THE RESPONSE SHALL CLEARLY STATE THE CORRECTIVE ACTION TAKEN OR PLANNED TO PREVENT RECURRENCE. IN THE EVENT THAT CORRECTIVE ACTION CANNOT BE COMPLETED WITHIN THIRTY DAYS, THE AUDITED ORGANIZATION'S RESPONSE SHALL INCLUDE A SCHEDULED DATE FOR THE CORRECTIVE ACTION." CONTRARY TO THE ABOVE, THE LICENSEE'S NSDRC AUDIT PROCEDURE WAS INADEQUATE IN THAT IT ALLOWED A RESPONSE TIME OF 45 DAYS AFTER A CORRECTIVE ACTION REQUEST WAS ISSUED AND THE AUDIT COVER SHEET WAS INADEQUATE IN THAT IT ALLOWED 46 DAYS, RATHER THAN THE REQUIRED 30 DAYS. ADDITIONALLY, THERE WERE NUMEROUS EXAMPLES OF LATE RESPONSES TO CARS. (AUDIT 74 ON STAFF PERFORMANCE, QUALIFICATIONS, AND TRAINING, JULY 1981, CONTAINED FOUR CARS, NONE OF WHICH WERE RESPONDED TO IN LESS THAN 70 DAYS.) TECHNICAL SPECIFICATION 6.5.2.8.D REQUIRES AUDITS OF THE "PERFORMANCE OF ACTIVITIES REQUIRED BY THE QUALITY ASSURANCE PROGRAM TO MEET THE CRITERIA OF APPENDIX B OF 10 CFR 50..." CONTRARY TO THE ABOVE, NSDRC AUDITS WERE INSUFFICIENT AND FAILED TO EXAMINE ANY OF THE CORPORATE OFFICE ACTIVITIES RELATED TO APPENDIX B REQUIREMENTS. 10 CFR 50, APPENDIX B, CRITERION III, REQUIRES THAT DESIGN CONTROL MEASURES SHALL PROVIDE FOR DESIGN VERIFICATION, PROVIDE FOR VERIFYING OR CHECKING THE ADEQUACY OF THE DESIGN, AND THAT DESIGN CHANGES SHALL BE SUBJECT TO DESIGN CONTROL MEASURES COMMENSURATE WITH THOSE APPLIED TO THE ORIGINAL DESIGN. THE QA PROGRAM FOR THE DONALD C. COOK NUCLEAR PLANT, SECTION 1.7.4.3, REQUIRES THAT MODIFICATIONS TO THE PLANT ARE CONTROLLED TO ASSURE COMPLIANCE WITH THE EXISTING DESIGN. ANSI N45.2.11-1974, TO WHICH THE LICENSEE IS COMMITTED, REQUIRES THAT DESIGN CONTROL MEASURES BE ESTABLISHED, THAT DESIGN VERIFICATIONS BE CLEARLY DOCUMENTED WITH THE IDENTIFICATION OF THE VEHICLE CLEARLY INDICATED, AND THAT DOCUMENTATION OF THE VERIFICATION RESULTS BE AUDITABLE AGAINST THE VERIFICATION METHODS IDENTIFIED BY THE RESPONSIBLE DESIGN ORGANIZATION. THE STANDARD DEFINES SUPERVISORY TECHNICAL REVIEW AS NOT SATISFYING THE DESIGN VERIFICATION REQUIREMENT. CONTRARY TO THE ABOVE: (A) THE LICENSEE'S PROCEDURES FOR DESIGN CONTROL WERE INADEQUATE AS EVIDENCED

ENFORCEMENT SUMMARY

BY: MECHANICAL ENGINEERING DESIGN PROCEDURE NO. 10, "DESIGN CONTROL", REVISION 1, REFERENCED G-P-25 WHICH ALLOWED DESIGN VERIFICATION BY THE DISCIPLINE ENGINEER'S SECTION MANAGER FOR AEPSC PERFORMED MECHANICAL DESIGN ACTIVITIES, ELECTRICAL GENERATION - ELECTRICAL ENGINEERING PROCEDURE MANUAL, "RFC PROCEDURE", DATED MAY 22, 1981, REFERENCED GP-25 WHICH ALLOWED DESIGN VERIFICATION BY THE DISCIPLINE ENGINEERS SECTION MANAGER FOR AEPSC PERFORMED ELECTRICAL DESIGN ACTIVITIES, GP-25, "DESIGN CHANGES", REVISIONS 0 AND 1, ALLOWED DESIGN VERIFICATION BY THE DISCIPLINE ENGINEER'S SECTION MANAGER FOR ALL AEPSC PERFORMED DESIGN ACTIVITIES, AND (B) DESIGN VERIFICATIONS FOR THE FOLLOWING EMERGENCY REQUESTS FOR CHANGES (RFCS) WERE INADEQUATE: RFC 12-1803 HAD NO DOCUMENTED DESIGN VERIFICATION, AND RFC 02-1885 CONTAINED A SKETCH THAT HAD BEEN INITIALED "OK", WHICH WAS THE EXTENT OF DESIGN VERIFICATION. (8319 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

OPERATING ROUTINELY.

LAST IE SITE INSPECTION DATE: JANUARY 1-31, 1984

INSPECTION REPORT NO: 84-03

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-01	01/23/84	02/22/84	DURING CALIBRATION OF REACTOR COOLIANT LOOP FLOW TRANSMITTERS FLOW INDICATION ON CHANNEL 1 OBSERVED TO DROP, FOLLOWED BY TRIP.

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1. Docket: 50-316 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: W. T. GILLETT (616) 465-5901

4. Licensed Thermal Power (MWt): 3411

5. Nameplate Rating (Gross MWe): 1333 X 0.85 = 1133

6. Design Electrical Rating (Net MWe): 1100

7. Maximum Dependable Capacity (Gross MWe): 1100

8. Maximum Dependable Capacity (Net MWe): 1060

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	54,024.0
13. Hours Reactor Critical	672.2	1,416.2	39,201.4
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	663.9	1,407.9	38,208.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,235,476	4,671,207	123,124,175
18. Gross Elec Ener (MWH)	738,120	1,548,680	39,775,110
19. Net Elec Ener (MWH)	713,057	1,495,371	38,348,724
20. Unit Service Factor	95.4	97.8	74.0
21. Unit Avail Factor	95.4	97.8	74.0
22. Unit Cap Factor (MDC Net)	96.7	98.0	70.7
23. Unit Cap Factor (DER Net)	93.1	94.4	69.5
24. Unit Forced Outage Rate	4.6	2.2	13.5
25. Forced Outage Hours	32.1	32.1	5,883.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

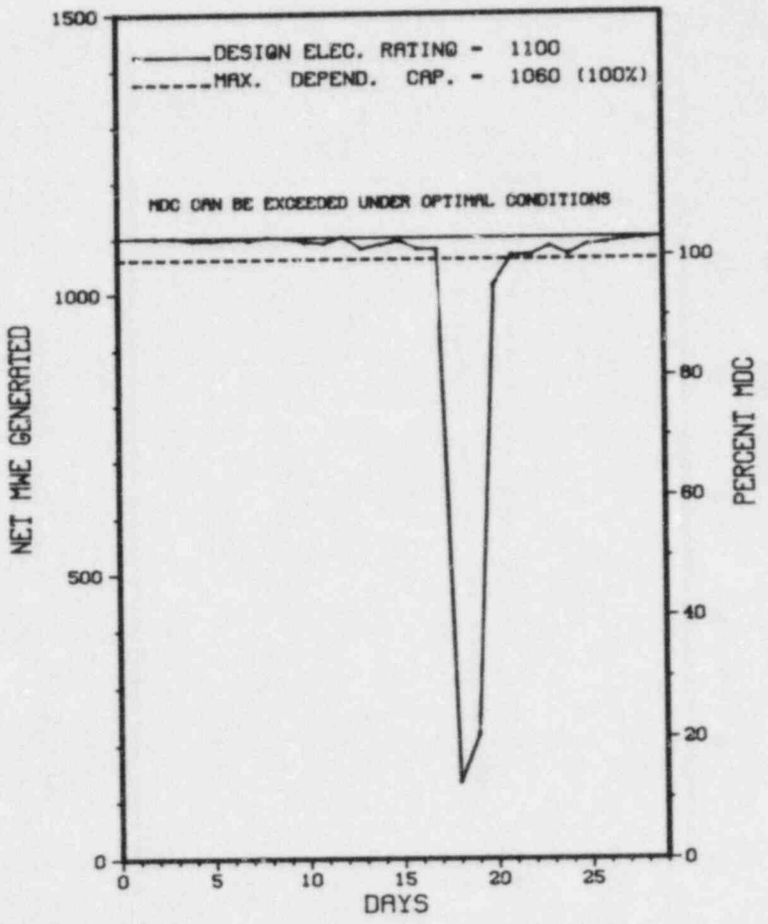
REFUELING OUTAGE-3/9/84 - 90 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* COOK 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

COOK 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

COOK 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
146	02/18/84	F	32.1	G	3	84-002	HJ	ZZZZZZ	THE UNIT TRIPPED DUE TO AN APPARENT HIGH WATER LEVEL IN ONE OF THE MOISTURE SEPARATOR REHEATER, MSR, SHELLS. THE TRIP OCCURRED WHILE RECOVERING FROM AN AUTOMATIC ISOLATION OF THE MSR COILS DUE TO A CYCLE UPSET FOLLOWING TURBINE CONTROL VALVE TESTING. WHILE ATTEMPTING TO REPRESSURIZE THE MSR COILS AN OPERATING ERROR WAS COMMITTED WHICH RAPIDLY ADMITTED FULL PRESSURE MAIN STEAM TO THE SOUTH SET OF MSR COILS. IT IS BELIEVED THAT THIS CAUSED A VIBRATION THAT SHOOK THE MSR SHELL HIGH WATER LEVEL TRIP SWITCH WHICH RESULTED IN THE UNIT TRIP FROM A FALSE INDICATION OF HIGH LEVEL.

SUMMARY

COOK 2 OPERATED WITH 1 OUTAGE DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* COOK 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MICHIGAN

COUNTY.....BERRIEN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI S OF
BENTON HARBOR, MI

TYPE OF REACTOR.... .....PWR

DATE INITIAL CRITICALITY...MARCH 10, 1978

DATE ELEC ENER 1ST GENER...MARCH 22, 1978

DATE COMMERCIAL OPERATE....JULY 1, 1978

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....INDIANA & MICHIGAN ELECTRIC

CORPORATE ADDRESS.....1 RIVERSIDE PLAZA
COLUMBUS, OHIO 43216

CONTRACTOR
ARCHITECT/ENGINEER.....AMERICAN ELEC. POWER SERVICE CORP.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....J. A. JONES CONSTRUCTION

TURBINE SUPPLIER.....BROWN BOVERI

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....E. SWANSON

LICENSING PROJ MANAGER.....D. WIGGINTON
DOCKET NUMBER.....50-316

LICENSE & DATE ISSUANCE....DPR-74, DECEMBER 23, 1977

PUBLIC DOCUMENT ROOM.....MAUDE PRESTON PALENSKE MEMORIAL LIBRARY
500 MARKET STREET
ST. JOSEPH, MICHIGAN 49085

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON DECEMBER 11, THROUGH JANUARY 20, (83-22): ROUTINE UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTOR OF LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS; OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; LICENSEE EVENT REPORTS; AND OPERATING ACTIVITIES. THE INSPECTION INVOLVED A TOTAL OF 180 INSPECTOR-HOURS BY TWO NRC INSPECTORS INCLUDING 34 INSPECTOR-HOURS OFF-SHIFTS. OF THE SIX AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE REMAINING AREA (FAILURE TO SUBMIT A SPECIAL REPORT).

INSPECTION ON JANUARY 23-27, (84-01): ROUTINE, ANNOUNCED INSPECTION OF THE DONALD C. COOK NUCLEAR PLANT EMERGENCY PREPAREDNESS EXERCISE INVOLVING OBSERVATIONS BY SIX NRC REPRESENTATIVES OF KEY FUNCTIONS AND LOCATIONS DURING THE EXERCISE. THE INSPECTION INVOLVED 116 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS AND THREE CONSULTANTS. NO ITEMS OF NONCOMPLIANCE, DEFICIENCIES, OR DEVIATIONS WERE IDENTIFIED.

MANAGEMENT MEETING ON JANUARY 31, (84-03): THE PROGRESS AND STATUS OF THE LICENSEE'S REGULATORY PERFORMANCE IMPROVEMENT PROGRAM. THE MEETING INVOLVED A TOTAL OF 16 MAN-HOURS BY 8 NRC PERSONNEL.

INSPECTION ON FEBRUARY 7, (84-04): REVIEW OF DOCUMENTATION AND WALK-DOWNS PERTAINING TO EQUIPMENT REFERRED TO IN INFORMATION LETTER 81-14 DATED FEBRUARY 10, 1981. THE INSPECTION UTILIZED A TOTAL OF 16 HRS ONSITE BY TWO NRC INSPECTORS. BASED UPON THIS REVIEW AND THE ASSOCIATED WALK-DOWNS OF THE AREAS AFFECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

TS 6.5.2.10.C REQUIRES THAT AUDIT REPORTS BE FORWARDED TO THE SENIOR EXECUTIVE VICE PRESIDENT WITHIN 30 DAYS AFTER COMPLETION OF THE AUDIT. ANSI N.45.2.12-1977, PARAGRAPH 4.4, REQUIRES THE AUDIT REPORT TO BE ISSUED WITHIN THIRTY DAYS AFTER THE POST-AUDIT CONFERENCE. THE NSDRC PROCEDURE REQUIRED THAT "THE COMPLETED AUDIT CHECKLIST (THE LICENSEE'S DETAILED REPORT), OTHER AUDIT DOCUMENTATION VERIFYING COMPLIANCE, AND COPIES OF ALL ISSUED CARS...BE DISTRIBUTED WITHIN 30 DAYS AFTER COMPLETION OF THE AUDIT." CONTRARY TO THE ABOVE, AUDIT REPORTS EXAMINED FOR THE LAST FOUR AND ONE-HALF YEARS REVEALED THAT THE AUDIT REPORTS HAD SELDOM BEEN ISSUED OR FORWARDED TO THE SENIOR EXECUTIVE VICE PRESIDENT WITHIN 30 DAYS AFTER COMPLETION OF THE AUDIT. TECHNICAL SPECIFICATION 6.5.2.7.A REQUIRES THAT THE NSDRC REVIEW SAFETY EVALUATIONS FOR CHANGES TO EQUIPMENT OR SYSTEMS. GP-25 "DESIGN CHANGES", REVISION 0, TEMPORARY CHANGE NO. 3 TO GP-25, REVISION 0, AND PMI-5040, "DESIGN CHANGES", REVISION 4, DESCRIBED THE PROCESSING OF RFCS, INCLUDING THE PREPARATION OF A SAFETY EVALUATION WHICH WAS SUBSEQUENTLY TO BE REVIEWED BY THE NSDRC IN ACCORDANCE WITH TS 6.5.2.7.A. CONTRARY TO THE ABOVE, SAFETY EVALUATIONS WERE REVIEWED AT THE SUBCOMMITTEE LEVEL WITHOUT REVIEW AND APPROVAL BY THE FULL NSDRC.
(8319 5)

UNITS 1 AND 2 TECHNICAL SPECIFICATION 3.11.2.5 SPECIFIES LIMITS ON EXPLOSIVE GAS MIXTURES IN THE WASTE GAS HOLDUP SYSTEM. ACTION ITEM B. SPECIFIES: "WITH THE CONCENTRATION OF OXYGEN IN THE WASTE GAS HOLDUP SYSTEM OR TANK GREATER THAN 4% BY VOLUME AND GREATER THAN 4% HYDROGEN BY VOLUME WITHOUT DELAY SUSPEND ALL ADDITIONS OF WASTE GASES TO THE SYSTEM OR TANK AND REDUCE THE CONCENTRATION OF OXYGEN TO LESS THAN OR EQUAL TO 3% OR THE CONCENTRATION OF HYDROGEN TO LESS THAN OR EQUAL TO 4% WITHIN 96 HOURS IN THE SYSTEM OR TANK." CONTRARY TO THE ABOVE, ON NOVEMBER 3, 1983 AT 1400 HOURS, SAMPLES OF THE NO. 3 GAS DECAY TANK SHOWED GREATER THAN 4% BY VOLUME OF BOTH OXYGEN AND HYDROGEN, AND THE TANK WWAS NOT ISOLATED UNTIL 0615 HOURS ON NOVEMBER 4, 1983.

10 CFR 50 APPENDIX B, CRITERION XVI "CORRECTIVE ACTION" STATES IN PART: "...MEASURES SHALL ASSURE THAT THE CAUSE OF THE CONDITION IS DETERMINED AND CORRECTIVE ACTION TAKEN TO PRECLUDE REPETITION...." AEPSC GENERAL PROCEDURE 2.2; NONCONFORMANCE/NONCOMPLIANCE AND CORRECTIVE ACTION, SECTION 4.2.1 STATES: "CONDITIONS ADVERSE TO QUALITY (SUCH AS FAILURES, MALFUNCTIONS, DEFICIENCIES, DEVIATIONS, DEFECTIVE MATERIAL AND EQUIPMENT, AND NONCONFORMANCES) SHALL BE PROMPTLY IDENTIFIED, AS SOON AS PRACTICAL, BY MEASURES WHICH ARE ESTABLISHED AND DOCUMENTED." SECTION 4.2.4 STATES: "MEASURES SHALL BE ESTABLISHED TO INCLUDE FOLLOWUP ACTIONS FOR CORRECTIVE ACTION THAT CANNOT BE IMMEDIATELY COMPLETED TO ASSURE; TIMELY RESOLUTION, AND/OR COMPLETION OF THE PLANNED CORRECTIVE ACTION ACTIVITY." PMI-7030, CONDITION REPORTS, STATES IN PART: "IT IS A REQUIREMENT FOR THE SAFE AND EFFICIENT OPERATION OF THE PLANT THAT AS SOMETHING IS KNOWN OR SUSPECTED TO BE NONCOMPLIANT...OR INCONSISTENT WITH THE DESIGN OF PHYSICAL COMPONENTS, SYSTEMS AND STRUCTURES, THAT THE CONDITION BE PROMPTLY IDENTIFIED, AND RESOLVED THROUGH THE USE OF A CONDITION REPORT." CONTRARY TO THE ABOVE, IN A LETTER DATED DECEMBER 7, 1981, THE LICENSEE STATED THAT THE ROOT CAUSE FOR THE DISCREPANCY BETWEEN THE CONTAINMENT VENTILATION ISOLATION SYSTEM CONSTRUCTION AND THE WAY IT WAS FUNCTIONALLY UNDERSTOOD TO WORK ON BOTH UNITS (AS REFLECTED IN THE TECHNICAL SPECIFICATIONS AND CORRESPONDENCE TO THE NRC) WAS DUE TO THE USE OF INCORRECT FUNCTIONAL LOGIC DIAGRAMS. NEITHER A NONCONFORMANCE REPORT OR A CONDITION REPORT WAS ISSUED. THE LOGIC DIAGRAMS HAVE NOT BEEN CORRECTED AND ARE STILL IN USE. NONE OF THE LICENSEE'S CORRECTIVE ACTION SYSTEMS WERE EFFECTIVE IN ENSURING THE TIMELY RESOLUTION OF THE DISCREPANT DRAWINGS.
(8320 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

OTHER ITEMS

NONE

PLANT STATUS:

PLANT OPERATING ROUTINELY. A PLANNED THREE MONTH REFUELING OUTAGE IS SCHEDULED TO START IN MARCH, 1984.

LAST IE SITE INSPECTION DATE: JANUARY 1-31, 1984

INSPECTION REPORT NO: 84-03

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-01	01/17/84	02/16/84	BIT DILUTED BY PRIMARY WATER AFTER EQUIPMENT CLEARANCE PERMIT RELEASED AND SYSTEM REALIGNED.

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1. Docket: 50-298 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: P. L. BALLINGER (402) 825-3811

4. Licensed Thermal Power (MWt): 2381

5. Nameplate Rating (Gross MWe): 983 X 0.85 = 836

6. Design Electrical Rating (Net MWe): 778

7. Maximum Dependable Capacity (Gross MWe): 787

8. Maximum Dependable Capacity (Net MWe): 764

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	84,745.0
13. Hours Reactor Critical	696.0	1,414.0	68,417.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,396.6	67,314.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,459,992	2,838,504	133,351,662
18. Gross Elec Ener (MWH)	491,010	963,123	42,369,478
19. Net Elec Ener (MWH)	469,584	921,681	40,838,340
20. Unit Service Factor	100.0	97.0	79.4
21. Unit Avail Factor	100.0	97.0	79.4
22. Unit Cap Factor (MDC Net)	88.3	83.8	63.1
23. Unit Cap Factor (DER Net)	86.7	82.3	61.9
24. Unit Forced Outage Rate	.0	3.0	3.8
25. Forced Outage Hours	.0	43.4	2,000.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

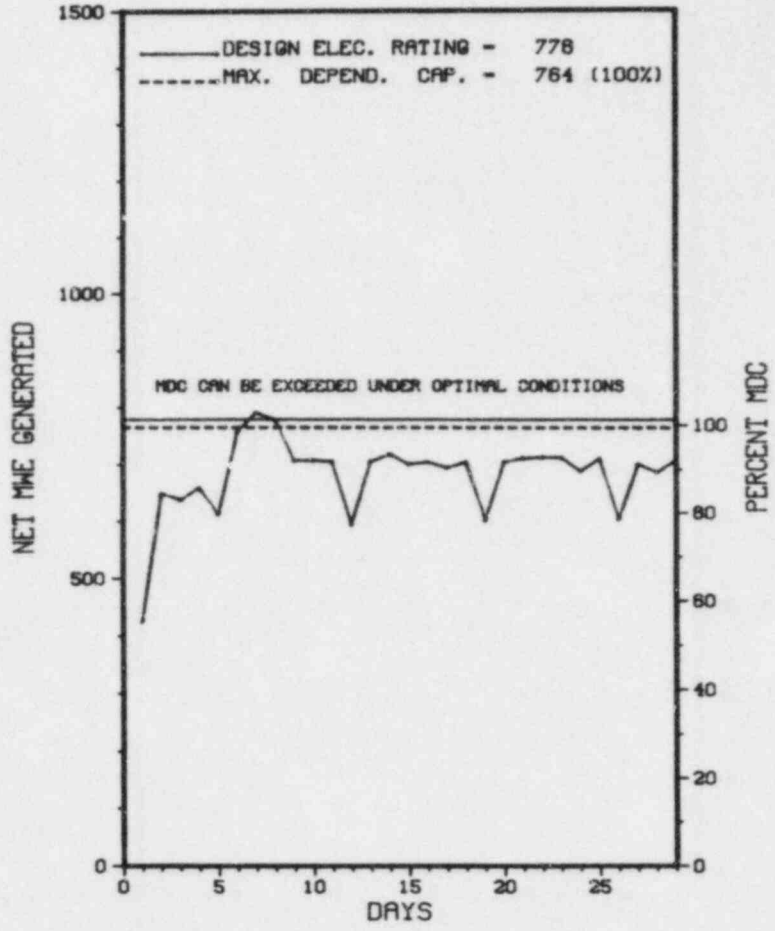
MAINTENANCE, APRIL 7, 1984, 10 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* COOPER STATION *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

COOPER STATION



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

COOPER STATION

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

COOPER STATION OPERATED WITH NO REPORTED REDUCTIONS OR OUTAGES DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training	5-Reduced Load	Licensee Event Report
	& License Examination	9-Other	(LER) File (NUREG-0161)

****
* COOPER STATION *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NEBRASKA

COUNTY.....NEMAHA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...23 MI S OF
NEBRASKA CITY, NEB

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...FEBRUARY 21, 1974

DATE ELEC ENER 1ST GENER...MAY 10, 1974

DATE COMMERCIAL OPERATE....JULY 1, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSOURI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NEBRASKA PUBLIC POWER DISTRICT

CORPORATE ADDRESS.....P.O. BOX 499
COLUMBUS, NEBRASKA 68601

CONTRACTOR
ARCHITECT/ENGINEER.....BURNS & ROE

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BURNS & ROE

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....D. DUBOIS

LICENSING PROJ MANAGER.....B. SIEGEL
DOCKET NUMBER.....50-298

LICENSE & DATE ISSUANCE....DPR-46, JANUARY 18, 1974

PUBLIC DOCUMENT ROOM.....AUBURN PUBLIC LIBRARY
1118 15TH STREET
AUBURN, NEBRASKA 68305

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION CONDUCTED

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

OTHER ITEMS

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

ROUTINE POWER OPERATION

LAST IE SITE INSPECTION DATE: DECEMBER 30, 1983

INSPECTION REPORT NO: 50-298/8334

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
L84-001	1/20/84	2/24/84	FAILURE OF REACTOR VESSSEL WATER LEVEL INDICATING SWITCH NBI-LIS-101B TO RESET.
L84-002	1/29/84	2/24/84	REACTOR TRIP
L84-003	1/30/84	2/24/84	REACTOR TRIP

1. Docket: 50-302 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: D. BOGART (904) 795-6486
4. Licensed Thermal Power (MWt): 2544
5. Nameplate Rating (Gross MWe): 989 X 0.9 = 890
6. Design Electrical Rating (Net MWe): 825
7. Maximum Dependable Capacity (Gross MWe): 850
8. Maximum Dependable Capacity (Net MWe): 821
9. If Changes Occur Above Since Last Report, Give Reasons:
MDC CHANGE BASED ON OPERATING EXPERIENCE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

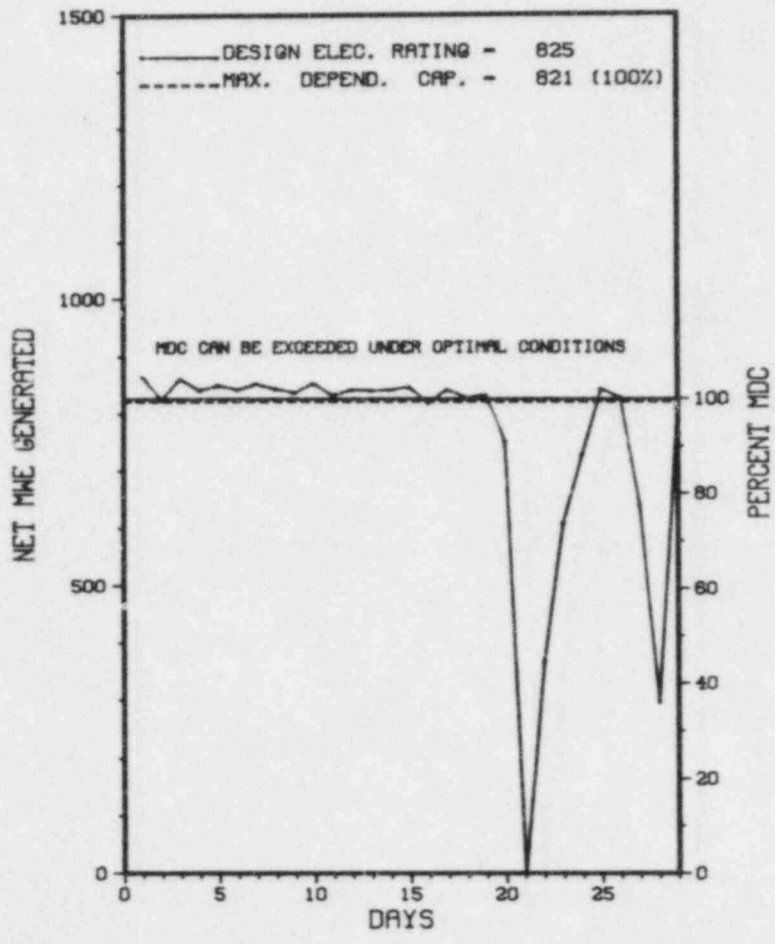
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	61,080.0
13. Hours Reactor Critical	667.2	1,385.9	38,955.9
14. Rx Reserve Shtdwn Hrs	.0	.0	1,275.5
15. Hrs Generator On-Line	653.8	1,365.6	37,984.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,574,903	3,274,207	85,238,342
18. Gross Elec Ener (MWH)	549,204	1,142,975	29,069,711
19. Net Elec Ener (MWH)	523,896	1,090,241	27,607,324
20. Unit Service Factor	93.9	94.8	62.2
21. Unit Avail Factor	93.9	94.8	62.2
22. Unit Cap Factor (MDC Net)	91.7	92.2	55.1
23. Unit Cap Factor (DER Net)	91.2	91.8	54.8
24. Unit Forced Outage Rate	6.1	3.0	23.5
25. Forced Outage Hours	42.2	42.2	11,657.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* CRYSTAL RIVER 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

CRYSTAL RIVER 3



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

CRYSTAL RIVER 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-06	02/01/84	F	0.0	A	5		CC	HTEXCH	POWER REDUCTION FORCED BY NECESSITY TO ISOLATE LEAKING MSR HIGH PRESSURE STEAM BUNDLE.
84-07	02/08/84	S	0.0	F	5		CC	INSTRU	REDUCED POWER TO REPAIR THE 'B' OTSG LEVEL TRANSMITTER.
84-08	02/20/84	F	32.0	B	1		CB	MOTORX	SHUTDOWN TO INVESTIGATE OIL LEVEL ALARM ON THE 'C' RCP PUMP.
84-09	02/22/84	S	0.0	B	5		HC	HTEXCH	REDUCED POWER TO CLEAN CONDENSER TUBES.
84-10	02/27/84	S	0.0	B	5		HC	HTEXCH	REDUCED POWER TO APPLY EPOXY COATING TO 'A' CONDENSER TUBE SHEET TO PREVENT LEAKAGE.
84-11	02/28/84	F	10.2	H	3		EA	CKTBRK	LIGHTNING STRIKE CAUSED MOMENTARY LOSS OF OFF-SITE POWER AND SUBSEQUENT REACTOR SCRAM.

SUMMARY

CRYSTAL RIVER 3 OPERATED ROUTINELY DURING THE REPORT PERIOD.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training	5-Reduced Load	Licensee Event Report
	& License Examination	9-Other	(LER) File (NUREG-0161)

****
* CRYSTAL RIVER 3 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....CITRUS

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...7 MI NW OF
CRYSTAL RIVER, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JANUARY 14, 1977

DATE ELEC ENER 1ST GENER...JANUARY 30, 1977

DATE COMMERCIAL OPERATE....MARCH 13, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....GULF OF MEXICO

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER CORPORATION

CORPORATE ADDRESS.....3201 34TH STREET, SOUTH
ST PETERSBURG, FLORIDA 33733

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....J. A. JONES CONSTRUCTION

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....T. STETKA

LICENSING PROJ MANAGER.....R. HERNAN
DOCKET NUMBER.....50-302

LICENSE & DATE ISSUANCE....DPR-72, JANUARY 28, 1977

PUBLIC DOCUMENT ROOM.....CRYSTAL RIVER PUBLIC LIBRARY
668 N.W. FIRST
CRYSTAL RIVER, FLORIDA 32639

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JANUARY 30 - FEBRUARY 3 (84-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF QUALITY ASSURANCE, TRAINING, POSTING AND LABELING, EXTERNAL EXPOSURE CONTROL, RADIATION PROTECTION PROCEDURES, PLANT TOURS, AND OUTAGE PREPLANNING. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 3-30 (84-02): THIS ROUTINE INSPECTION INVOLVED 128 INSPECTOR-HOURS ON SITE BY ONE RESIDENT INSPECTOR IN THE AREAS OF PLANT OPERATIONS, SECURITY, RADIOLOGICAL CONTROLS, PLANT REVIEW COMMITTEE ACTIVITIES, LICENSEE EVENT REPORTS AND NONCONFORMING OPERATIONS REPORTS, AND LICENSEE ACTION ON PREVIOUS INSPECTION ITEMS. THIS INSPECTION ALSO DOCUMENTS UTILITY COMPLIANCE WITH THE LICENSED OPERATOR STAFFING RULE 10 CFR 50.54 (TMI ITEM I.A.1.3). NUMEROUS FACILITY TOURS WERE CONDUCTED AND FACILITY OPERATIONS OBSERVED. SOME OF THESE TOURS AND OBSERVATIONS WERE CONDUCTED ON BACK SHIFTS. TWO VIOLATIONS WERE IDENTIFIED (FAILURE TO FOLLOW RADIATION PROTECTION PROCEDURES; AND FAILURE TO MAINTAIN CALIBRATION OF INSTRUMENTATION.

ENFORCEMENT SUMMARY

CONTRARY TO TECHNICAL SPECIFICATION 6.11 RADIATION PROTECTION PROCEDURES WERE NOT ADHERED TO. (8402 4)

CONTRARY TO 10CFR50, APPENDIX B, CRITERION V AS IMPLEMENTED BY THE LICENSEE'S QUALITY ASSURANCE PROGRAM SECTION 1.7.1.5,

ENFORCEMENT SUMMARY

UNCALIBRATED INSTRUMENTATION WAS UTILIZED IN THE PERFORMANCE OF QUALITY ACTIVITIES.
(8402 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: JANUARY 3-30, 1984 +

INSPECTION REPORT NO: 50-302/84-02 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-346 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: BILAL SARSOUR (419) 259-5000 X384
4. Licensed Thermal Power (MWt): 2772
5. Nameplate Rating (Gross MWe): 1069 X 0.9 = 962
6. Design Electrical Rating (Net MWe): 906
7. Maximum Dependable Capacity (Gross MWe): 918
8. Maximum Dependable Capacity (Net MWe): 874
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	48,961.0
13. Hours Reactor Critical	696.0	1,057.1	28,559.5
14. Rx Reserve Shtdwn Hrs	.0	0	3,879.3
15. Hrs Generator On-Line	696.0	1,035.4	27,188.2
16. Unit Reserve Shtdwn Hrs	.0	0	1,732.7
17. Gross Therm Ener (MWH)	1,890,726	2,696,380	63,740,194
18. Gross Elec Ener (MWH)	626,758	891,950	21,183,243
19. Net Elec Ener (MWH)	595,680	834,609	19,833,308
20. Unit Service Factor	100.0	72.0	55.5
21. Unit Avail Factor	100.0	72.0	59.1
22. Unit Cap Factor (MDC Net)	97.9	66.3	46.3
23. Unit Cap Factor (DER Net)	94.5	64.0	44.7
24. Unit Forced Outage Rate	.0	28.0	19.0
25. Forced Outage Hours	.0	403.6	6,987.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

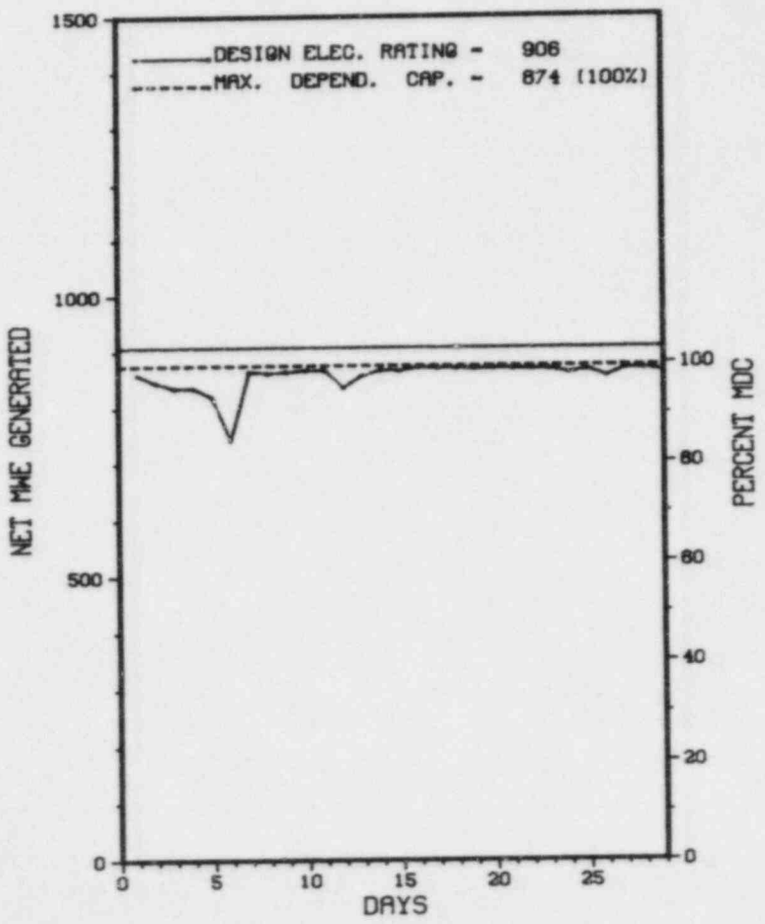
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* DAVIS-BESSE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DAVIS-BESSE 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

DAVIS-BESSE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

DAVIS-BESSE 1 OPERATED WITH NO OUTAGES OR REDUCTIONS DURING THE REPORT PERIOD.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
		9-Other	(LER) File (NUREG-0161)

****
* DAVIS-BESSE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....OHIO

COUNTY.....OTTAWA

DIST AND DIRECTION FROM NEAREST POPULATION CTR...21 MI E OF TOLEDO, OH

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 12, 1977

DATE ELEC ENER 1ST GENER...AUGUST 28, 1977

DATE COMMERCIAL OPERATE....JULY 31, 1978

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....LAKE ERIE

ELECTRIC RELIABILITY COUNCIL.....EAST CENTRAL AREA RELIABILITY COORDINATION AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TOLEDO EDISON

CORPORATE ADDRESS.....300 MADISON AVENUE
TOLEDO, OHIO 43652

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....W. ROGERS

LICENSING PROJ MANAGER.....A. DEAGAZIO
DOCKET NUMBER.....50-346

LICENSE & DATE ISSUANCE....NPF-3, APRIL 22, 1977

PUBLIC DOCUMENT ROOM.....UNIVERSITY OF TOLEDO LIBRARY
GOVERNMENT DOCUMENTS COLLECTION
2801 WEST BANCROFT AVENUE
TOLEDO, OHIO 43606

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON NOVEMBER 19, THROUGH JANUARY 12, (83-24): ROUTINE UNANNOUNCED INSPECTION OF OPERATIONAL SAFETY, MAINTENANCE, SURVEILLANCE, PLANT TRIPS, MANAGEMENT MEETING AND INDEPENDENT INSPECTION. THE INSPECTION INVOLVED 120 INSPECTOR-HOURS ONSITE BY ONE INSPECTOR INCLUDING 20 INSPECTOR-HOURS ONSITE DURING OFFSHIFTS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN FOUR AREAS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE AREA OF PLANT TRIPS (FAILURE TO FOLLOW RESPIRATOR QUALIFICATION PROCEDURE) AND ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE AREA OF INDEPENDENT INSPECTION (FAILURE TO CONTROL DESIGN DOCUMENTS).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

OTHER ITEMS

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: NOVEMBER 19, 1983 THROUGH JANUARY 2, 1984

INSPECTION REPORT NO: 83-24

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-74/ 03L-0	12/24/83	01/24/84	BWST HI FAIL ALARM FOR SFAS CHANNEL 1 RECEIVED BY OPERATORS.
84-01/	01/03/84		REACTOR TRIP CAUSED BY AUTO. INSERTION OF AXIAL POWER SHAPING RODS.
84-02/	01/19/84	02/17/84	FUEL TRANSFER TUBE DRAIN VALVES LEFT OPEN AND UNCAPPED.

1. Docket: 50-237 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: BEN SCHROEDER (815) 942-2920

4. Licensed Thermal Power (MWt): 2527

5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828

6. Design Electrical Rating (Net MWe): 794

7. Maximum Dependable Capacity (Gross MWe): 812

8. Maximum Dependable Capacity (Net MWe): 772

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	120,960.0
13. Hours Reactor Critical	696.0	1,440.0	93,665.5
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,418.1	89,314.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,570,780	3,303,982	180,041,578
18. Gross Elec Ener (MWH)	513,257	1,080,893	57,584,060
19. Net Elec Ener (MWH)	489,348	1,031,446	54,428,890
20. Unit Service Factor	100.0	98.5	73.8
21. Unit Avail Factor	100.0	98.5	73.8
22. Unit Cap Factor (MDC Net)	91.1	92.8	58.3
23. Unit Cap Factor (DER Net)	88.5	90.2	56.7
24. Unit Forced Outage Rate	.0	1.5	11.8
25. Forced Outage Hours	.0	21.9	4,442.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

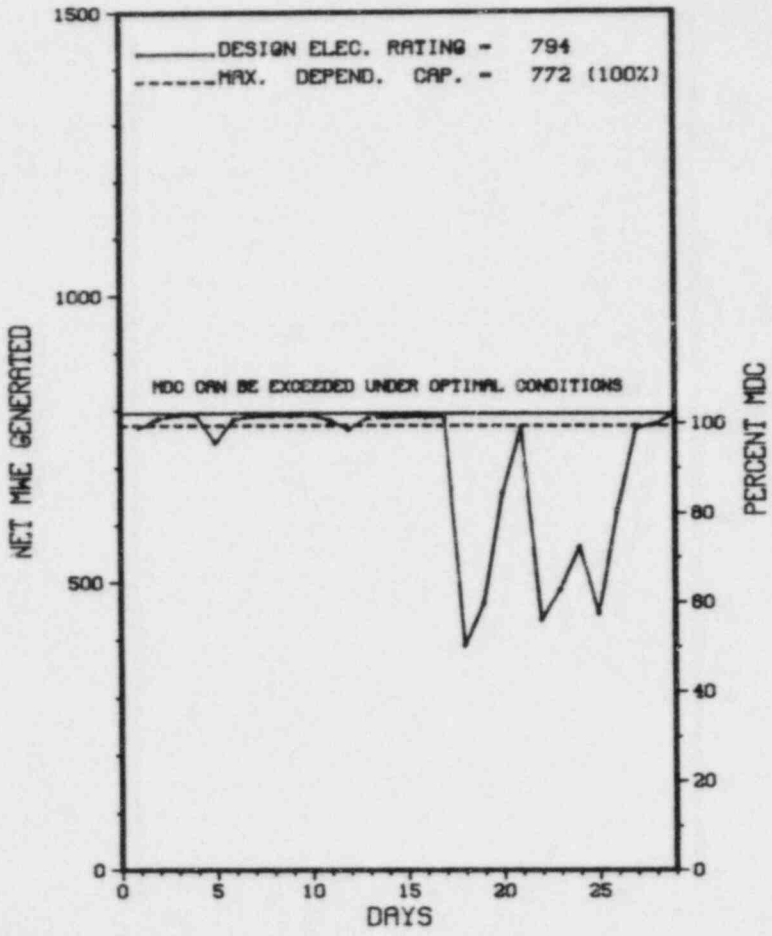
SNUBBER INSPECTION: MAY, 1984.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* DRESDEN 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DRESDEN 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

DRESDEN 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

DRESDEN 2 OPERATED WITH NO REPORTED REDUCTIONS OR OUTAGES DURING FEBRUARY

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* DRESDEN 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....GRUNDY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9 MI E OF
MORRIS, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JANUARY 7, 1970

DATE ELEC ENER 1ST GENER...APRIL 13, 1970

DATE COMMERCIAL OPERATE....JUNE 9, 1970

CONDENSER COOLING METHOD...COOLING LAKE

CONDENSER COOLING WATER....KANKAKEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....T. TONGUE

LICENSING PROJ MANAGER.....R. GILBERT
DOCKET NUMBER.....50-237

LICENSE & DATE ISSUANCE....DPR-19, DECEMBER 22, 1969

PUBLIC DOCUMENT ROOM.....MORRIS PUBLIC LIBRARY
604 LIBERTY STREET
MORRIS, ILLINOIS 60450

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON NOVEMBER 18, THROUGH JANUARY 19, (83-32): ROUTINE, UNANNOUNCED INSPECTION BY THREE RESIDENT INSPECTORS OF PREVIOUS INSPECTION FINDINGS; HEADQUARTERS/REGIONAL REQUESTS; OPERATIONAL SAFETY; I.E. BULLETINS; MONTHLY MAINTENANCE; MONTHLY SURVEILLANCES; REFUELING ACTIVITIES; REFUELING SURVEILLANCES; ONSITE FOLLOWUP OF EVENTS; LICENSEE EVENT REPORTS; SYSTEMATIC EVALUATION PROGRAM; PERFORMANCE APPRAISAL STAFF FINDINGS; THREE MILE ISLAND MODIFICATIONS; SPENT NUCLEAR FUEL SHIPMENTS AND REPORT REVIEW. THE INSPECTION INVOLVED A TOTAL OF 352 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 45 INSPECTOR-HOURS ONSITE DURING OFFSHIFTS. OF THE 15 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 13 AREAS; TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS (FAILURE TO HAVE A PROCEDURE AND FAILURE TO FOLLOW OR HAVE ADEQUATE PROCEDURES).

INSPECTION ON NOVEMBER 9-10 AND DECEMBER 5-9, 12, AND 20 (83-33): ROUTINE, UNANNOUNCED INSPECTION OF RADIATION PROTECTION PROGRAM, INCLUDING QUALIFICATIOS, TRAINING, EXPOSURE CONTROL, POSTING AND CONTROL, SURVEYS, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, AND THE CIRCUMSTANCES SURROUNDING THE EVENT WHICH ALLOWED UNAUTHORIZED ENTRY OF TWO CONTRACT EMPLOYEES INTO A HIGH RADIATION AREA. THE INSPECTION INVOLVED 85 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (FAILURE TO GIVE TRAINING, FAILURE TO CONTROL ENTRY INTO A HIGH RADIATION AREA AND MAINTAIN LOCKED A HIGH RADIATION AREA, FAILURE TO PROVIDE PERSONAL DOSIMETERS TO PERSONS ENTERING A HIGH RADIATION AREA, AND FAILURE TO MAKE ADEQUATE SURVEYS.

SPECIAL INSPECTION ON FEBRUARY 3 - 14,(84-01): INSPECTION FOLLOWUP ON COMMONWEALTH EDISON COMPANY REMARKS TO SPECIAL INSPECTION REPORT CONCERNING ALLEGATIONS OF IMPROPER OPERATION AT DRESDEN, QUAD CITIES, AND ZION NUCLEAR POWER PLANTS. THE INSPECTION INVOLVED 14 INSPECTION-HOURS BY THREE NRC INSPECTORS. MEASURES TO CORRECT IDENTIFIED WEAKNESSES WERE TAKEN AS DESCRIBED IN THE REPONSE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SUMMARY

INSPECTION ON JANUARY 10-13, 16, 18-21, AND 23-26, (84-02): A SPECIAL, ANNOUNCED INSPECTION BY TWO RESIDENT INSPECTORS AND ONE REGION III SECTION CHIEF OF THE ACTIVITIES RELATED TO AN EVENT WHERE CONTROL RODS WERE INSERTED OUT OF SEQUENCE DURING A REACTOR SHUTDOWN ON JANUARY 9, 1984. THE INSPECTION INVOLVED A TOTAL OF 62 INSPECTOR-HOURS ONSITE BY 3 NRC INSPECTORS INCLUDING 23 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. THE INSPECTION RESULTED IN IDENTIFYING TWO ITEMS OF NONCOMPLIANCE. (FAILURE TO FOLLOW THE CONTROL ROD SEQUENCE PROCEDURE AND FAILURE TO FOLLOW THE OUT-OF-SERVICE PROCEDURE).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: JANUARY 23 THROUGH FEBRUARY 3, 1984

INSPECTION REPORT NO: 84-04

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-62/ 01X-1	08/19/83	02/16/84	HPCI MOTOR MOVED BETWEEN HI AND LOW SPEED STOPS WITHOUT OPERATION ACTION.
84-01/	01/02/84	01/25/84	D2 MAIN STEAM LINE LOW PRESSURE ISOLATION.
84-02/	01/17/84	02/07/84	STANDBY GAS TREATMENT 'B' FAN DISCHARGE DAMPER FOUND TRIPPED.

1. Docket: 50-249 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: BEN SCHROEDER (815) 942-2920
4. Licensed Thermal Power (MWt): 2527
5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828
6. Design Electrical Rating (Net MWe): 794
7. Maximum Dependable Capacity (Gross MWe): 812
8. Maximum Dependable Capacity (Net MWe): 773
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	110,545.0
13. Hours Reactor Critical	.0	.0	82,835.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	79,862.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	*59,963,004
18. Gross Elec Ener (MWH)	0	0	51,952,909
19. Net Elec Ener (MWH)	-2,250	-5,010	49,225,573
20. Unit Service Factor	.0	.0	72.2
21. Unit Avail Factor	.0	.0	72.2
22. Unit Cap Factor (MDC Net)	.0	.0	57.6
23. Unit Cap Factor (DER Net)	.0	.0	56.1
24. Unit Forced Outage Rate	.0	.0	12.6
25. Forced Outage Hours	.0	.0	6,415.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

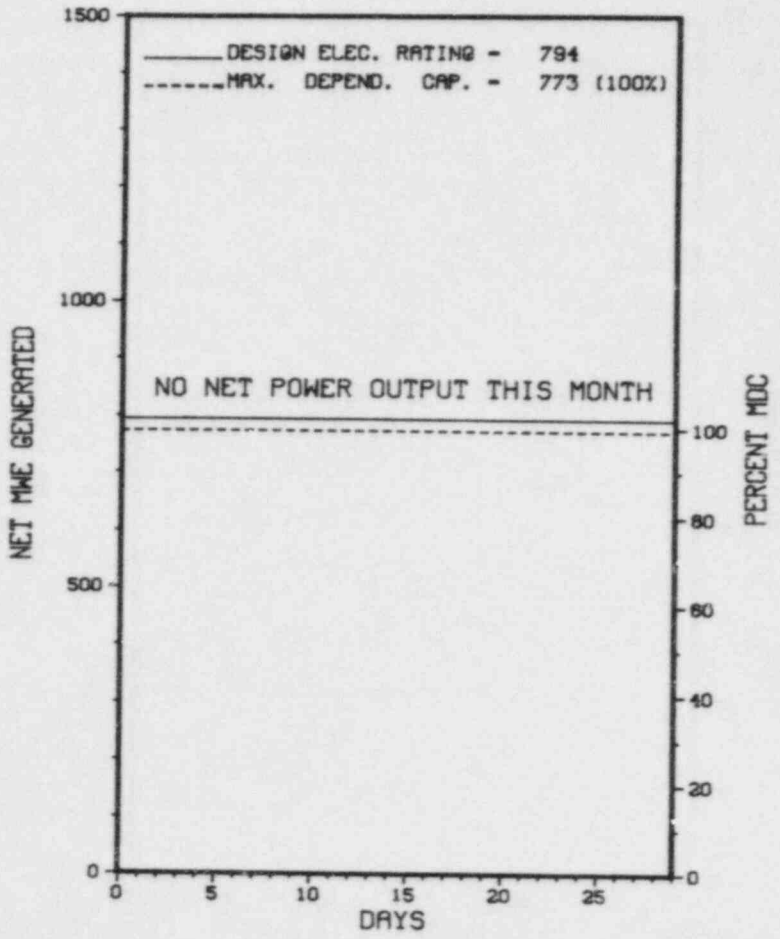
NONE

27. If Currently Shutdown Estimated Startup Date: 03/15/84

****
* DRESDEN 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DRESDEN 3



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

DRESDEN 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
8	09/30/83	S	696.0	C	4				REFUELING, ISI AND TURBINE OVERHAUL OUTAGE CONTINUES.

SUMMARY

DRESDEN 3 REMAINS SHUTDOWN IN A CONTINUING REFUELING AND REPAIR OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* DRESDEN 3 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....GRUNDY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9 MI E OF
MORRIS, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JANUARY 31, 1971

DATE ELEC ENER 1ST GENER...JULY 22, 1971

DATE COMMERCIAL OPERATE....NOVEMBER 16, 1971

CONDENSER COOLING METHOD...COOLING LAKE

CONDENSER COOLING WATER....KANKAKEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....T. TONGUE

LICENSING PROJ MANAGER.....R. GILBERT
DOCKET NUMBER.....50-249

LICENSE & DATE ISSUANCE....DPR-25, MARCH 2, 1971

PUBLIC DOCUMENT ROOM.....MORRIS PUBLIC LIBRARY
604 LIBERTY STREET
MORRIS, ILLINOIS 60450

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON NOVEMBER 18, THROUGH JANUARY 19, (83-30): ROUTINE, UNANNOUNCED INSPECTION BY THREE RESIDENT INSPECTORS OF PREVIOUS INSPECTION FINDINGS; HEADQUARTERS/REGIONAL REQUESTS; OPERATIONAL SAFETY; I.E. BULLETINS; MONTHLY MAINTENANCE; MONTHLY SURVEILLANCES; REFUELING ACTIVITIES; REFUELING SURVEILLANCES; ONSITE FOLLOWUP OF EVENTS; LICENSEE EVENT REPORTS; SYSTEMATIC EVALUATION PROGRAM; PERFORMANCE APPRAISAL STAFF FINDINGS; THREE MILE ISLAND MODIFICATIONS; SPENT NUCLEAR FUEL SHIPMENTS AND REPORT REVIEW. THE INSPECTION INVOLVED A TOTAL OF 352 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 45 INSPECTOR-HOURS ONSITE DURING OFFSHIFTS. OF THE 15 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 13 AREAS; TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS (FAILURE TO HAVE A PROCEDURE AND FAILURE TO FOLLOW OR HAVE ADEQUATE PROCEDURES).

INSPECTION ON NOVEMBER 9-10 AND DECEMBER 5-9, 12, AND 20 (83-31): ROUTINE, UNANNOUNCED INSPECTION OF RADIATION PROTECTION PROGRAM, INCLUDING QUALIFICATIOS, TRAINING, EXPOSURE CONTROL, POSTING AND CONTROL, SURVEYS, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, AND THE CIRCUMSTANCES SURROUNDING THE EVENT WHICH ALLOWED UNAUTHORIZED ENTRY OF TWO CONTRACT EMPLOYEES INTO A HIGH RADIATION AREA. THE INSPECTION INVOLVED 85 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (FAILURE TO GIVE TRAINING, FAILURE TO CONTROL ENTRY INTO A HIGH RADIATION AREA AND MAINTAIN LOCKED A HIGH RADIATION AREA, FAILURE TO PROVIDE PERSONAL DOSIMETERS TO PERSONS ENTERING A HIGH RADIATION AREA, AND FAILURE TO MAKE ADEQUATE SURVEYS.

SPECIAL INSPECTION ON FEBRUARY 3 - 14,(84-01): INSPECTION FOLLOWUP ON COMMONWEALTH EDISON COMPANY REMARKS TO SPECIAL INSPECTION REPORT CONCERNING ALLEGATIONS OF IMPROPER OPERATION AT DRESDEN, QUAD CITIES, AND ZION NUCLEAR POWER PLANTS. THE INSPECTION INVOLVED 14 INSPECTION-HOURS BY THREE NRC INSPECTORS. MEASURES TO CORRECT IDENTIFIED WEAKNESSES WERE TAKEN AS DESCRIBED IN THE REPONSE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

****
* DRESDEN 3 *
****

INSPECTION SUMMARY

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT WAS SHUT DOWN ON 9/30/83 FOR AN EXTENDED REFUELING AND MAINTENANCE OUTAGE. THE LICENSEE ANTICIPATES RETURNING THE UNIT TO SERVICE IN ON MARCH 11, 1984.

LAST IE SITE INSPECTION DATE: JANUARY 23 THROUGH FEBRUARY 3, 1984

INSPECTION REPORT NO: 84-03

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-331 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: MATT ANDERSON (319) 851-7308
4. Licensed Thermal Power (MWt): 1658
5. Nameplate Rating (Gross MWe): 663 X 0.9 = 597
6. Design Electrical Rating (Net MWe): 538
7. Maximum Dependable Capacity (Gross MWe): 545
8. Maximum Dependable Capacity (Net MWe): 515
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	79,584.0
13. Hours Reactor Critical	696.0	1,274.6	57,209.6
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,248.9	55,691.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,098,451	1,875,408	69,623,970
18. Gross Elec Ener (MWH)	374,414	636,841	23,330,898
19. Net Elec Ener (MWH)	353,529	601,011	21,837,381
20. Unit Service Factor	100.0	86.7	70.0
21. Unit Avail Factor	100.0	86.7	70.0
22. Unit Cap Factor (MDC Net)	98.6	81.0	53.3
23. Unit Cap Factor (DER Net)	94.4	77.6	51.0
24. Unit Forced Outage Rate	.0	13.3	17.2
25. Forced Outage Hours	.0	191.1	11,525.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

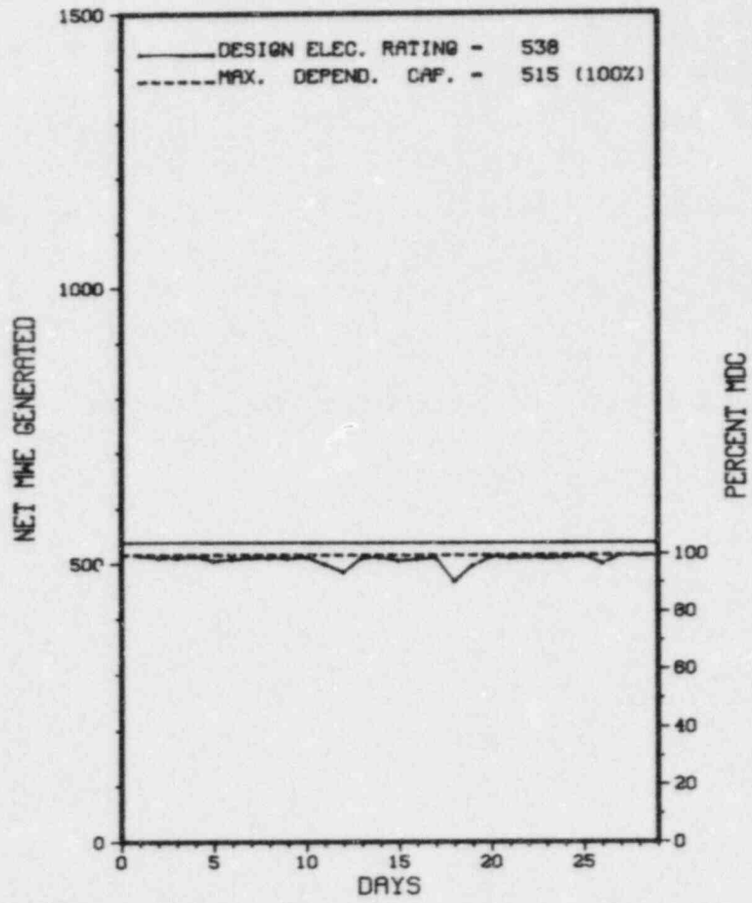
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* DUANE ARNOLD *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DUANE ARNOLD



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

DUANE ARNOLD

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

SUMMARY

DUANE ARNOLD OPERATED AT OR NEAR FULL POWER DURING THE REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* DUANE ARNOLD *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....IOWA

COUNTY.....LINN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...8 MI NW OF
CEDAR RAPIDS, IA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MARCH 23, 1974

DATE ELEC ENER 1ST GENER...MAY 19, 1974

DATE COMMERCIAL OPERATE....FEBRUARY 1, 1975

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CEDAR RAPIDS RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....IOWA ELECTRIC POWER & LIGHT

CORPORATE ADDRESS.....I E TOWERS, P.O. BOX 351
CEDAR RAPIDS, IOWA 52406

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....L. CLARDY

LICENSING PROJ MANAGER.....M. THADANI
DOCKET NUMBER.....50-331

LICENSE & DATE ISSUANCE....DPR-49, FEBRUARY 22, 1974

PUBLIC DOCUMENT ROOM.....REFERENCE SERVICE
CEDAR RAPIDS PUBLIC LIBRARY
428 THIRD AVENUE, S.E.
CEDAR RAPIDS, IOWA 52401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JANUARY 9-13, (84-01): ROUTINE, UNANNOUNCED INSPECTION OF RADIATION PROTECTION PROGRAM, INCLUDING: ORGANIZATION AND MANAGEMENT CONTROL; QUALIFICATIONS AND TRAINING; EXTERNAL EXPOSURE CONTROL AND PERSONAL DOSIMETRY; INTERNAL EXPOSURE CONTROL AND ASSESSMENT; ALARA; AND CONTROL OF RADIOACTIVE MATERIALS. THE INSPECTION INVOLVED 74 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JANUARY 3 - JANUARY 31, (84-02): ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTOR OF LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS; OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; PLANT TRIPS; LICENSEE IDENTIFIED ITEMS; IE BULLETINS; REGIONAL REQUESTS; PERSONNEL ERRORS; UNUSUAL EVENTS; PROCEDURES; AND INDEPENDENT INSPECTION. THE INSPECTION INVOLVED A TOTAL OF 88 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR INCLUDING 12 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 12 AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (FAILURE TO FOLLOW PROCEDURES).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: JANUARY 1-31, 1984

INSPECTION REPORT NO: 84-02

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-48/ 03L-0	12/30/83	01/27/84	B CONTAINMENT ATMOSPHERE MONITOR DECLARED INOP DUE TO INADEQUATE PROCESS LINE FLOW.
84-01	01/07/84	02/06/84	SCRAM DUE TO F.W. REDUCTION AND SRV ACTUATION.
84-02	01/03/84	02/02/84	HPCI INOPERABLE.
84-03	01/02/84	02/01/84	CONTROL ROOM INTAKE AIR STANDBY FILTER UNITS INITIATION.
84-04	01/08/84	02/09/84	CONTROL ROOM AIR TREATMENT SYSTEM - SFU INITIATION FAILURES.
84-05	01/28/84	02/01/84	RHR/CORE SPRAY FILL ENERGY CENTER.
84-06	01/26/84	02/24/84	H&V DAMPER ACTUATORS DOCUMENTATION DEFICIENCIES.
84-07	01/27/84	02/26/84	1/2 GROUP III ISOLATION.
	84-08	01/23/84	02/22/84 REACTOR SCRAM DUE TO TOO FEW LPRM'S FOR APRM'S.
83-47/	12/20/83	01/19/84	ASME IN-SERVICE TEST FOUND 15.2 SEC CLOSING OF INBOARD MAIN STEAMLINE DRAIN VALVE (MO-442[illegible]).

03L-0

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1. Docket: 50-348 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: DENNIS HERRIN (205) 899-5156
4. Licensed Thermal Power (MWt): 2652
5. Nameplate Rating (Gross MWe): 1045 X 0.85 = 888
6. Design Electrical Rating (Net MWe): 829
7. Maximum Dependable Capacity (Gross MWe): 845
8. Maximum Dependable Capacity (Net MWe): 804
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	54,768.0
13. Hours Reactor Critical	240.0	938.0	36,061.2
14. Rx Reserve Shtdwn Hrs	.0	.0	3,650.7
15. Hrs Generator On-Line	240.0	904.5	35,007.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	631,428	2,360,784	88,462,308
18. Gross Elec Ener (MWH)	203,000	751,662	27,993,526
19. Net Elec Ener (MWH)	188,394	706,670	26,407,732
20. Unit Service Factor	34.5	62.8	63.9
21. Unit Avail Factor	34.5	62.8	63.9
22. Unit Cap Factor (MDC Net)	33.7	61.0	60.5*
23. Unit Cap Factor (DER Net)	32.7	59.2	58.2
24. Unit Forced Outage Rate	.0	8.1	15.2
25. Forced Outage Hours	.0	79.5	6,246.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

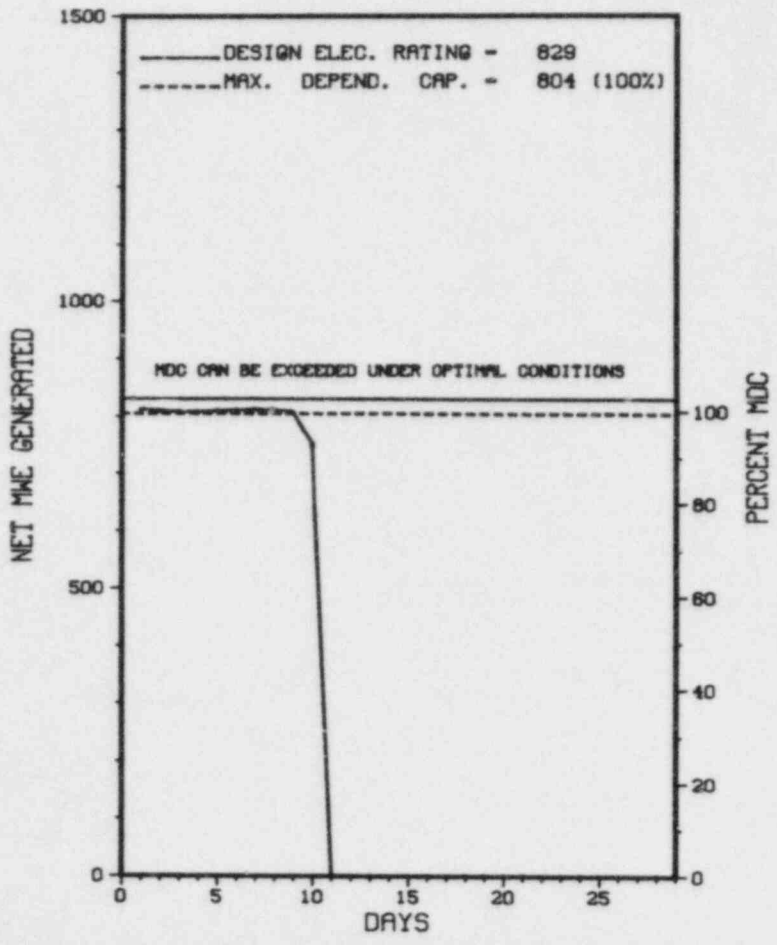
NONE

27. If Currently Shutdown Estimated Startup Date: 04/14/84

****
* FARLEY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FARLEY 1



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

FARLEY 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
003	02/10/84	S	456.0	C	3	84-002-00			THE UNIT WAS TAKEN OFF LINE FOR THE CYCLE V-VI REFUELING OUTAGE. DURING NORMAL SHUTDOWN OPERATIONS, A REACTOR TRIP OCCURRED FROM 10% POWER.

SUMMARY

FARLEY 1 SHUTDOWN ON FEBRUARY 10TH FOR REFUELING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FARLEY 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....HOUSTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...28 MI SE OF
DOTHAN, ALA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 9, 1977

DATE ELEC ENER 1ST GENER...AUGUST 18, 1977

DATE COMMERCIAL OPERATE....DECEMBER 1, 1977

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CHATAHOOCHEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ALABAMA POWER CO.

CORPORATE ADDRESS.....600 NORTH 18TH STREET
BIRMINGHAM, ALABAMA 35203

CONTRACTOR
ARCHITECT/ENGINEER.....SOUTHERN SERVICES INCORPORATED

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. BRADFORD

LICENSING PROJ MANAGER.....E. REEVES
DOCKET NUMBER.....50-348

LICENSE & DATE ISSUANCE....NPF-2, JUNE 25, 1977

PUBLIC DOCUMENT ROOM.....G.S. HOUSTON MEMORIAL LIBRARY
212 W. BURDESHAW STREET
DOTHAN, ALABAMA 36301

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 11, 1983 - JANUARY 10, 1984 (83-33): THIS ROUTINE INSPECTION INVOLVED 90 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT STATUS, MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, INDEPENDENT INSPECTION EFFORT, PHYSICAL PROTECTION, TECHNICAL SPECIFICATION COMPLIANCE, LICENSEE EVENT REPORTS, OPERATOR ACTIONS FOR REACTOR TRIP AND ANTICIPATED TRANSIENTS WITHOUT A TRIP (ATWT), AND REACTOR TRIP BREAKERS. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE FOUND IN SEVEN AREAS; ONE VIOLATION WAS FOUND IN ONE AREA (VIOLATION OF TECHNICAL SPECIFICATION 6.8.1, PARAGRAPH 6) AND ONE VIOLATION WAS FOUND IN ONE AREA (VIOLATION OF 10 CFR 50, APPENDIX B, CRITERION XVI, PARAGRAPH 8).

INSPECTION JANUARY 18-19 (84-01): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE IN THE EMERGENCY PREPAREDNESS AREA OF PROTECTIVE ACTION DECISION MAKING. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 16-20 (84-02): THE INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. TWO INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED REVIEW OF SECURITY ORGANIZATION (PERSONNEL AND RESPONSE); SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); SECURITY SYSTEM POWER SUPPLY; ASSESSMENT AIDS; ACCESS CONTROLS (PERSONNEL, PACKAGE AND VEHICLES); DETECTION AIDS (PROTECTED AND VITAL AREAS); ALARM STATIONS; AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 12 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION JANUARY 11 - FEBRUARY 5 (84-03): THIS ROUTINE INSPECTION INVOLVED 80 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT STATUS, MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, INDEPENDENT INSPECTION

****
* FARLEY 1 *
****

INSPECTION SUMMARY

EFFORT, PHYSICAL PROTECTION, TECHNICAL SPECIFICATION COMPLIANCE, LICENSEE EVENT REPORTS, CIRCULAR FOLLOW-UP, PLANT TRIPS, TMI ACTION ITEM, CONTROL ROOM MANNING, AND CONTROL OF HEAVY LOADS. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 12 AREAS; ONE VIOLATION WAS FOUND IN 1 AREA (VIOLATION OF TECHNICAL SPECIFICATION 6.8.1, PARAGRAPH 7).

INSPECTION JANUARY 31 - FEBRUARY 2 (84-04): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF AN EMERGENCY EXERCISE. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES SHALL BE IMPLEMENTED FOR SURVEILLANCE TEST PROCEDURES AND OPERATING PROCEDURES. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT IMPLEMENT SURVEILLANCE AND OPERATING PROCEDURES AS FOLLOWS: (A) ALTHOUGH FNP-1-STP-201.6B REQUIRES THAT A SIGN-OFF SHEET BE INITIALED AS EACH STEP IS COMPLETED, SUCH SIGN-OFFS WERE INITIATED AFTER SIX STEPS WERE PERFORMED. (B) ALTHOUGH FNP-1-SOP-2.6C REQUIRES VALVE VERIFICATION, ON DECEMBER 13, 1984, NO VERIFICATION WAS PERFORMED ON A CHECKLIST THAT THE FIRST MAN PERFORMED ON OCTOBER 22, 1983. (C) ALTHOUGH FNP-2-STP-256.15 REQUIRED THE REMOVAL OF A TEST TRANSMITTER ON OCTOBER 18, 1983, THE TEST TRANSMITTER WAS NOT REMOVED AS OF DECEMBER 20, 1983 AND NO EXCEPTION WAS NOTED DURING THE PROCEDURE REVIEW O N DECEMBER 14, 1983. (D) ALTHOUGH FNP-2-STP-24.7 REQUIRES MOV 536 TO BE VERIFIED OPEN, MOV 536 WAS NOT VERIFIED OPEN ON DECEMBER 29, 1983. THE INSPECTOR FOUND MOV 536 SHUT ON DECEMBER 30, 1983. THE FOUR ITEMS NOTED ABOVE ARE EXAMPLES OF FAILURE TO FOLLOW ESTABLISHED, APPROVED PROCEDURES. 10 CFR 50, APPENDIX B, CRITERION XVI, AS IMPLEMENTED BY THE LICENSEE'S OPERATIONAL QUALITY ASSURANCE MANUAL, REQUIRES THE LICENSEE TO ESTABLISH MEASURES TO ASSURE THAT, IN THE CASE OF SIGNIFICANT CONDITIONS ADVERSE TO QUALITY, CORRECTIVE ACTION IS TAKEN TO PREVENT REPETITION. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO ESTABLISH CORRECTIVE MEASURES TO INSURE THAT SERVICE WATER VALVE MOV 536 REMAINED OPEN TO SUPPLY TRAIN B, UNIT 2, SERVICE WATER COOLING FROM THE DIESEL GENERATORS. MOV 536 WAS FOUND INADVERTENTLY CLOSED ON THREE OCCASIONS: MARCH 31, 1981, SEPTEMBER 25, 1983 AND DECEMBER 30, 1983.
(8333 4)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES SHALL BE IMPLEMENTED FOR THE CONTROL OF MAINTENANCE ACTIVITIES. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT IMPLEMENT PROCEDURES FOR THE CONTROL OF MAINTENANCE ACTIVITES. FNP-0-AP-16 REQUIRES THAT A CLEAR AND CONCISE DESCRIPTION OF THE CAUSE OF A PROBLEM BE DOCUMENTED ON A MAINTENANCE WORK REQUEST. THE LICENSEE DISASSEMBLED AN RHR SUCTION RELIEF VALVE AND DISCOVERED THAT THE DISC RING PINS WERE MISSING. NO ENTRY WAS MADE ON A MAINTENANCE WORK REQUEST CONCERNING THE MISSING RINGS.

(8403 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

I N S P E C T I O N S T A T U S - (CONTINUED)

FARLEY

OTHER ITEMS

PLANT STATUS:

SHUTDOWN FOR REFUELING:

LAST IE SITE INSPECTION DATE: JANUARY 31 - FEBRUARY 2, 1984 +

INSPECTION REPORT NO: 50-348/84-04 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-364 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: DENNIS HERRIN (205) 899-5156

4. Licensed Thermal Power (MWt): 2652

5. Nameplate Rating (Gross MWe): 860

6. Design Electrical Rating (Net MWe): 829

7. Maximum Dependable Capacity (Gross MWe): 855

8. Maximum Dependable Capacity (Net MWe): 814

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	22,681.0
13. Hours Reactor Critical	696.0	1,415.8	19,952.6
14. Rx Reserve Shtdwn Hrs	.0	.0	138.4
15. Hrs Generator On-Line	696.0	1,378.7	19,677.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,845,593	3,557,486	50,468,178
18. Gross Elec Ener (MWH)	607,908	1,164,766	16,151,614
19. Net Elec Ener (MWH)	579,714	1,107,326	15,307,352
20. Unit Service Factor	100.0	95.7	86.8
21. Unit Avail Factor	100.0	95.7	86.8
22. Unit Cap Factor (MDC Net)	102.3	94.5	82.9
23. Unit Cap Factor (DER Net)	100.5	92.8	81.4
24. Unit Forced Outage Rate	.0	4.3	5.3
25. Forced Outage Hours	.0	61.3	1,093.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

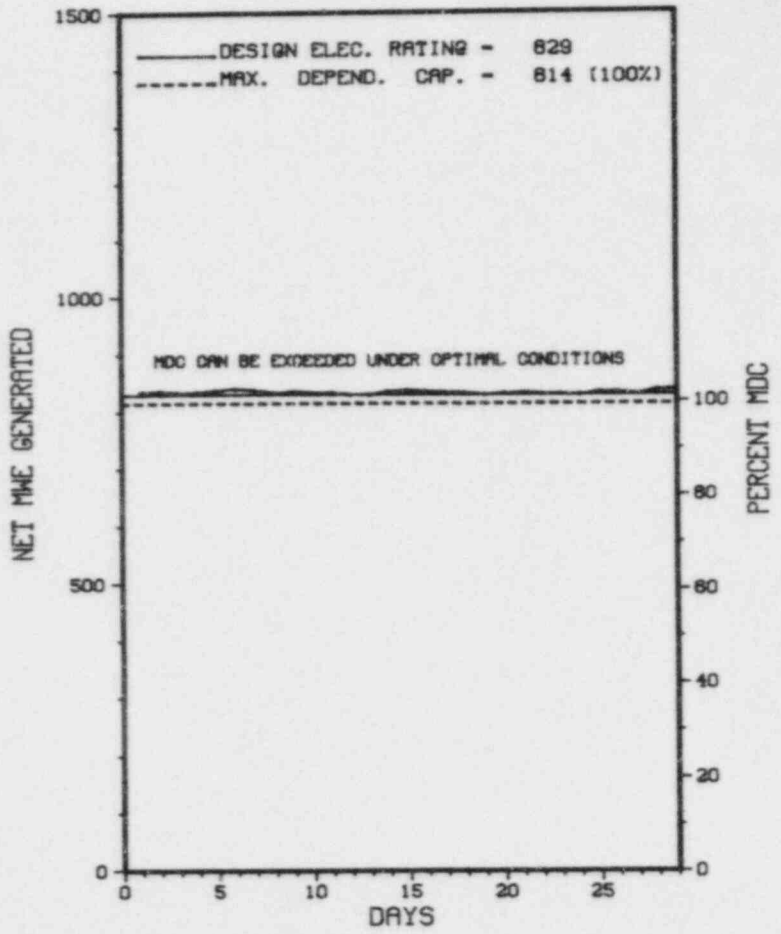
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* FARLEY 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FARLEY 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

FARLEY 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

FARLEY 2 OPERATED AT FULL POWER DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FARLEY 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....HOUSTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...28 MI SE OF
DOTHAN, ALA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 5, 1981

DATE ELEC ENER 1ST GENER...MAY 25, 1981

DATE COMMERCIAL OPERATE....JULY 30, 1981

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CHATAHOOCHEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ALABAMA POWER CO.

CORPORATE ADDRESS.....600 NORTH 18TH STREET
BIRMINGHAM, ALABAMA 35203

CONTRACTOR
ARCHITECT/ENGINEER.....SOUTHERN SERVICES INCORPORATED

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. BRADFORD

LICENSING PROJ MANAGER.....E. REEVES
DOCKET NUMBER.....50-364

LICENSE & DATE ISSUANCE....NPF-8, MARCH 31, 1981

PUBLIC DOCUMENT ROOM.....G.S. HOUSTON MEMORIAL LIBRARY
212 W. BURDESHAW STREET
DOTHAN, ALABAMA 36301

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 11, 1983 - JANUARY 10, 1984 (83-31): THIS ROUTINE INSPECTION INVOLVED 90 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT STATUS, MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, INDEPENDENT INSPECTION EFFORT, PHYSICAL PROTECTION, TECHNICAL SPECIFICATION COMPLIANCE, LICENSEE EVENT REPORTS, OPERATOR ACTIONS FOR REACTOR TRIP AND ANTICIPATED TRANSIENTS WITHOUT A TRIP (ATWT), AND REACTOR TRIP BREAKERS. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE FOUND IN SEVEN AREAS; ONE VIOLATION WAS FOUND IN ONE AREA (VIOLATION OF TECHNICAL SPECIFICATION 6.8.1, PARAGRAPH 6) AND ONE VIOLATION WAS FOUND IN ONE AREA (VIOLATION OF 10 CFR 50, APPENDIX B, CRITERION XVI, PARAGRAPH 8).

INSPECTION JANUARY 18-19 (84-01): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE IN THE EMERGENCY PREPAREDNESS AREA OF PROTECTIVE ACTION DECISION MAKING. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 16-20 (84-02): THE INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. TWO INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED REVIEW OF SECURITY ORGANIZATION (PERSONNEL AND RESPONSE); SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); SECURITY SYSTEM POWER SUPPLY; ASSESSMENT AIDS; ACCESS CONTROLS (PERSONNEL, PACKAGE AND VEHICLES); DETECTION AIDS (PROTECTED AND VITAL AREAS); ALARM STATIONS; AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 12 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION JANUARY 11 - FEBRUARY 5 (84-03): THIS ROUTINE INSPECTION INVOLVED 80 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT STATUS, MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, INDEPENDENT INSPECTION

INSPECTION SUMMARY

EFFORT, PHYSICAL PROTECTION, TECHNICAL SPECIFICATION COMPLIANCE, LICENSEE EVENT REPORTS, CIRCULAR FOLLOW-UP, PLANT TRIPS, TMI ACTION ITEM, CONTROL ROOM MANNING, AND CONTROL OF HEAVY LOADS. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 12 AREAS; ONE VIOLATION WAS FOUND IN ONE AREA (VIOLATION OF TECHNICAL SPECIFICATION 6.8.1, PARAGRAPH 7).

INSPECTION JANUARY 31 - FEBRUARY 2 (84-04): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF AN EMERGENCY EXERCISE. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES SHALL BE IMPLEMENTED FOR SURVEILLANCE TEST PROCEDURES AND OPERATING PROCEDURES. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT IMPLEMENT SURVEILLANCE AND OPERATING PROCEDURES AS FOLLOWS: (A) ALTHOUGH FNP-1-STP-201.6B REQUIRES THAT A SIGN-OFF SHEET BE INITIALED AS EACH STEP IS COMPLETED, SUCH SIGN-OFFS WERE INITIATED AFTER SIX STEPS WERE PERFORMED. (B) ALTHOUGH FNP-1-SOP-2.6C REQUIRES VALVE VERIFICATION, ON DECEMBER 13, 1984, NO VERIFICATION WAS PERFORMED ON A CHECKLIST THAT THE FIRST MAN PERFORMED ON OCTOBER 22, 1983. (C) ALTHOUGH FNP-2-STP-256.15 REQUIRED THE REMOVAL OF A TEST TRANSMITTER ON OCTOBER 18, 1983, THE TEST TRANSMITTER WAS NOT REMOVED AS OF DECEMBER 20, 1983 AND NO EXCEPTION WAS NOTED DURING THE PROCEDURE REVIEW O N DECEMBER 14, 1983. (D) ALTHOUGH FNP-2-STP-24.7 REQUIRES MOV 536 TO BE VERIFIED OPEN, MOV 536 WAS NOT VERIFIED OPEN ON DECEMBER 29, 1983. THE INSPECTOR FOUND MOV 536 SHUT ON DECEMBER 30, 1983. THE FOUR ITEMS NOTED ABOVE ARE EXAMPLES OF FAILURE TO FOLLOW ESTABLISHED, APPROVED PROCEDURES. 10 CFR 50, APPENDIX B, CRITERION XVI, AS IMPLEMENTED BY THE LICENSEE'S OPERATIONAL QUALITY ASSURANCE MANUAL, REQUIRES THE LICENSEE TO ESTABLISH MEASURES TO ASSURE THAT, IN THE CASE OF SIGNIFICANT CONDITIONS ADVERSE TO QUALITY, CORRECTIVE ACTION IS TAKEN TO PREVENT REPETITION. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO ESTABLISH CORRECTIVE MEASURES TO INSURE THAT SERVICE WATER VALVE MOV 536 REMAINED OPEN TO SUPPLY TRAIN B, UNIT 2, SERVICE WATER COOLING FROM THE DIESEL GENERATORS. MOV 536 WAS FOUND INADVERTENTLY CLOSED ON THREE OCCASIONS: MARCH 31, 1981, SEPTEMBER 25, 1983 AND DECEMBER 30, 1983.
(8331 4)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES SHALL BE IMPLEMENTED FOR THE CONTROL OF MAINTENANCE ACTIVITIES. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT IMPLEMENT PROCEDURES FOR THE CONTROL OF MAINTENANCE ACTIVITES. FNP-O-AP-16 REQUIRES THAT A CLEAR AND CONCISE DESCRIPTION OF THE CAUSE OF A PROBLEM BE DOCUMENTED ON A MAINTENANCE WORK REQUEST. THE LICENSEE DISASSEMBLED AN RHR SUCTION RELIEF VALVE AND DISCOVERED THAT THE DISC RING PINS WERE MISSING. NO ENTRY WAS MADE ON A MAINTENANCE WORK REQUEST CONCERNING THE MISSING RINGS.

(8403 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

OTHER ITEMS

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JANUARY 31 - FEBRUARY 2, 1984 +

INSPECTION REPORT NO: 50-364/84-04 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-333 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: J. COOK (315) 342-3840
4. Licensed Thermal Power (MWt): 2436
5. Nameplate Rating (Gross MWe): 981 X 0.9 = 883
6. Design Electrical Rating (Net MWe): 821
7. Maximum Dependable Capacity (Gross MWe): 830
8. Maximum Dependable Capacity (Net MWe): 810
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	75,337.0
13. Hours Reactor Critical	696.0	1,440.0	53,968.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,440.0	52,639.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,627,008	3,434,472	111,171,058
18. Gross Elec Ener (MWH)	546,230	1,155,820	37,813,140
19. Net Elec Ener (MWH)	528,615	1,118,290	36,616,930
20. Unit Service Factor	100.0	100.0	69.9
21. Unit Avail Factor	100.0	100.0	69.9
22. Unit Cap Factor (MDC Net)	93.8	95.9	63.6*
23. Unit Cap Factor (DER Net)	92.5	94.6	59.2
24. Unit Forced Outage Rate	.0	.0	14.2
25. Forced Outage Hours	.0	.0	8,883.2

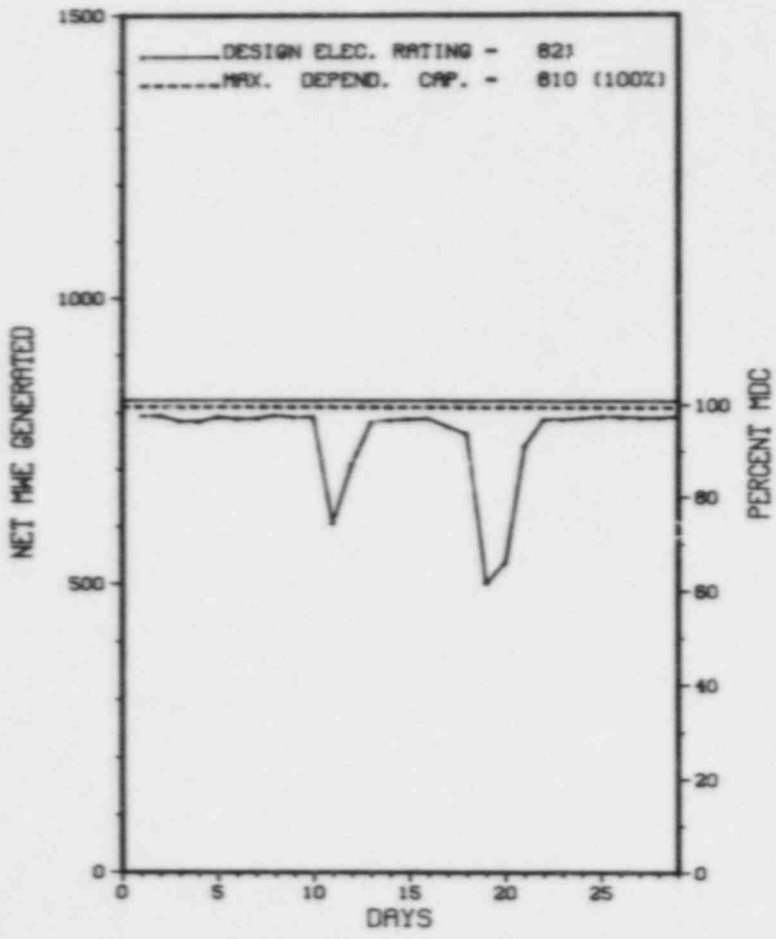
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

MAINTENANCE OUTAGE: 03/03/84 - 10 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* FITZPATRICK *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* FITZPATRICK *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	02/10/84	S	0.0	B	5				REDUCED POWER FOR ROD SEQUENCE EXCHANGE.
2	02/18/84	S	0.0	B	5				REDUCE POWER FOR CONTROL ROD PATTERN ADJUSTMENT.

****
* SUMMARY *
****

THE FITZPATRICK PLANT OPERATED AT NEAR FULL THERMAL POWER FOR THIS REPORTING PERIOD WITH ONE POWER REDUCTION FOR CONTROL ROD SEQUENCE EXCHANGE AND ONE FOR ROD PATTERN ADJUSTMENT.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report (LER) File (NUREG-0161)
			9-Other	

****
* FITZPATRICK *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....OSWEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...8 MI NE OF
OSWEGO, NY

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...NOVEMBER 17, 1974

DATE ELEC ENER 1ST GENER...FEBRUARY 1, 1975

DATE COMMERCIAL OPERATE....JULY 28, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ONTARIO

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....POWER AUTHORITY OF STATE OF N.Y.

CORPORATE ADDRESS.....10 COLUMBUS CIRCLE
NEW YORK, NEW YORK 10019

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....L. DOERFLEIN

LICENSING PROJ MANAGER.....H. ABELSON
DOCKET NUMBER.....50-333

LICENSE & DATE ISSUANCE....DPR-59, OCTOBER 17, 1974

PUBLIC DOCUMENT ROOM.....STATE UNIVERSITY COLLEGE OF OSWEGO
PENFIELD LIBRARY - GOVERNMENT DOCUMENTS COL
OSWEGO, NY 13126
(315) 341-2323

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-285 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: T. P. MATTHEWS (402) 536-4733
4. Licensed Thermal Power (MWt): 1500
5. Nameplate Rating (Gross MWe): 591 X 0.85 = 502
6. Design Electrical Rating (Net MWe): 478
7. Maximum Dependable Capacity (Gross MWe): 461
8. Maximum Dependable Capacity (Net MWe): 438
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	91,441.0
13. Hours Reactor Critical	696.0	1,440.0	72,053.9
14. Rx Reserve Shtdwn Hrs	.0	.0	1,309.5
15. Hrs Generator On-Line	696.0	1,440.0	70,792.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	997,883	2,103,771	88,363,485
18. Gross Elec Ener (MWH)	319,880	675,442	29,304,866
19. Net Elec Ener (MWH)	303,975	642,756	27,722,616
20. Unit Service Factor	100.0	100.0	77.4
21. Unit Avail Factor	100.0	100.0	77.4
22. Unit Cap Factor (MDC Net)	99.7	101.9	66.1*
23. Unit Cap Factor (DER Net)	91.4	93.4	63.4
24. Unit Forced Outage Rate	.0	.0	3.5
25. Forced Outage Hours	.0	.0	1,398.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

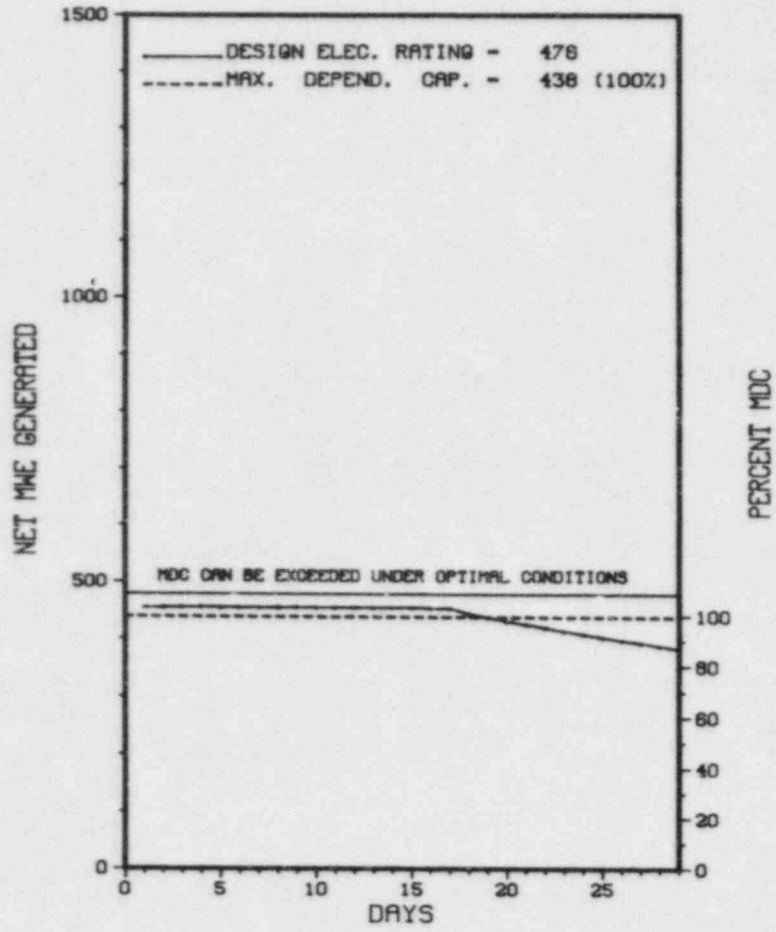
REFUELING - 03/03/84.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* FORT CALHOUN 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FORT CALHOUN 1



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* FORT CALHOUN 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

****
* SUMMARY *
****

FORT CALHOUN OPERATED AT OR NEAR FULL POWER DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FORT CALHOUN 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NEBRASKA

COUNTY.....WASHINGTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI N OF
OMAHA, NEB

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 6, 1973

DATE ELEC ENER 1ST GENER...AUGUST 25, 1973

DATE COMMERCIAL OPERATE....JUNE 20, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSOURI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....OMAHA PUBLIC POWER DISTRICT

CORPORATE ADDRESS.....1623 HARNEY STREET
OMAHA,, NEBRASKA 68102

CONTRACTOR
ARCHITECT/ENGINEER.....GIBBS, HILL, DURHAM & RICHARDSON

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....GIBBS, HILL, DURHAM & RICHARDSON

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....L. YANDELL

LICENSING PROJ MANAGER.....E. TOURIGNY
DOCKET NUMBER.....50-285

LICENSE & DATE ISSUANCE....DPR-40, AUGUST 9, 1973

PUBLIC DOCUMENT ROOM.....W. DALE CLARK LIBRARY
215 S. 15TH STREET
OMAHA, NEBRASKA 68102

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED DECEMBER 12-14, 1983 (83-36): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S TRANSPORTATION AND SOLID RADWASTE PROGRAM INCLUDING: MANAGEMENT CONTROLS, SELECTION OF PACKAGES, PREPARATION OF PACKAGES FOR SHIPMENT, DELIVERY OF COMPLETED PACKAGES TO CARRIER, PERIODIC MAINTENANCE OF PACKAGES, RECORDS AND REPORTS, 10 CFR PARTS 20 AND 61 REQUIREMENTS, AND LOW-LEVEL RADIOACTIVE WASTE STORAGE FACILITY. WITHIN THE NINE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED JANUARY 1-31, 1984 (84-01): ROUTINE, ANNOUNCED INSPECTION OF OPERATIONAL SAFETY VERIFICATION, ESF SYSTEM WALKDOWN, SURVEILLANCE TESTING, MAINTENANCE ACTIVITIES, SPENT FUEL STORAGE RACK MODIFICATIONS, PREPARATION FOR REFUELING, FOLLOWUP OF IE BULLETINS, AND FOLLOWUP OF NUREG 0737 (TMI) ITEMS. WITHIN THE EIGHT AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

REFUELING OUTAGE TO COMMENCE ON MARCH 2, 1984

LAST IE SITE INSPECTION DATE: JANUARY 31, 1984

INSPECTION REPORT NO: 50-285/84-01

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
L84-001	01/22/84	02/22/84	AUXILIARY BUILDING CRANE INTERLOCKS BYPASSED.

1. Docket: 50-267 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: C. H. FULLER (303) 785-2224
4. Licensed Thermal Power (MWt): 842
5. Nameplate Rating (Gross MWe): 403 X 0.85 = 343
6. Design Electrical Rating (Net MWe): 330
7. Maximum Dependable Capacity (Gross MWe): 342
8. Maximum Dependable Capacity (Net MWe): 330
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe): 280
11. Reasons for Restrictions, If Any:

B-O STARTUP TESTING.

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	40,921.0
13. Hours Reactor Critical	.0	468.0	26,295.3
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	446.6	18,250.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	240,819	9,610,571
18. Gross Elec Ener (MWH)	0	77,412	3,230,862
19. Net Elec Ener (MWH)	-1,952	69,350	2,940,880
20. Unit Service Factor	.0	31.0	44.6
21. Unit Avail Factor	.0	31.0	44.6
22. Unit Cap Factor (MDC Net)	.0	14.6	21.8
23. Unit Cap Factor (DER Net)	.0	14.6	21.8
24. Unit Forced Outage Rate	.0	1.5	39.0
25. Forced Outage Hours	.0	6.9	11,683.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

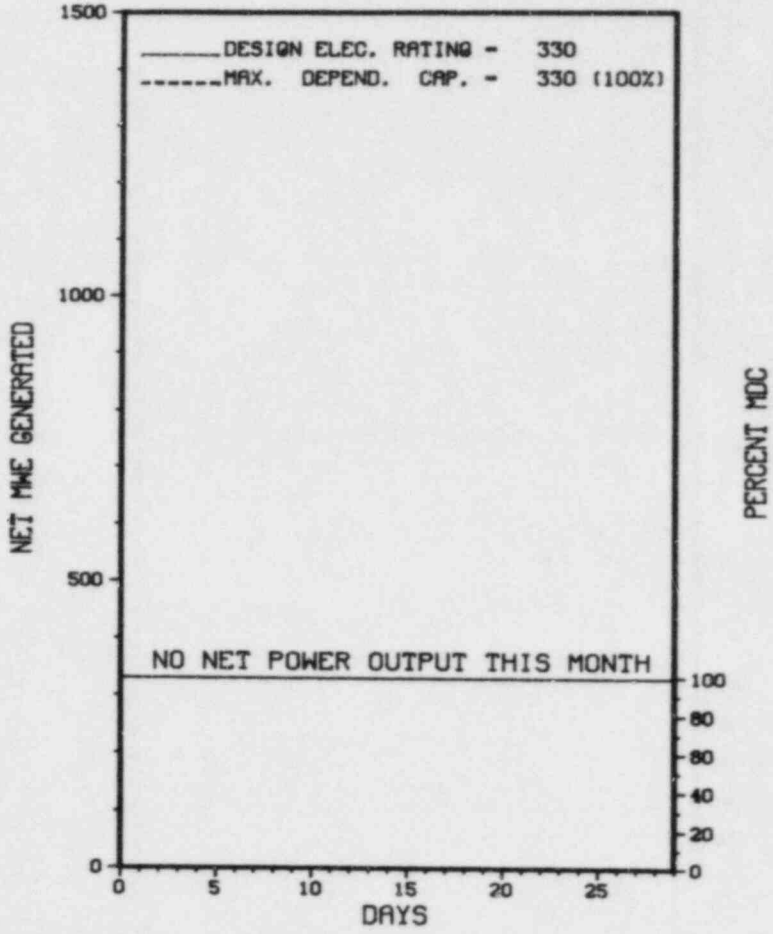
REFUELING - 03/01/84 THRU 05/02/84

27. If Currently Shutdown Estimated Startup Date: 05/02/84

****
* FORT ST VRAIN *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FORT ST VRAIN



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

FORT ST VRAIN

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-002	01/19/84	S	696.0	C	4		RC	FUELXX	REFUELING, TURBINE OVERHAUL, ROUTINE CORRECTIVE AND PREVENTIVE MAINTENANCE.

SUMMARY

FORT ST. VRAIN REMAINS OFF-LINE IN A CONTINUING REFUELING/MAINTENANCE OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FORT ST VRAIN *
****

FACILITY DATA

Report Period FEB 1984

ILITY DESCRIPTION

LOCATION
STATE.....COLORADO

COUNTY.....WELD

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...35 MI N OF
DENVER, COL

TYPE OF REACTOR.....HTGR

DATE INITIAL CRITICALITY...JANUARY 31, 1974

DATE ELEC ENER 1ST GENER...DECEMBER 11, 1976

DATE COMMERCIAL OPERATE....JULY 1, 1979

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....S. PLATTE RIVER

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PUBLIC SERVICE OF COLORADO

CORPORATE ADDRESS.....P.O. BOX 840
DENVER, COLORADO 80201

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ATOMIC CORP.

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....G. PLUMLEE

LICENSING PROJ MANAGER.....P. WAGNER
DOCKET NUMBER.....50-267

LICENSE & DATE ISSUANCE....DPR-34, DECEMBER 21, 1973

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CITY COMPLEX BUILDING
GREELEY, COLORADO 80631

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED JANUARY 16-20, 1984 (84-03): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S RADIOACTIVE WASTE PROGRAM INCLUDING: ORGANIZATION AND MANAGEMENT CONTROLS, TRAINING AND QUALIFICATIONS, AUDITS OF RADIOACTIVE WASTE ACTIVITIES, SOLID RADIOACTIVE WASTE PROCESSING, LIQUID AND GASEOUS RADIOACTIVE EFFLUENT RELEASES, PROCEDURES FOR CONTROLLING RADIOACTIVE EFFLUENT RELEASES, RADIOACTIVE EFFLUENT CONTROL INSTRUMENTATION, RECORDS AND REPORTS OF RADIOACTIVE EFFLUENTS, TESTING OF AIR CLEANING SYSTEMS, AND REACTOR COOLANT QUALITY. WITHIN THE TEN AREAS INSPECTED, NO VIOALTIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

CONTRARY TO 10 CFR 50, APPENDIX B, TECHNICAL SPECIFICATION 7.4.A, AND ADMINISTRATIVE PROCEDURE P-6, THE NRC INSPECTOR DETERMINED THAT FOR CHANGE NOTICE/CONTROL WORK PERMIT CN 1295/ CWP82-122 NO MEASURE HAD BEEN ESTABLISHED TO PROVIDE UPDATED WRITTEN PROCEDURES THAT WOULD REFLECT THIS PLANT MODIFICATION APPARENTLY DUE TO THE FAILURE OF THE WORK REVIEW COMMITTEE TO EVALUATE THE EFFECT OF THIS CWP ON OTHER PLANT SYSTEMS, CONDITIONS OR OPERATIONS. CONTRARY TO 10 CFR 50, APPENDIX B, FORT ST. VRAIN FSAR COMMITMENT TO ANSI 45.2.5, AND ASTM C-94, DEVIATION REPORT 82-206-1-H HAS DELETED THE CONCRETE TRANSPORT TIME REQUIREMENT FROM SPECIFICATION 75-J-02 WITHOUT IMPOSITION OF ALTERNATE ACCEPTANCE CRITERIA.

CONTRARY TO TECHNICAL SPECIFICATION 7.4.A AND ADMINISTRATIVE PROCEDURE G-9, THE NRC INSPECTOR DETERMINED THAT CONTROL WORK PERMIT CWP 82-255 HAD BEEN SURRENDERED DUE TO WORK COMPLETION AND THE AFFECTED SYSTEM RETURNED TO SERVICE WITHOUT HAVING A WRITEN

ENFORCEMENT SUMMARY

PROCEDURE TO OPERATE THE MODIFIED SYSTEM, WITHOUT HAVING THE SHIFT SUPERVISOR'S SIGNATURE ON THE CWP VERIFYING WORK COMPLETION, AND WITHOUT HAVING UPDATED CONTROL ROOM/SHIFT SUPERVISOR DESIGN DOCUMENTS. CWP'S 82-48, 82-83, AND 82-173 WERE IDENTIFIED AS ADDITIONAL EXAMPLES OF THE ABOVE VIOLATION.
(8324 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

MAJOR ELECTRICAL MODIFICATIONS TO THE AUXILIARY ELECTRICAL SYSTEM, INSTRUMENT POWER SYSTEM, 480V AC DISTRIBUTION SYSTEM, AND 4160/480V AC TRANSFORMERS ARE SCHEDULED FOR THIS REFUELING OUTAGE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THIRD REFUELING OUTAGE BEGAN JANUARY 19, 1984, AND IS STILL CONTINUING

LAST IE SITE INSPECTION DATE: JANUARY 16-20, 1984

INSPECTION REPORT NO: 50-267/84-03

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-244 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: ROBERT E. DODGE (315) 524-4446
4. Licensed Thermal Power (MWt): 1520
5. Nameplate Rating (Gross MWe): 608 X 0.85 = 517
6. Design Electrical Rating (Net MWe): 470
7. Maximum Dependable Capacity (Gross MWe): 490
8. Maximum Dependable Capacity (Net MWe): 470
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

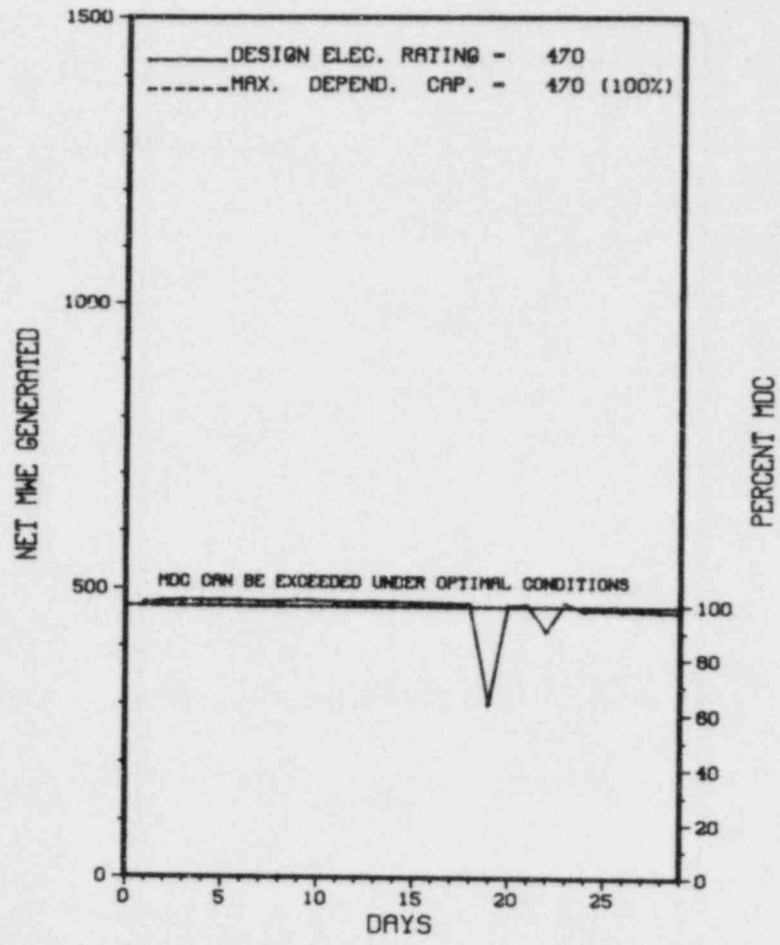
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	125,016.0
13. Hours Reactor Critical	696.0	1,440.0	95,039.4
14. Rx Reserve Shtdwn Hrs	.0	.0	1,631.5
15. Hrs Generator On-Line	696.0	1,440.0	92,951.3
16. Unit Reserve Shtdwn Hrs	.0	.0	8.5
17. Gross Therm Ener (MWH)	1,027,536	2,142,528	128,399,897
18. Gross Elec Ener (MWH)	341,611	712,460	41,876,831
19. Net Elec Ener (MWH)	324,804	677,767	39,704,011
20. Unit Service Factor	100.0	100.0	74.4
21. Unit Avail Factor	100.0	100.0	74.4
22. Unit Cap Factor (MDC Net)	99.3	100.1	69.3*
23. Unit Cap Factor (DER Net)	99.3	100.1	69.3*
24. Unit Forced Outage Rate	.0	.0	7.7
25. Forced Outage Hours	.0	.0	3,802.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING AND MAINTENANCE - 03/03/84 - 65 DAYS.
27. If Currently Shutdown Estimated Startup Date: N/A

****
* GINNA *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

GINNA



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* GINNA *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
	02/18/84	F	0.0	A	5		SF	ACCUMU	POWER REDUCTION DUE TO S.I. ACCUMULATOR LOW PRESSURE.
	02/22/84	F	0.0	A	5		HB	PIPEXX	POWER REDUCTION DUE TO 2B REHEATER STEAM LEAK.
	02/24/84	S	0.0	C	5				COASTDOWN COMMENCED.
	02/24/84	F	0.0	A	5		HH	XXXXXX	POWER REDUCTION DUE TO LOSS OF A AND B HEATER DRAIN TANK PUMPS.

****
* SUMMARY *
****

GINNA OPERATED WITH 4 REDUCTIONS AND NO OUTAGES DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* GINNA *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....WAYNE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...15 MI NE OF
ROCHESTER, NY

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 8, 1969

DATE ELEC ENER 1ST GENER...DECEMBER 2, 1969

DATE COMMERCIAL OPERATE....JULY 1, 1970

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ONTARIO

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ROCHESTER GAS & ELECTRIC

CORPORATE ADDRESS.....89 EAST AVENUE
ROCHESTER, NEW YORK 14604

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....R. ZIMMERMAN

LICENSING PROJ MANAGER.....G. DICK
DOCKET NUMBER.....50-244

LICENSE & DATE ISSUANCE....DPR-18, SEPTEMBER 19, 1969

PUBLIC DOCUMENT ROOM.....ROCHESTER PUBLIC LIBRARY
BUSINESS AND SOCIAL SCIENCE DIVISION
115 SOUTH AVENUE
ROCHESTER, NEW YORK 14604

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-213 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: R.L. EPPINGER (203) 267-2556 X274
4. Licensed Thermal Power (MWt): 1825
5. Nameplate Rating (Gross MWe): 667 X 0.9 = 600
6. Design Electrical Rating (Net MWe): 582
7. Maximum Dependable Capacity (Gross MWe): 596
8. Maximum Dependable Capacity (Net MWe): 569
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	141,696.0
13. Hours Reactor Critical	696.0	1,440.0	122,641.4
14. Rx Reserve Shtdwn Hrs	.0	.0	1,200.5
15. Hrs Generator On-Line	696.0	1,440.0	117,347.3
16. Unit Reserve Shtdwn Hrs	.0	.0	373.7
17. Gross Therm Ener (MWH)	1,264,967	2,618,211	203,990,771
18. Gross Elec Ener (MWH)	421,078	871,510	66,984,753
19. Net Elec Ener (MWH)	402,159	832,195	63,732,896
20. Unit Service Factor	100.0	100.0	82.8
21. Unit Avail Factor	100.0	100.0	83.1
22. Unit Cap Factor (MDC Net)	101.5	101.6	82.7*
23. Unit Cap Factor (DER Net)	99.3	99.3	75.8*
24. Unit Forced Outage Rate	.0	.0	6.1
25. Forced Outage Hours	.0	.0	1,158.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

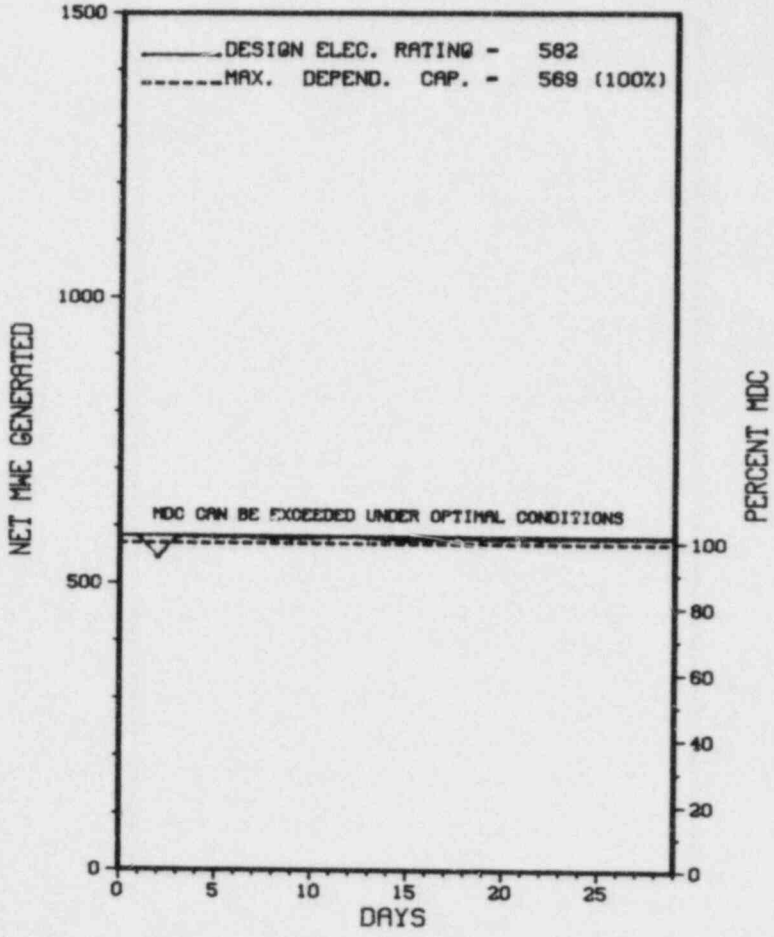
REFUELING: 06/16/84, 10 WEEK DURATION.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* HADDAM NECK *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

HADDAM NECK



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

HADDAM NECK

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

HADDAM NECK (CONNECTICUT YANKEE) OPERATED AT FULL POWER DURING THE REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* HADDAM NECK *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CONNECTICUT

COUNTY.....MIDDLESEX

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...13 MI E OF
MERIDEN, CONN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 24, 1967

DATE ELEC ENER 1ST GENER...AUGUST 7, 1967

DATE COMMERCIAL OPERATE....JANUARY 1, 1968

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CONNECTICUT RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONNECTICUT YANKEE ATOMIC POWER

CORPORATE ADDRESS.....P.O. BOX 270
HARTFORD, CONNECTICUT 06101

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....P. SWETLAND

LICENSING PROJ MANAGER.....J. LYONS
DOCKET NUMBER.....50-213

LICENSE & DATE ISSUANCE....DPR-61, DECEMBER 27, 1974

PUBLIC DOCUMENT ROOM.....RUSSELL LIBRARY
119 BROAD STREET
MIDDLETOWN, CONNECTITCUT 06457

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* HADDAM NECK *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-321 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: D.P. RAFEEDIE (912) 367-7851
4. Licensed Thermal Power (MWt): 2436
5. Nameplate Rating (Gross MWe): 1000 X 0.85 = 850
6. Design Electrical Rating (Net MWe): 777
7. Maximum Dependable Capacity (Gross MWe): 801
8. Maximum Dependable Capacity (Net MWe): 752
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

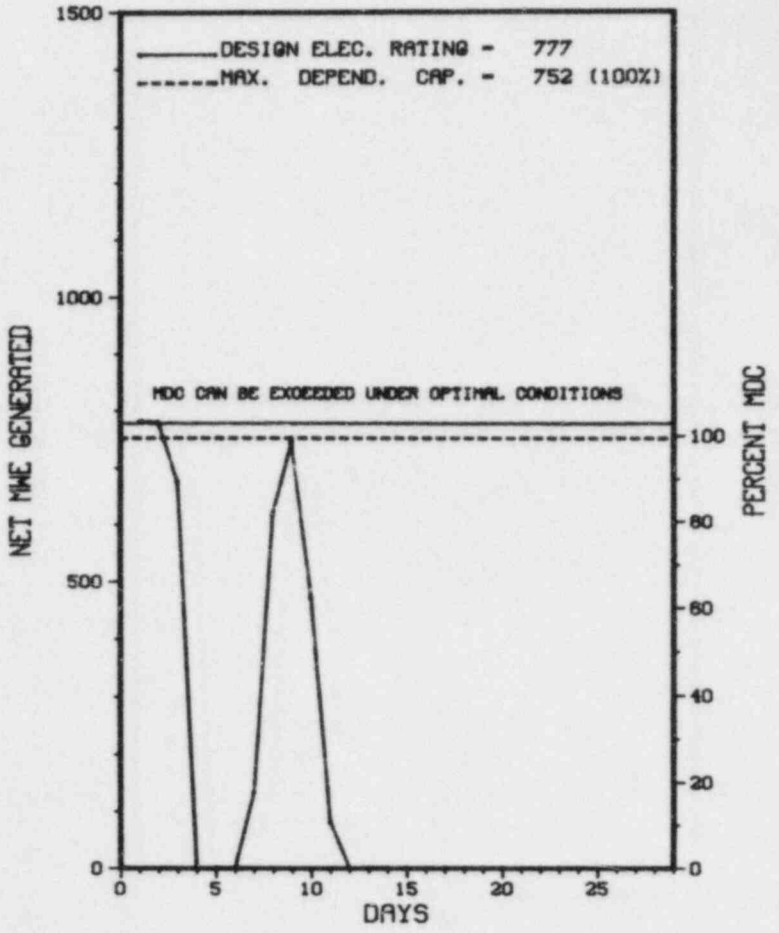
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	71,568.0
13. Hours Reactor Critical	179.5	923.5	50,429.3
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	163.1	907.1	47,300.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	347,208	2,147,184	99,282,299
18. Gross Elec Ener (MWH)	108,350	707,510	32,156,490
19. Net Elec Ener (MWH)	99,187	673,533	30,524,024
20. Unit Service Factor	23.4	63.0	66.1
21. Unit Avail Factor	23.4	63.0	66.1
22. Unit Cap Factor (MDC Net)	19.0	62.2	56.7
23. Unit Cap Factor (DER Net)	18.3	60.2	54.9
24. Unit Forced Outage Rate	73.3	33.1	16.4
25. Forced Outage Hours	448.3	448.3	9,058.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: 03/08/84

****
* HATCH 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

HATCH 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* HATCH 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-5	02/03/84	S	0.0	H	5		SA	VESSEL	RAMPING DOWN TO PERFORM TORUS VENT HEADER INSPECTION.
84-6	02/03/84	S	84.6	H	2		SA	VESSEL	OUTAGE TO INSPECT TORUS VENT HEADER.
84-7	02/07/84	S	0.0	H	5		SA	VESSEL	RAMP TO RATED POWER FROM TORUS INSPECTION OUTAGE.
84-8	02/10/84	F	0.0	A	5		CB	PUMPXX	"B" RECIRC PUMP TRIPPED.
84-9	02/11/84	F	448.3	A	3		HA	TURBIN	REACTOR SCRAM FROM TURBINE HIGH VIBRATION UPON INSPECTION REVEALED 13TH STAGE LP TURBINE BUCKETS DAMAGED.

****
* SUMMARY *
****

HATCH 1 SHUTDOWN ON FEBRUARY 11TH FOR TURBINE REPAIR.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* HATCH 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....GEORGIA

COUNTY.....APPLING

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI N OF
BAXLEY, GA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 12, 1974

DATE ELEC ENER 1ST GENER...NOVEMBER 11, 1974

DATE COMMERCIAL OPERATE....DECEMBER 31, 1975

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....ALTAMAHA RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GEORGIA POWER

CORPORATE ADDRESS.....333 PIEDMONT AVENUE
ATLANTA, GEORGIA 30308

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....GEORGIA POWER CO.

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. CRLENJAK

LICENSING PROJ MANAGER.....G. RIVENBARK
DOCKET NUMBER.....50-321

LICENSE & DATE ISSUANCE....DPR-57, OCTOBER 13, 1974

PUBLIC DOCUMENT ROOM.....APPLING COUNTY PUBLIC LIBRARY
301 CITY HALL DRIVE
BAXLEY, GEORGIA 31563

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JULY 21 - AUGUST 20 (83-25): THIS INSPECTION INVOLVED 84 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, AND SURVEILLANCE ACTIVITIES. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 24-27 (84-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION. OF THE AREAS INSPECTED, 1 VIOLATION WAS FOUND - FAILURE TO FOLLOW PROCEDURE FOR RECORDING LOCATION AND EXTENT OF ISI NDE EXAMINATION OF REACTOR VESSEL WELD.

INSPECTION FEBRUARY 4-7 (84-04): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF TORUS VENT HEADER CRACKING. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ SHUTDOWN - MAIN TURBINE GENERATOR DAMAGED.

LAST IE SITE INSPECTION DATE: FEBRUARY 4-7, 1984 +

INSPECTION REPORT NO: 50-321/84-04 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-366 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: D.P. RAFEEDIE (912) 367-7851
4. Licensed Thermal Power (MWt): 2436
5. Nameplate Rating (Gross MWe): 1000 X 0.85 = 850
6. Design Electrical Rating (Net MWe): 784
7. Maximum Dependable Capacity (Gross MWe): 806
8. Maximum Dependable Capacity (Net MWe): 748
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	39,337.0
13. Hours Reactor Critical	.0	308.2	27,547.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	308.2	26,241.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	726,912	56,293,208
18. Gross Elec Ener (MWH)	0	242,640	18,547,990
19. Net Elec Ener (MWH)	0	230,541	17,648,783
20. Unit Service Factor	.0	21.4	66.7
21. Unit Avail Factor	.0	21.4	66.7
22. Unit Cap Factor (MDC Net)	.0	21.4	60.0
23. Unit Cap Factor (DER Net)	.0	20.4	57.2
24. Unit Forced Outage Rate	.0	.0	11.5
25. Forced Outage Hours	.0	.0	3,425.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

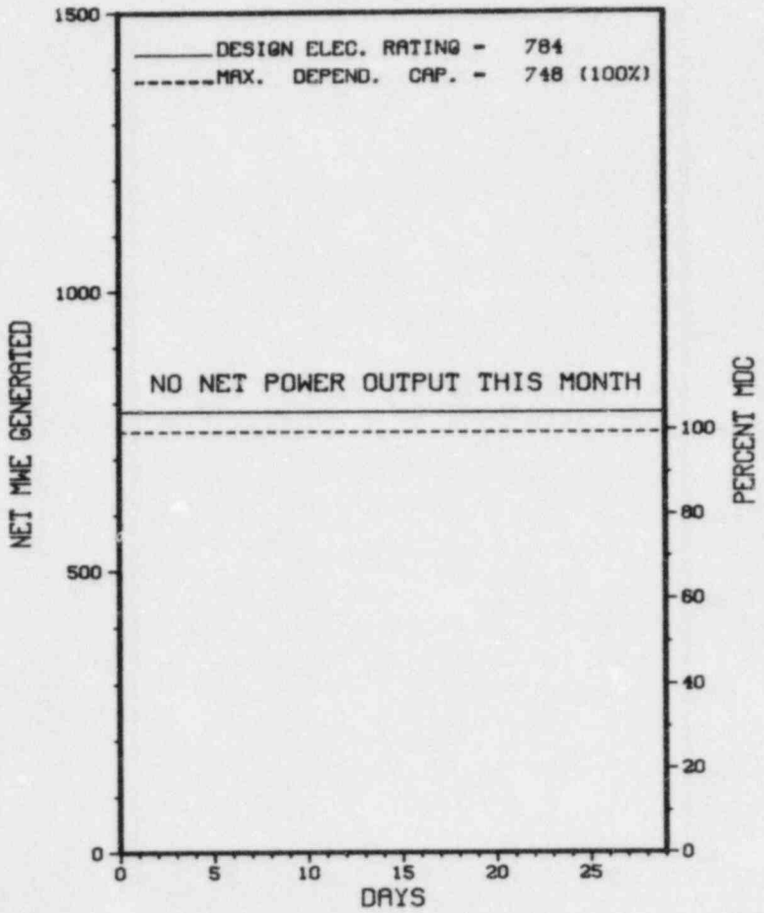
NONE

27. If Currently Shutdown Estimated Startup Date: 09/10/84

****
* HATCH 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

HATCH 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

HATCH 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-5	01/13/84	S	696.0	H	4		CB	PIPEXX	RECIRC PIPE REPLACEMENT OUTAGE.

SUMMARY

HATCH 2 REMAINS SHUTDOWN IN A CONTINUING REPAIR STAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* HATCH 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....GEORGIA

COUNTY.....APPLING

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI N OF
BAXLEY, GA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JULY 4, 1978

DATE ELEC ENER 1ST GENER...SEPTEMBER 22, 1978

DATE COMMERCIAL OPERATE....SEPTEMBER 5, 1979

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....ALTAMAHA RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GEORGIA POWER

CORPORATE ADDRESS.....333 PIEDMONT AVENUE
ATLANTA, GEORGIA 30308

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....GEORGIA POWER CO.

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. CRLENJAK

LICENSING PROJ MANAGER.....G. RIVENBARK
DOCKET NUMBER.....50-366

LICENSE & DATE ISSUANCE....NPF-5, JUNE 13, 1978

PUBLIC DOCUMENT ROOM.....APPLING COUNTY PUBLIC LIBRARY
301 CITY HALL DRIVE
BAXLEY, GEORGIA 31563

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JULY 21 - AUGUST 20 (83-26): THIS INSPECTION INVOLVED 84 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, AND SURVEILLANCE ACTIVITIES. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 24-27 (84-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION AND RECIRCULATION SYSTEM PIPING REPLACEMENT. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 4-7 (84-04): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF TORUS VENT HEADER CRACKING, FEEDWATER LINE ULTRASONIC (UT) INDICATION AND RECIRC PIPING REPLACEMENT. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

CRITE..ION XIV OF APPENDIX B TO 10 CFR PART 50 AS IMPLEMENTED BY HATCH UNIT 2 FSAR CHAPTER 17.2.14 REQUIRES IN PART THAT MEASURES SHALL BE ESTABLISHED FOR INDICATING THE OPERATING STATUS OF STRUCTURES, SYSTEMS AND COMPONENTS TO PREVENT INADVERTENT OPERATION.

****
* HATCH 2 *
****

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.8.1.A REQUIRES THAT WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX "A" OF REGULATORY GUIDE 1.33 REVISION 2, FEBRUARY 1978. CONTRARY TO THE ABOVE, DURING THE PERIOD FROM JULY 12-25, 1983, AFTER UNIT 2 HAD RETURNED TO POWER FOLLOWING AN EXTENDED REFUELING OUTAGE, IT WAS FOUND THAT SAFETY RELATED EQUIPMENT AND SYSTEMS WERE NOT ALIGNED AS REQUIRED BY APPROVED OPERATING OR SURVEILLANCE PROCEDURES TO ASSURE SAFE OPERATING UNDER ALL CONDITIONS. SPECIFICALLY, THROUGH INADEQUACIES IN THE ADMINISTRATIVE AND MANAGERIAL CONTROL SYSTEMS, PREREQUISITES FOR STARTUP DID NOT CONFIRM THAT NECESSARY INSTRUMENTS WERE OPERABLE, VALVES PROPERLY ALIGNED, AND ELECTRICAL LINKS TO BE IN THE CORRECT POSITIONS. EXAMPLES WHERE THIS LACK OF PROPER CONFIRMATION EXISTED WERE: (A) THE ANTICIPATED TRANSIENT WITHOUT SCRAM (ATWS) TRIP WAS FOUND TO HAVE A VALVE SHUT WHICH ISOLATED THE REDUNDANT SIGNAL TO EACH TRIP SYSTEM. (B) JET PUMPS NUMBERS 18 AND 19 WERE FOUND TO HAVE FLOW INSTRUMENTATION ISOLATED. (C) THE STANDBY LIQUID CONTROL SYSTEM WAS FOUND TO HAVE THE PUMP DISCHARGE PRESSURE INSTRUMENTATION ISOLATED. (D) THE HIGH PRESSURE INJECTION SYSTEM WAS FOUND TO HAVE THE AUTOMATIC SHUT FEATURE OF THE MINIMUM FLOW VALVE ELECTRICALLY ISOLATED. (E) THE "B" HYDROGEN RECOMBINER, WAS FOUND TO HAVE THE "B" RECOMBINER DISCHARGE VALVE IMPROPERLY SHUT. TECHNICAL SPECIFICATION 3.0.4 REQUIRES THAT ENTRY INTO AN OPERATIONAL CONDITION SHALL NOT BE MADE UNLESS THE CONDITIONS OF THE LIMITING CONDITION FOR OPERATIONS ARE MET WITHOUT RELIANCE ON PROVISIONS CONTAINED IN THE ACTION STATEMENTS UNLESS OTHERWISE EXCEPTED. TECHNICAL SPECIFICATION 3.6.1.4 REQUIRES THAT TWO MAIN STEAM ISOLATION LEAKAGE CONTROL SYSTEM SUBSYSTEMS SHALL BE OPERABLE IN OPERATIONAL CONDITIONS 1, 2, AND 3. CONTRARY TO THE ABOVE, ON JULY 12, 1983, THE UNIT ENTERED OPERATIONAL CONDITION 1, POWER OPERATION, WITHOUT MEETING THE LIMITING CONDITION FOR OPERATION FOR THE MAIN STEAM ISOLATION LEAKAGE CONTROL SYSTEM SUBSYSTEM IN THAT BOTH SUBSYSTEMS WERE INOPERABLE.
(8326 4)

10 CFR 50, APPENDIX B, CRITERION V, AS IMPLEMENTED IN HATCH UNIT 2 FSAR SECTION 17.2.5 REQUIRES INPART THAT ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCCES. CONTRARY TO THE ABOVE, ON JUNE 6, 1983 THROUGH JUNE 18, 1983, WORK ON THE RESIDUAL HEAT REMOVAL SYSTEM TESTABLE CHECK VALVE, 2E11-F050B, WAS PERFORMED WITHOUT DOCUMENTED INSTRUCTIONS, PROCEDURES OR DRAWINGS APPROPRIATE TO THE CIRCUMSTANCES IN THAT THE MAINTENANCE REQUEST PROCEDURE, MR-8, AS UTILIZED UNDER MAINTENANCE REQUEST NUMBER 2-83-3188 DID NOT PREVENT THE REVERSAL OF THE AIR ACTUATOR AIRLINES.

(8338 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

RECIRCULATION PIPE REPLACEMENT IN PROGRESS.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN.

LAST IE SITE INSPECTION DATE: FEBRUARY 4-7, 1984 +

INSPECTION REPORT NO: 50-366/84-04 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-247 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: E. EICH (914) 694-6000 @ I.P.
4. Licensed Thermal Power (MWt): 2758
5. Nameplate Rating (Gross MWe): 1126 X 0.9 = 1013
6. Design Electrical Rating (Net MWe): 873
7. Maximum Dependable Capacity (Gross MWe): 900
8. Maximum Dependable Capacity (Net MWe): 864
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	84,745.0
13. Hours Reactor Critical	305.6	997.4	56,945.0
14. Rx Reserve Shtdwn Hrs	.0	.0	2,119.1
15. Hrs Generator On-Line	293.8	973.5	55,169.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	772,960	2,599,755	143,640,254
18. Gross Elec Ener (MWH)	238,730	813,190	44,470,766
19. Net Elec Ener (MWH)	224,033	775,379	42,402,471
20. Unit Service Factor	42.2	67.6	65.1
21. Unit Avail Factor	42.2	67.6	65.1
22. Unit Cap Factor (MDC Net)	37.3	62.3	58.2*
23. Unit Cap Factor (DER Net)	36.9	61.7	57.3
24. Unit Forced Outage Rate	57.8	32.4	9.9
25. Forced Outage Hours	402.2	466.5	5,842.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

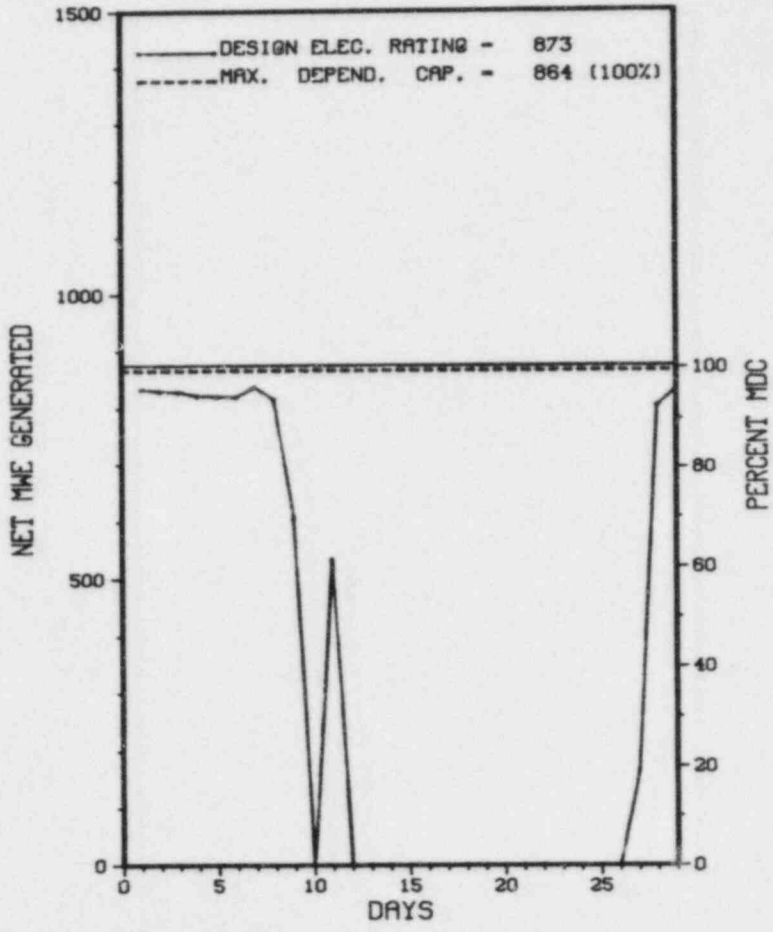
REFUELING/INSPECTION - 06/03/84.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* INDIAN POINT 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

INDIAN POINT 2



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

INDIAN POINT 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2	02/09/84	F	27.7	D	1	84-003	IB	INSTRU	CONTROL ROD POSITION INDICATOR REPAIR.
3	02/11/84	F	374.5	A	1		CC	HTEXCH	STEAM GENERATOR TUBE LEAK.

SUMMARY

INDIAN POINT 2 OPERATED WITH 2 OUTAGES DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* INDIAN POINT 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....WESTCHESTER

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI N OF
NEW YORK CITY, NY

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 22, 1973

DATE ELEC ENER 1ST GENER...JUNE 26, 1973

DATE COMMERCIAL OPERATE....AUGUST 1, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....HUDSON RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONSOLIDATED EDISON

CORPORATE ADDRESS.....4 IRVING PLACE
NEW YORK, NEW YORK 10003

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....WESTINGHOUSE DEVELOPMENT CORP

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....P. KOLTAY

LICENSING PROJ MANAGER.....R. PEDERSEN
DOCKET NUMBER.....50-247

LICENSE & DATE ISSUANCE....DPR-26, SEPTEMBER 28, 1973

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100 MARTINE AVENUE
WHITE PLAINS, NEW YORK 10601

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-286 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: L. KELLY (914) 739-8200
4. Licensed Thermal Power (MWt): 3025
5. Nameplate Rating (Gross MWe): 1126 X 0.9 = 1013
6. Design Electrical Rating (Net MWe): 965
7. Maximum Dependable Capacity (Gross MWe): 1000
8. Maximum Dependable Capacity (Net MWe): 965
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	65,761.0
13. Hours Reactor Critical	691.3	753.1	35,177.6
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	667.6	667.9	33,810.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,542,818	1,543,241	85,913,077
18. Gross Elec Ener (MWH)	482,660	482,665	26,849,276
19. Net Elec Ener (MWH)	460,451	460,451	25,704,629
20. Unit Service Factor	95.9	46.4	51.4
21. Unit Avail Factor	95.9	46.4	51.4
22. Unit Cap Factor (MDC Net)	68.6	33.1	40.5
23. Unit Cap Factor (DER Net)	68.6	33.1	40.5
24. Unit Forced Outage Rate	3.2	53.4	24.5
25. Forced Outage Hours	22.3	766.0	10,962.8

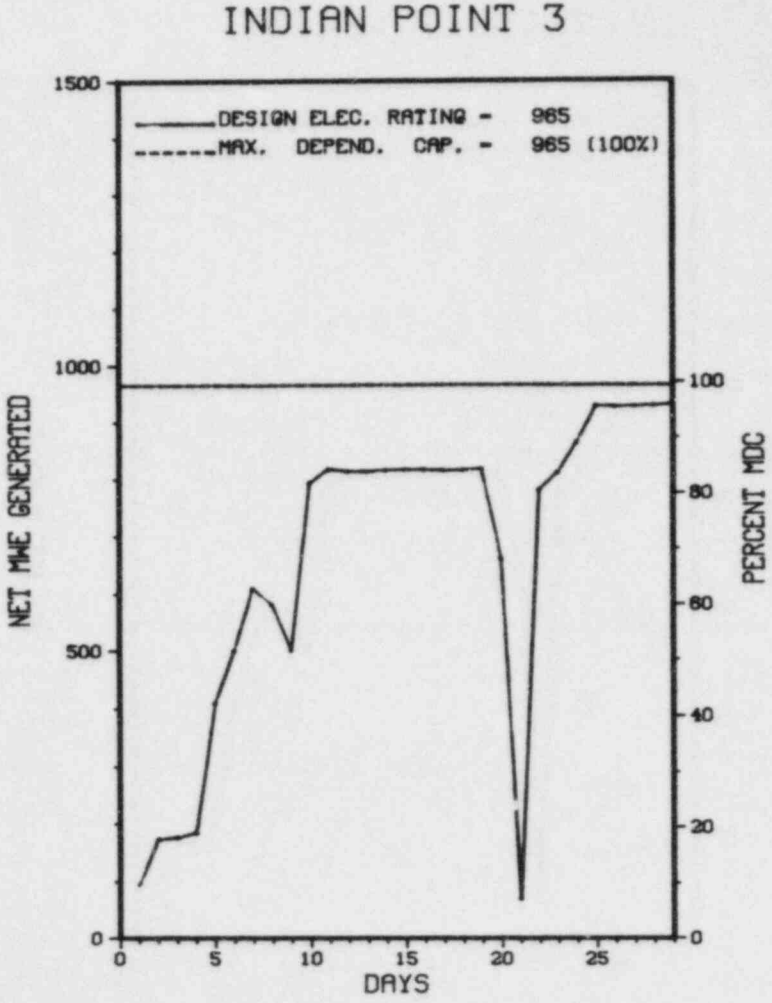
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* INDIAN POINT 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* INDIAN POINT 3 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
01	02/01/84	S	6.1	B	9		ZZ	ZZZZZZ	UNIT REMOVED FROM SERVICE FOR TURBINE OVERSPEED TRIP TEST.
02	02/09/84	F	4.6	A	3		HH	INSTRU	REACTOR TRIP CAUSED BY TRIP OF #31 & #32 HEATER DRAIN TANK PUMPS.
03	02/20/84	F	7.0	A	3		HH	VALVOP	#32 STEAM GENERATOR LOW LEVEL MISMATCH CAUSED BY A FAILED FEEDWATER REGULATING VALVE SOLENOID.
04	02/21/84	F	10.7	B	9		ZZ	ZZZZZZ	REPAIRS TO DRAIN LINE ON FEEDWATER DISCHARGE HEADER.

****
* SUMMARY *
****

INDIAN POINT 3 OPERATED ROUTINELY DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* INDIAN POINT 3 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....WESTCHESTER

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI N OF
NEW YORK CITY, NY

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 6, 1976

DATE ELEC ENER 1ST GENER...APRIL 27, 1976

DATE COMMERCIAL OPERATE....AUGUST 30, 1976

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....HUDSON RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....POWER AUTHORITY OF STATE OF N.Y.

CORPORATE ADDRESS.....10 COLUMBUS CIRCLE
NEW YORK, NEW YORK 10019

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....WESTINGHOUSE DEVELOPMENT CORP

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. KENNY

LICENSING PROJ MANAGER.....P. POLK
DOCKET NUMBER.....50-286

LICENSE & DATE ISSUANCE....DPR-64, APRIL 5, 1976

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WHITE PLAINS, NEW YORK 10601

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-305 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: G.RUITER (414) 388-2560 X207
4. Licensed Thermal Power (MWt): 1650
5. Nameplate Rating (Gross MWe): 622 X 0.9 = 560
6. Design Electrical Rating (Net MWe): 535
7. Maximum Dependable Capacity (Gross MWe): 529
8. Maximum Dependable Capacity (Net MWe): 503
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	85,105.0
13. Hours Reactor Critical	696.0	1,440.0	72,620.1
14. Rx Reserve Shtdwn Hrs	.0	.0	2,330.5
15. Hrs Generator On-Line	696.0	1,440.0	71,252.5
16. Unit Reserve Shtdwn Hrs	.0	.0	10.0
17. Gross Therm Ener (MWH)	1,139,329	2,365,226	111,336,312
18. Gross Elec Ener (MWH)	374,200	777,200	36,635,300
19. Net Elec Ener (MWH)	357,376	742,369	34,874,405
20. Unit Service Factor	100.0	100.0	83.7
21. Unit Avail Factor	100.0	100.0	83.7
22. Unit Cap Factor (MDC Net)	102.1	102.5	78.7*
23. Unit Cap Factor (DER Net)	96.0	96.4	76.6
24. Unit Forced Outage Rate	.0	.0	3.8
25. Forced Outage Hours	.0	.0	2,729.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

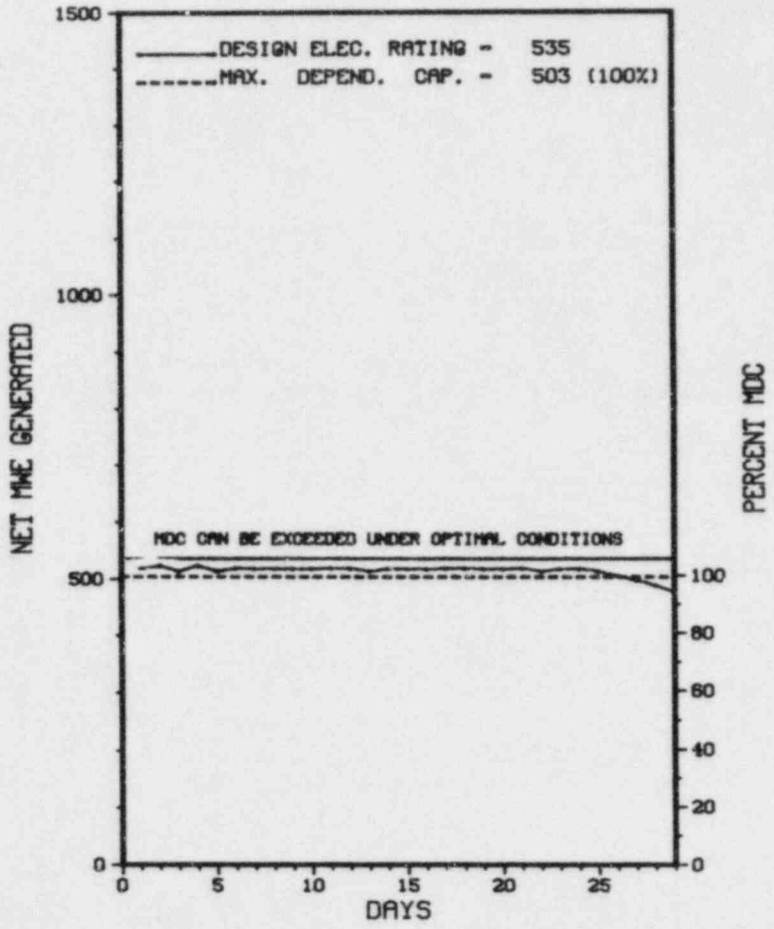
REFUELING - 03/16/84 - 6 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* KEWAUNEE *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

KEWAUNEE



FEBRUARY 1984

* Item calculated with a Waighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

KEWAUNEE

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

KEWAUNEE OPERATED AT FULL POWER WITH NO OUTAGES OR REDUCTIONS DURING THE REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* KEWAUNEE *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....KEWAUNEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...27 MI E OF
GREEN BAY, WI.

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MARCH 7, 1974

DATE ELEC ENER 1ST GENER...APRIL 8, 1974

DATE COMMERCIAL OPERATE....JUNE 16, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....WISCONSIN PUBLIC SERVICE

CORPORATE ADDRESS.....P.O. BOX 1200
GREEN BAY, WISCONSIN 54305

CONTRACTOR
ARCHITECT/ENGINEER.....PIONEER SERVICES & ENGINEERING

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....PIONEER SERVICES & ENGINEERING

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....R. NELSON

LICENSING PROJ MANAGER.....M. GROTENHUIS
DOCKET NUMBER.....50-305

LICENSE & DATE ISSUANCE....DPR-43, DECEMBER 21, 1973

PUBLIC DOCUMENT ROOM.....KEWAUNEE PUBLIC LIBRARY
822 JUNEAU STREET
KEWAUNEE, WISCONSIN 54216

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON DECEMBER 19-23, JANUARY 3-6, 9-13, 30, 31, FEBRUARY 6-10, 13-15, (83-17): ROUTINE UNANNOUNCED INSPECTION BY RESIDENT INSPECTOR OF OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; INDEPENDENT INSPECTION; AND REGIONAL REQUESTS. THE INSPECTION INVOLVED 101 INSPECTOR-HOURS BY ONE INSPECTOR INCLUDING 18 INSPECTION-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: DECEMBER 16 THROUGH FEBRUARY 15, 1984

INSPECTION REPORT NO: 83-17

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-409 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: G. R. GADOW (608) 689-2331

4. Licensed Thermal Power (MWt): 165

5. Nameplate Rating (Gross MWe): 76.8 X 0.85 = 65

6. Design Electrical Rating (Net MWe): 50

7. Maximum Dependable Capacity (Gross MWe): 50

8. Maximum Dependable Capacity (Net MWe): 48

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	125,619.0
13. Hours Reactor Critical	696.0	1,315.7	82,060.1
14. Rx Reserve Shtdwn Hrs	.0	.0	478.0
15. Hrs Generator On-Line	696.0	1,211.7	76,048.0
16. Unit Reserve Shtdwn Hrs	.0	.0	79.0
17. Gross Therm Ener (MWH)	113,026	180,283	10,462,587
18. Gross Elec Ener (MWH)	36,817	57,835	3,115,063
19. Net Elec Ener (MWH)	35,051	54,638	2,881,873
20. Unit Service Factor	100.0	84.1	60.5
21. Unit Avail Factor	100.0	84.1	60.6
22. Unit Cap Factor (MDC Net)	104.9	79.0	47.8
23. Unit Cap Factor (DER Net)	100.7	75.9	45.9
24. Unit Forced Outage Rate	.0	1.8	9.4
25. Forced Outage Hours	.0	22.1	6,865.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

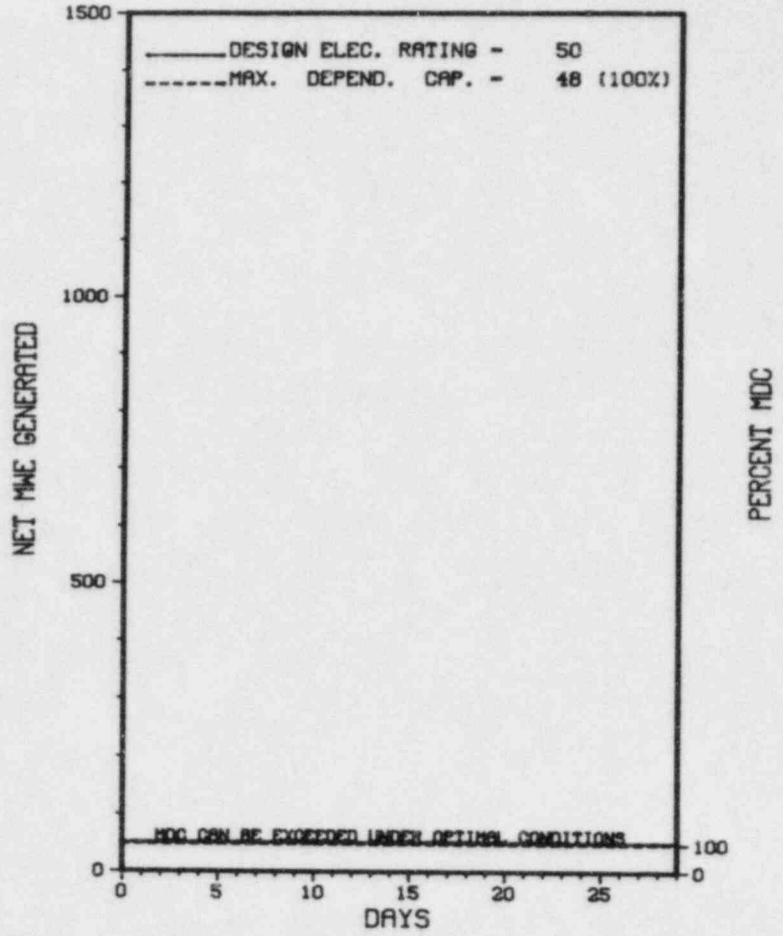
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* LA CROSSE *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

LA CROSSE



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* LA CROSSE *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

****
* SUMMARY *
****

LACROSSE OPERATED AT FULL POWER WITH NO OUTAGES OR REDUCTIONS DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* LA CROSSE *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....VERNON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI S OF
LACROSSE, WISC

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JULY 11, 1967

DATE ELEC ENER 1ST GENER...APRIL 26, 1968

DATE COMMERCIAL OPERATE....NOVEMBER 1, 1969

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DAIRYLAND POWER

CORPORATE ADDRESS.....2615 EAST AVENUE SOUTH
LACROSSE, WISCONSIN 54601

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...ALLIS-CHALMERS

CONSTRUCTOR.....MAXON CONSTRUCTION COMPANY

TURBINE SUPPLIER.....ALLIS-CHALMERS

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. WIEBE

LICENSING PROJ MANAGER.....R. DUDLEY
DOCKET NUMBER.....50-409

LICENSE & DATE ISSUANCE....DPR-45, AUGUST 28, 1973

PUBLIC DOCUMENT ROOM.....LA CROSSE PUBLIC LIBRARY
800 MAIN STREET
LA CROSSE, WISCONSIN 54601

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON NOVEMBER 18 - JANUARY 18, (83-22): ROUTINE, UNANNOUNCED INSPECTION OF: FOLLOWUP ON PREVIOUS INSPECTION FINDINGS; LICENSEE EVENT REPORT FOLLOW-UP; OPERATIONAL SAFETY VERIFICATION; SYSTEMATIC EVALUATION PROGRAM ACTION ITEMS; TMI TASK ACTION PLAN ITEM FOLLOW-UP; REFUELING ACTIVITIES; REACTOR STARTUP; AND INDEPENDENT INSPECTION. THE INSPECTION INVOLVED 92 INSPECTOR-HOURS ONSITE BY FIVE INSPECTORS INCLUDING 9 INSPECTOR-HOURS ONSITE DURING OFFSHIFTS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SEVEN AREAS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE AREA OF LICENSEE EVENT REPORT FOLLOW-UP (FAILURE TO CALIBRATE INSTRUMENTATION).

INSPECTION ON JANUARY 9-13, (84-01): INCLUDED A REVIEW OF SECURITY ORGANIZATION (MANAGEMENT/PERSONNEL/RESPONSE); SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS (VITAL AND PROTECTED); SECURITY SYSTEM POWER SUPPLY; ASSESSMENT AIDS; ACCESS CONTROL (PERSONNEL/PACKAGES/VEHICLES); DETECTION AIDS (PROTECTED/VITAL); ALARM STATIONS; AND COMMUNICATIONS. THE INSPECTION INVOLVED 37 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. THE INSPECTION BEGAN DURING THE DAY-SHIFT; 3 OF THE INSPECTION HOURS WERE ACCOMPLISHED DURING THE OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THIS INSPECTION.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: JANUARY 9-13, 1984

INSPECTION REPORT NO: 84-01

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-373 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: DIANA L. LIN (815) 357-6761 X481
4. Licensed Thermal Power (MWt): 3323
5. Nameplate Rating (Gross MWe): 1078
6. Design Electrical Rating (Net MWe): 1078
7. Maximum Dependable Capacity (Gross MWe): 1078
8. Maximum Dependable Capacity (Net MWe): 1078
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	1,440.0
13. Hours Reactor Critical	273.7	817.7	817.7
14. Rx Reserve Shtdwn Hrs	407.3	589.4	589.4
15. Hrs Generator On-Line	267.4	734.9	734.9
16. Unit Reserve Shtdwn Hrs	.0	1.0	1.0
17. Gross Therm Ener (MWH)	7,040,020	8,139,690	8,139,690
18. Gross Elec Ener (MWH)	229,288	576,910	576,910
19. Net Elec Ener (MWH)	211,758	539,632	539,632
20. Unit Service Factor	38.4	51.0	51.0
21. Unit Avail Factor	38.4	51.1	51.1
22. Unit Cap Factor (MDC Net)	28.2	34.8	34.8
23. Unit Cap Factor (DER Net)	28.2	34.8	34.8
24. Unit Forced Outage Rate	61.6	44.9	44.9
25. Forced Outage Hours	428.6	597.8	597.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

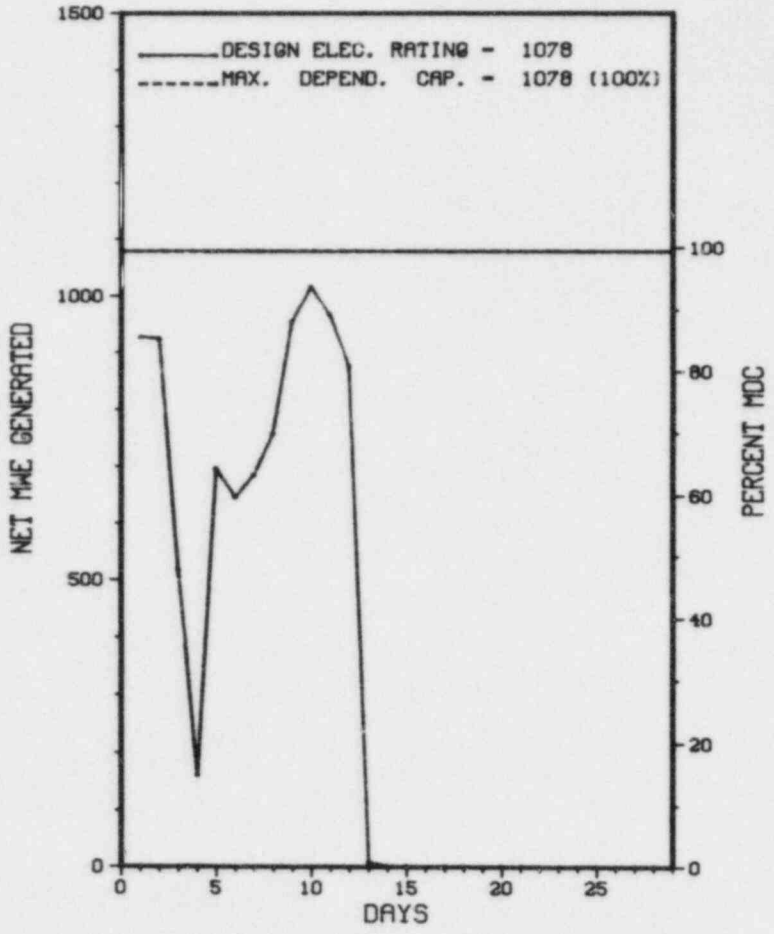
NONE

27. If Currently Shutdown Estimated Startup Date: 03/06/84

****
* LASALLE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

LASALLE 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* LASALLE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
5	02/03/84	F	21.4	B	3				RX SCRAM DUE TO AN INADVERTENT INITIATION OF RCIC DURING PERFORMANCE OF LIS-NB-04.
6	02/13/84	F	407.2	A	3				RX SCRAM DUE TO "A" CONDENSER BOOT SEAL WHICH FAILED AND SUBSEQUENT LOSS OF CONDENSER VACUUM.

****
* SUMMARY *
****

LASALLE 1 OPERATED WITH 2 OUTAGES, SHUTTING DOWN ON FEBRUARY 13TH FOR EQUIPMENT REPAIR.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* LASALLE 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....LA SALLE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI SE OF
OTTAWA, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JUNE 21, 1982

DATE ELEC ENER 1ST GENER...SEPTEMBER 4, 1982

DATE COMMERCIAL OPERATE....JANUARY 1, 1984

CONDENSER COOLING METHOD...POND

CONDENSER COOLING WATER....RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....W. GULDEMOND

LICENSING PROJ MANAGER.....A. BOURNIA
DOCKET NUMBER.....50-373

LICENSE & DATE ISSUANCE....NPF-11, AUGUST 13, 1982

PUBLIC DOCUMENT ROOM.....ILLINOIS VALLEY COMMUNITY COLLEGE
RURAL ROUTE NO. 1
OGLESBY, ILLINOIS 16348

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON DECEMBER 17, THROUGH JANUARY 13, (83-53): ROUTINE, UNANNOUNCED INSPECTION BY RESIDENT INSPECTORS OF LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS; OPERATIONAL SAFETY; LICENSEE EVENT REPORTS; UNIT 2 LICENSE CONDITIONS; PERIODIC AND SPECIAL REPORTS; MAINTENANCE; UNIT 2 FUEL LOADING; REGIONAL REQUESTS; ONSITE FOLLOWUP OF SIGNIFICANT EVENTS; AND INDEPENDENT INSPECTION. THE INSPECTION INVOLVED A TOTAL OF 264 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 32 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 10 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 8 AREAS; TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ONE AREA (FAILURE TO PERFORM A REQUIRED SURVEILLANCE; FAILURE TO FOLLOW PROCEDURES) AND ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE REMAINING AREA (FAILURE TO FOLLOW PROCEDURES).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

OTHER ITEMS

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY FOLLOWING REPAIRS TO CONDENSER.

LAST IE SITE INSPECTION DATE: JANUARY 31 THROUGH FEBRUARY 3, 1984

INSPECTION REPORT NO: 84-04

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-01	01/23/84	01/25/84	RADWASTE DISCHARGE WITH NO SAMPLE FLOW.
84-02	01/06/84	02/03/84	LOSS OF FEEDWATER CAUSING A LOW REACTOR WATER LEVEL SCRAM.
84-03	01/13/84	02/06/84	REACTOR SCRAM - MAIN GENERATOR TRIP.
84-04	01/18/84	02/10/84	SECONDARY CONTAINMENT ISOLATION.
84-05	01/16/84	02/15/84	RX SCRAM/LOSS OF MAIN CONDENSER.

1. Docket: 50-309 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: S. BIEMILLER (617) 827-8100
4. Licensed Thermal Power (MWt): 2630
5. Nameplate Rating (Gross MWe): 864
6. Design Electrical Rating (Net MWe): 825
7. Maximum Dependable Capacity (Gross MWe): 850
8. Maximum Dependable Capacity (Net MWe): 810
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	99,132.6
13. Hours Reactor Critical	696.0	1,413.4	80,024.9
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,398.0	77,477.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,785,157	3,547,360	172,659,148
18. Gross Elec Ener (MWH)	581,520	1,157,480	56,510,630
19. Net Elec Ener (MWH)	562,062	1,118,625	53,820,327
20. Unit Service Factor	100.0	97.1	78.2
21. Unit Avail Factor	100.0	97.1	78.2
22. Unit Cap Factor (MDC Net)	99.7	95.9	69.2*
23. Unit Cap Factor (DER Net)	97.9	94.2	67.2*
24. Unit Forced Outage Rate	.0	2.9	7.6
25. Forced Outage Hours	.0	42.0	5,455.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

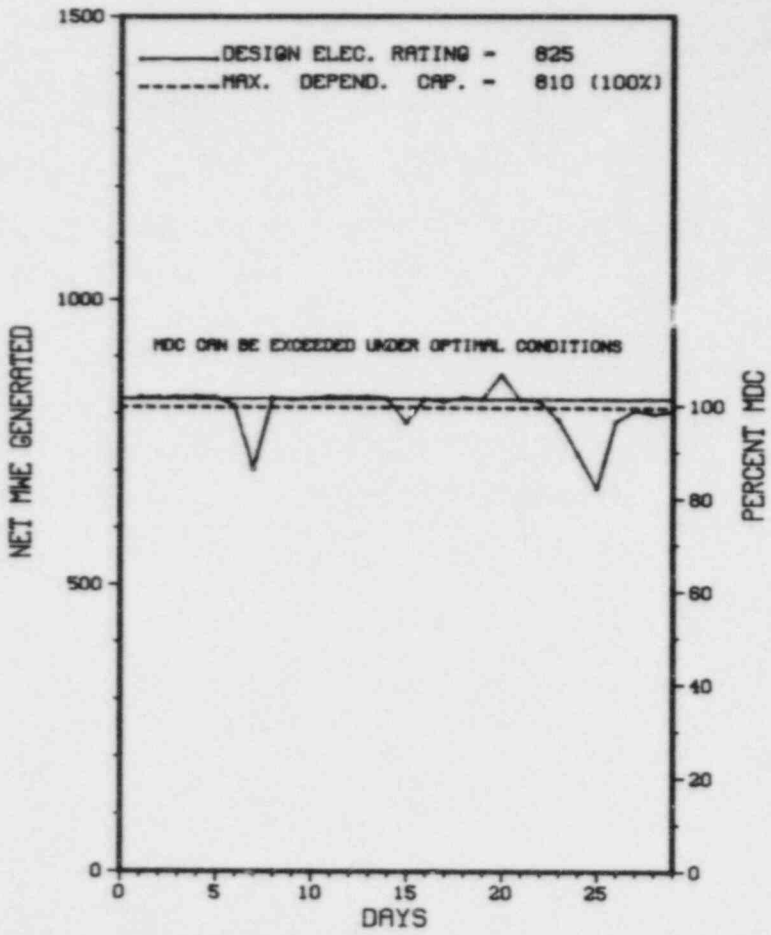
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MAINE YANKEE *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MAINE YANKEE



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

MAINE YANKEE

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
	02/06/84	F	0.0	B	5		HH	HTEXCH	REDUCED POWER DUE TO CHLORIDE IN-LEAKAGE IN "D" WATERBOX AND TURBINE VALVE AND EXCESS FLOW CHECK VALVE TESTING.
	02/15/84	F	0.0	A	5		RB	CRDRVE	REDUCED POWER DUE TO DROPPED CEA #47.
	02/24/84	F	0.0	B	5		HH	HTEXCH	REDUCED POWER DUE TO CHLORIDE IN-LEAKAGE IN ALL WATERBOXES AND TURBINE VALVE AND EXCESS FLOW CHECK VALVE TESTING.

SUMMARY

MAINE YANKEE OPERATED WITH 3 REDUCTIONS DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report (LER) File (NUREG-0161)
			9-Other	

****
MAINE YANKEE
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MAINE

COUNTY.....LINCOLN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI N OF
BATH, ME

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 23, 1972

DATE ELEC ENER 1ST GENER...NOVEMBER 8, 1972

DATE COMMERCIAL OPERATE....DECEMBER 28, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....BACK RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....MAINE YANKEE ATOMIC POWER

CORPORATE ADDRESS.....83 EDISON DRIVE
AUGUSTA, MAINE 04366

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....C. HOLDEN

LICENSING PROJ MANAGER.....K. HEITNER
DOCKET NUMBER.....50-309

LICENSE & DATE ISSUANCE....DPR-36, JUNE 29, 1973

PUBLIC DOCUMENT ROOM.....WISCASSET PUBLIC LIBRARY
HIGH STREET
WISCASSET, MAINE 04578

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-369 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: J. A. REAVIS (704) 373-8552
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1305
6. Design Electrical Rating (Net MWe): 1180
7. Maximum Dependable Capacity (Gross MWe): 1181
8. Maximum Dependable Capacity (Net MWe): 1180
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	19,704.0
13. Hours Reactor Critical	566.5	1,295.1	13,823.4
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	566.2	1,289.4	13,238.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,889,758	4,112,690	31,549,759
18. Gross Elec Ener (MWH)	641,299	1,442,257	10,959,381
19. Net Elec Ener (MWH)	615,331	1,386,041	10,342,296
20. Unit Service Factor	81.4	89.5	67.2
21. Unit Avail Factor	81.4	89.5	67.2
22. Unit Cap Factor (MDC Net)	74.9	81.6	44.5
23. Unit Cap Factor (DER Net)	74.9	81.6	44.5
24. Unit Forced Outage Rate	.0	1.6	19.0
25. Forced Outage Hours	.0	20.8	3,106.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

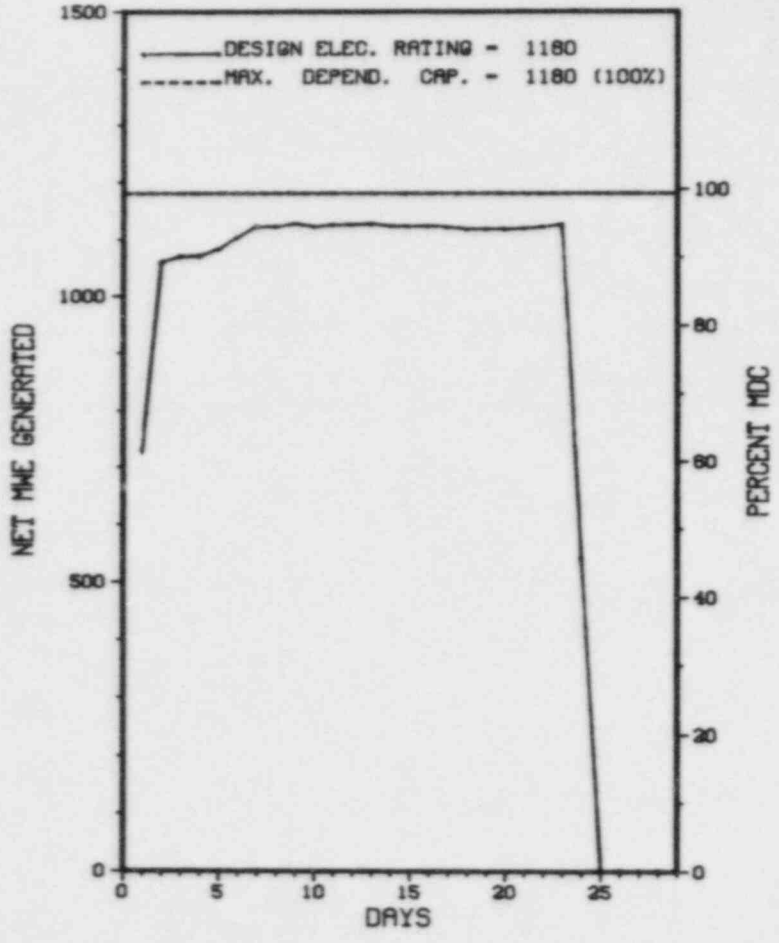
CURRENTLY REFUELING.

27. If Currently Shutdown Estimated Startup Date: 04/24/84

****
* MCGUIRE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MCGUIRE 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* MCGUIRE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
4-P	02/01/84	F	0.0	D	5		ZZ	ZZZZZZ	AXIAL FLUX DIFF PENALTY TIME.
5-P	02/01/84	F	0.0	A	5		CC	VALEX	#4 GOVERNOR VALVE ISOLATED.
3	02/24/84	S	129.8	C	1		RC	FUELXX	END OF CYCLE 1. REFUELING OUTAGE BEGINS.

****
* SUMMARY *
****

MCGUIRE 1 OPERATED WITH 2 REDUCTIONS. SHUTTING DOWN ON FEBRUARY 24TH FOR REFUELING.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
		9-Other	(LER) File (NUREG-0161)

****
* MCGUIRE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....MECKLENBURG

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI N OF
CHARLOTTE, NC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 8, 1981

DATE ELEC ENER 1ST GENER...SEPTEMBER 12, 1981

DATE COMMERCIAL OPERATE....DECEMBER 1, 1981

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE NORMAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE POWER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. ORDERS

LICENSING PROJ MANAGER.....R. BIRKEL
DOCKET NUMBER.....50-369

LICENSE & DATE ISSUANCE....NPF-9, JULY 8, 1981

PUBLIC DOCUMENT ROOM.....MS. DAWN HUBBS
ATKINS LIBRARY
UNIVERSITY OF NORTH CAROLINA - CHARLOTTE
UNCC STATION,
CHARLOTTE, NC 28223

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 20 - DECEMBER 20 (83-47): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 38 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY VERIFICATION, SURVEILLANCE TESTING, AND MAINTENANCE ACTIVITIES. OF THE THREE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (VIOLATION - FAILURE TO USE A PROCEDURE RESULTING IN LOSS OF CONTAINMENT INTEGRITY (50-369/83-47-02) - PARAGRAPH 9; VIOLATION - FAILURE TO USE A PROCEDURE RESULTING IN LOSS OF SOURCE RANGE INSTRUMENTATION (50-369/83-47-01) - PARAGRAPH 7).

INSPECTION DECEMBER 20, 1983 - JANUARY 20, 1984 (83-48): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 74 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF OPERATIONS SAFETY VERIFICATION, SURVEILLANCE TESTING, MAINTENANCE ACTIVITIES, INDEPENDENT VERIFICATION, AND SHIFT MANNING. OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE VIOLATION WAS FOUND IN ONE AREA (VIOLATION - FAILURE TO ABIDE BY PROCEDURE CONCERNING EMERGENCY PLAN IMPLEMENTATION RESULTING IN INADEQUATE EMERGENCY KITS (50-369/83-48-01).

INSPECTION JANUARY 23-27 (84-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 21 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT WATER CHEMISTRY AND INSERVICE TESTING OF PUMPS AND VALVES. OF THE TWO AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 30-31 (84-03): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE IN THE EMERGENCY

****
* MCGUIRE 1 *
****

INSPECTION SUMMARY

PREPAREDNESS AREA OF PROTECTIVE ACTION DECISION MAKING. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

10CFR20.201B REQUIRES THE LICENSEE TO PERFORM SURVEYS AS NECESSARY TO DEMONSTRATE COMPLIANCE WITH 10CFR20.106 WHICH LIMITS THE RELEASE OF RADIOACTIVITY IN UNRESTRICTED AREAS TO THE CONCENTRATION IN APPENDIX B, TABLE II. CONTRARY TO THE ABOVE, DURING THE PERIOD FROM JUNE TO DECEMBER 1983 FAILURE TO PROPERLY CALIBRATE GAMMA SPECTROSCOPY SYSTEMS FOR CHARCOAL CARTRIDGE GEOMETRIES RESULTED IN THE USE OF THESE IMPROPERLY CALIBRATED DETECTORS FOR THE MEASUREMENT OF RADIOACTIVE EFFLUENTS RELEASED TO UNRESTRICTED AREAS.
(8345 4)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT CURRENT WRITTEN APPROVED PROCEDURES BE USED TO FACILITATE THE CONDUCT OF SURVEILLANCE TESTING AND MAINTENANCE THAT CAN AFFECT THE PERFORMANCE OF SAFETY RELATED EQUIPMENT. TECHNICAL SPECIFICATION 3.6.1.3 REQUIRES THAT EACH CONTAINMENT AIR LOCK BE OPERABLE WITH BOTH DOORS CLOSED EXCEPT WHEN THE AIR LOCK IS BEING USED FOR NORMAL TRANSIT ENTRY AND EXITS THROUGH THE CONTAINMENT, THEN AT LEAST ONE AIR LOCK DOOR SHALL BE CLOSED WHEN IN MODES 1, 2, 3 AND 4. THE ACTION STATEMENT OF THIS SPECIFICATION WITH ONE CONTAINMENT AIR LOCK DOOR INOPERABLE REQUIRES THAT THE OPERABLE AIR LOCK DOOR BE MAINTAINED CLOSED. TECHNICAL SPECIFICATION 3.3.1, ITEM 6.C. REQUIRES ONE OPERABLE SOURCE RANGE DETECTOR CHANNEL DURING OPERATION IN MODES 3, 4 AND 5. CONTRARY TO THE ABOVE, ON NOVEMBER 11, 1983, TWO INSTRUMENT AND ELECTRICAL TECHNICIANS WHILE PERFORMING MAINTENANCE ON UNIT 1 PERSONNEL AIR LOCK WITH THE OUTER DOOR OPEN, FAILED TO USE A PROCEDURE, RESULTING IN A LOSS OF CONTAINMENT INTEGRITY WHEN THEY INADVERTENTLY DEFLATED THE SEALS OF THE INNER DOOR. THEREAFTER, THE OPERABLE DOOR WAS IMMEDIATELY SECURED. TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT CURRENT WRITTEN APPROVED PROCEDURES BE USED TO FACILITATE THE CONDUCT OF SURVEILLANCE TESTING AND MAINTENANCE THAT CAN AFFECT THE PERFORMANCE OF SAFETY RELATED EQUIPMENT. TECHNICAL SPECIFICATION 3.6.1.3 REQUIRES THAT EACH CONTAINMENT AIR LOCK BE OPERABLE WITH BOTH DOORS CLOSED EXCEPT WHEN THE AIR LOCK IS BEING USED FOR NORMAL TRANSIT ENTRY AND EXITS THROUGH THE CONTAINMENT, THEN AT LEAST ONE AIR LOCK DOOR SHALL BE CLOSED WHEN IN MODES 1, 2, 3 AND 4. THE ACTION STATEMENT OF THIS SPECIFICATION WITH ONE CONTAINMENT AIR LOCK DOOR INOPERABLE REQUIRES THAT THE OPERABLE AIR LOCK DOOR BE MAINTAINED CLOSED. TECHNICAL SPECIFICATION 3.3.1, ITEM 6.C. REQUIRES ONE OPERABLE SOURCE RANGE DETECTOR CHANNEL DURING OPERATION IN MODES 3, 4 AND 5. CONTRARY TO THE ABOVE, ON NOVEMBER 17, 1983, DURING THE PERFORMANCE OF PROCEDURE IP-0-A-3100-06, A FUNCTIONAL TEST OF SOURCE RANGE INSTRUMENTATION, THE TECHNICIANS FAILED TO FOLLOW THE PROCEDURE, WHICH RESULTED IN A LOSS OF BOTH SOURCE RANGE CHANNELS.
(8347 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

MCGUIRE 1

OTHER ITEMS

REFUELING OUTAGE:

LAST IE SITE INSPECTION DATE: JANUARY 30-31, 1984 +

INSPECTION REPORT NO: 50-369/84-03 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-370 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: J. A. REAVIS EXT (704) 373-7567
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1450 X .9 = 1305
6. Design Electrical Rating (Net MWe): 1180
7. Maximum Dependable Capacity (Gross MWe): 1180
8. Maximum Dependable Capacity (Net MWe): 1180
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	696.0	696.0
13. Hours Reactor Critical	.0	.0	.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	0
18. Gross Elec Ener (MWH)	0	0	0
19. Net Elec Ener (MWH)	0	0	0
20. Unit Service Factor			
21. Unit Avail Factor		NOT IN	
22. Unit Cap Factor (MDC Net)		COMMERCIAL	
23. Unit Cap Factor (DER Net)		OPERATION	
24. Unit Forced Outage Rate			
25. Forced Outage Hours	.0	.0	.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: N/A

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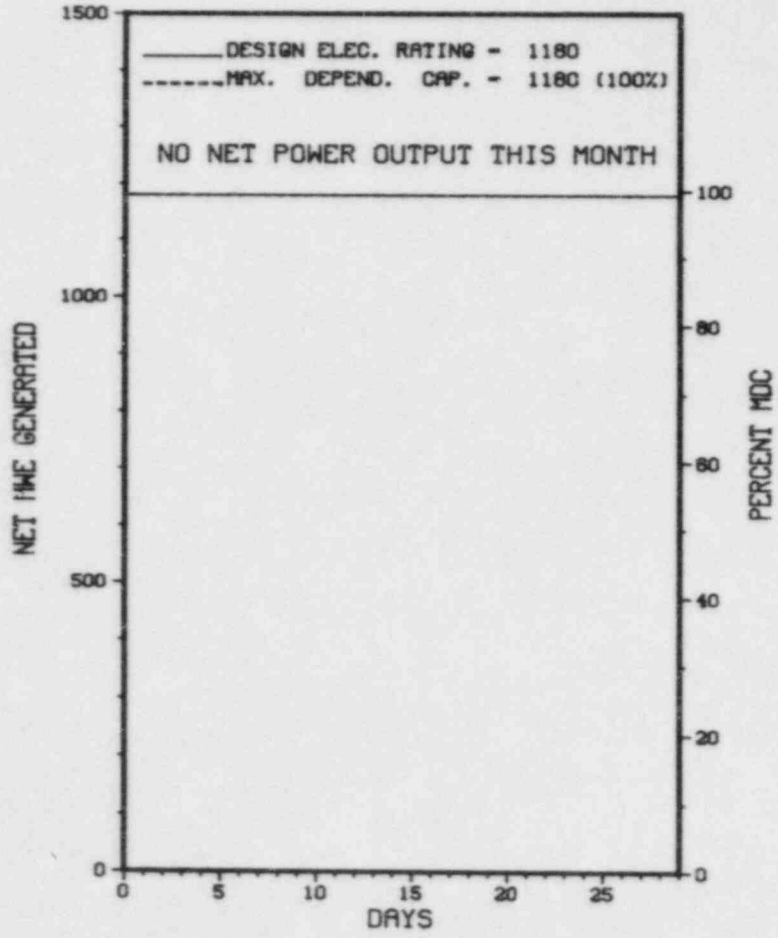
* MCGUIRE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MCGUIRE 2



FEBRUARY 1984

Report Period FEB 1984 UNIT SHUTDOWNS / REDUCTIONS

****
* MCGUIRE 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

****
* SUMMARY *
****

MCGUIRE 2 IS PRESENTLY IN POWER ASCENSION.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MCGUIRE 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....MECKLENBURG

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI N OF
CHARLOTTE, NC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 8, 1983

DATE ELEC ENER 1ST GENER...MAY 23, 1983

DATE COMMERCIAL OPERATE....N/S

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE NORMAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....POWER BLDG., BOX 2178
CHARLOTTE, NORTH CAROLINA 28201

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE POWER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. ORDERS

LICENSING PROJ MANAGER.....R. BIRKEL
DOCKET NUMBER.....50-370

LICENSE & DATE ISSUANCE....NPF-17, MAY 27, 1983

PUBLIC DOCUMENT ROOM.....

UNIVERSITY OF NORTH CAROLINA - CHARLOTTE
UNCC STATION,
CHARLOTTE, NC 28223

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 20 - DECEMBER 20 (83-54): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 38 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY VERIFICATION, SURVEILLANCE TESTING, AND MAINTENANCE ACTIVITIES. OF THE THREE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 20, 1983 - JANUARY 20, 1984 (83-55): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 74 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF OPERATIONS SAFETY VERIFICATION, SURVEILLANCE TESTING, MAINTENANCE ACTIVITIES, INDEPENDENT VERIFICATION, AND SHIFT MANNING. OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO VIOLATIONS WERE FOUND IN ONE AREA (VIOLATION - FAILURE TO ABIDE BY PROCEDURE CONCERNING EMERGENCY PLAN IMPLEMENTATION RESULTING IN INADEQUATE EMERGENCY KITS (50-370/83-55-02) - PARAGRAPH 11; VIOLATION - FAILURE TO FOLLOW PROCEDURE CONCERNING EQUIPMENT CONTROL RESULTING IN LOSS OF DECAY HEAT REMOVAL (50-370/83-55-01) - PARAGRAPH 7).

INSPECTION JANUARY 23-27 (84-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT WATER CHEMISTRY AND INSERVICE TESTING OF PUMPS AND VALVES. OF THE TWO AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 30-31 (84-03): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE IN THE EMERGENCY PREPAREDNESS AREA OF PROTECTIVE ACTION DECISION MAKING. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

****
* MCGUIRE 2 *
****

INSPECTION SUMMARY

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

MAX POWER LIMIT 50% PENDING STEAM GENERATOR MODIFICATION. DS-416 REACTOR TRIP BREAKER UNDER VOLTAGE COIL PROBLEMS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN TO COMPLETE WESTINGHOUSE "D-3" STEAM GENERATOR MODIFICATIONS. ALSO, FULL POWER LICENSE ISSUED ON MAY 27, 1983.

LAST IE SITE INSPECTION DATE: JANUARY 30-31, 1984 +

INSPECTION REPORT NO: 50-370/84-03 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-245 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: GEORGE HARRAN (203) 447-1791 X4194
4. Licensed Thermal Power (MWt): 2011
5. Nameplate Rating (Gross MWe): 735 X 0.9 = 662
6. Design Electrical Rating (Net MWe): 660
7. Maximum Dependable Capacity (Gross MWe): 684
8. Maximum Dependable Capacity (Net MWe): 654
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	116,184.0
13. Hours Reactor Critical	696.0	1,440.0	88,204.5
14. Rx Reserve Shtdwn Hrs	.0	.0	2,775.8
15. Hrs Generator On-Line	696.0	1,440.0	85,457.2
16. Unit Reserve Shtdwn Hrs	.0	.0	26.5
17. Gross Therm Ener (MWH)	1,374,192	2,839,710	155,888,578
18. Gross Elec Ener (MWH)	471,400	974,100	52,337,296
19. Net Elec Ener (MWH)	450,642	931,064	49,912,321
20. Unit Service Factor	100.0	100.0	73.6
21. Unit Avail Factor	100.0	100.0	73.6
22. Unit Cap Factor (MDC Net)	99.0	98.9	65.7
23. Unit Cap Factor (DER Net)	98.1	98.0	65.1
24. Unit Forced Outage Rate	.0	.0	13.8
25. Forced Outage Hours	.0	.0	5,673.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

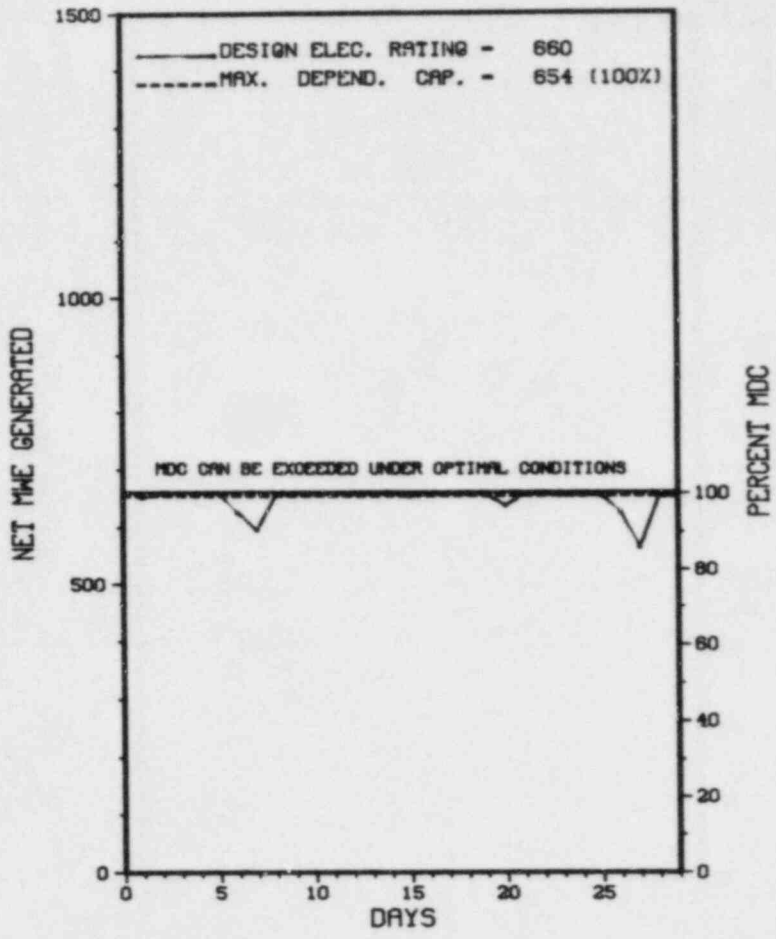
APRIL 15, 1984, REFUELING, 12 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MILLSTONE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MILLSTONE 1



FEBRUARY 1984

Report Period FEB 1984 UNIT SHUTDOWNS / REDUCTIONS

****
* MILLSTONE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

****
* SUMMARY *
****

MILLSTONE 1 OPERATED WITH NO REPORTED OUTAGES OR REDUCTIONS IN FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MILLSTONE 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....CONNECTICUT

COUNTY.....NEW LONDON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI SW OF
NEW LONDON, CONN

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...OCTOBER 26, 1970

DATE ELEC ENER 1ST GENER...NOVEMBER 29, 1970

DATE COMMERCIAL OPERATE....MARCH 1, 1971

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LONG ISLAND SOUND

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHEAST NUCLEAR ENERGY

CORPORATE ADDRESS.....P.O. BOX 270
HARTFORD, CONNECTICUT 06101

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....J. SHEDLOSKY

LICENSING PROJ MANAGER.....J. SHEA
DOCKET NUMBER.....50-245

LICENSE & DATE ISSUANCE....DPR-21, OCTOBER 26, 1970

PUBLIC DOCUMENT ROOM.....WATERFORD PUBLIC LIBRARY
45 ROPE FERRY ROAD
ROUTE 156
WATERFORD, CONNECTICUT 06385

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

****
* MILLSTONE 1 *
****

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-336 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: J. GIBSON (203) 447-1791 X 4419
4. Licensed Thermal Power (MWt): 2700
5. Nameplate Rating (Gross MWe): 1011 X 0.9 = 910
6. Design Electrical Rating (Net MWe): 870
7. Maximum Dependable Capacity (Gross MWe): 895
8. Maximum Dependable Capacity (Net MWe): 860
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	71,712.0
13. Hours Reactor Critical	696.0	1,316.9	49,681.8
14. Rx Reserve Shtdwn Hrs	.0	.0	2,166.9
15. Hrs Generator On-Line	577.7	1,014.1	47,196.0
16. Unit Reserve Shtdwn Hrs	.0	.0	468.2
17. Gross Therm Ener (MWH)	1,507,373	2,317,304	118,633,680
18. Gross Elec Ener (MWH)	486,800	735,401	38,532,773
19. Net Elec Ener (MWH)	464,632	692,899	36,909,647
20. Unit Service Factor	83.0	70.4	65.8
21. Unit Avail Factor	83.0	70.4	66.5
22. Unit Cap Factor (MDC Net)	77.6	56.0	61.2*
23. Unit Cap Factor (DER Net)	76.7	55.3	60.4*
24. Unit Forced Outage Rate	17.0	14.6	19.0
25. Forced Outage Hours	118.3	173.4	9,796.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

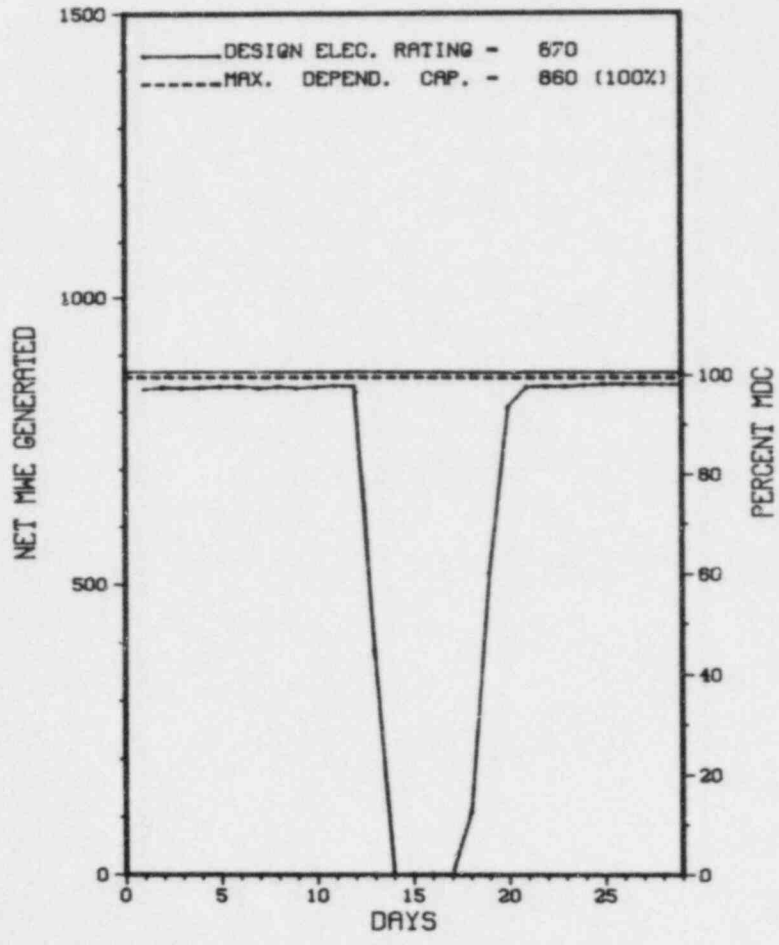
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MILLSTONE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MILLSTONE 2



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

MILLSTONE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2	02/13/84	F	118.3	A	1	84-006			SEVERAL REACTOR COOLANT SYSTEM RTD'S EXCEEDED THE TECH. SPEC. RESPONSE TIME. SEE LER 84-006.

SUMMARY

MILLSTONE 2 OPERATED WITH 1 OUTAGE FOR EQUIPMENT FAILURE IN FEBRUARY.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
		9-Other	(LER) File (NUREG-0161)

****
* MILLSTONE 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....CONNECTICUT

COUNTY.....NEW LONDON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI SW OF
NEW LONDON, CONN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 17, 1975

DATE ELEC ENER 1ST GENER...NOVEMBER 9, 1975

DATE COMMERCIAL OPERATE....DECEMBER 26, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LONG ISLAND SOUND

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHEAST NUCLEAR ENERGY

CORPORATE ADDRESS.....P.O. BOX 270
HARTFORD, CONNECTICUT 06101

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....J. SHEDLOSKY

LICENSING PROJ MANAGER.....K. HEITNER
DOCKET NUMBER.....50-336

LICENSE & DATE ISSUANCE....DPR-65, SEPTEMBER 30, 1975

PUBLIC DOCUMENT ROOM.....WATERFORD PUBLIC LIBRARY
45 ROPE FERRY ROAD
ROUTE 156
WATERFORD, CONNECTICUT 06385

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-263 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: A. L. Myrabo (612) 295-5151
4. Licensed Thermal Power (MWt): 1670
5. Nameplate Rating (Gross MWe): 632 X 0.9 = 569
6. Design Electrical Rating (Net MWe): 545
7. Maximum Dependable Capacity (Gross MWe): 553
8. Maximum Dependable Capacity (Net MWe): 525
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

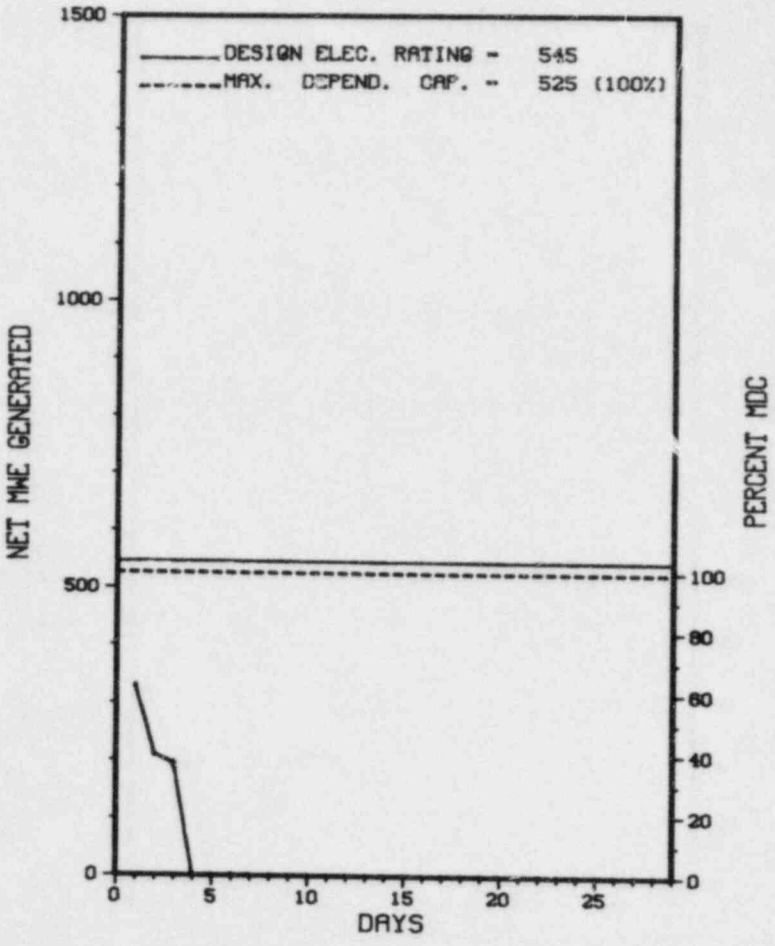
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	111,049.0
13. Hours Reactor Critical	66.5	810.5	89,915.4
14. Rx Reserve Shtdwn Hrs	.0	.0	940.7
15. Hrs Generator On-Line	64.8	808.8	88,003.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	57,552	897,898	141,233,814
18. Gross Elec Ener (MWH)	18,649	296,117	45,185,053
19. Net Elec Ener (MWH)	15,749	279,133	43,191,439
20. Unit Service Factor	9.3	56.2	79.2
21. Unit Avail Factor	9.3	56.2	79.2
22. Unit Cap Factor (MDC Net)	4.3	36.9	74.1
23. Unit Cap Factor (DER Net)	4.2	35.6	71.4
24. Unit Forced Outage Rate	.0	.0	5.3
25. Forced Outage Hours	.0	.0	1,288.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Start-up Date: 09/14/84

****
* MONTICELLO *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MONTICELLO



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

MONTICELLO

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	02/02/84	S	0.0	B	5		HA	TURBIN	REDUCED POWER TO PERFORM TURBINE AND FEEDWATER HEATER TESTING.
2	02/03/84	S	631.2	C	1		RC	FUELXX	START OF 1984 REFUELING OUTAGE.

SUMMARY

MONTICELLO SHUTDOWN ON FEBRUARY 3RD FOR REFUELING

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MONTICELLO *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MINNESOTA

COUNTY.....WRIGHT

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI NW OF
MINNEAPOLIS, MINN

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...DECEMBER 10, 1970

DATE ELEC ENER 1ST GENER...MARCH 5, 1971

DATE COMMERCIAL OPERATE....JUNE 30, 1971

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHERN STATES POWER

CORPORATE ADDRESS.....414 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....C. BROWN

LICENSING PROJ MANAGER.....H. NICOLARAS
DOCKET NUMBER.....50-263

LICENSE & DATE ISSUANCE....DPR-22, JANUARY 9, 1981

PUBLIC DOCUMENT ROOM.....ENVIRONMENTAL CONSERVATION LIBRARY
MINNEAPOLIS PUBLIC LIBRARY
300 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON DECEMBER 2, 10, (83-22): A ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTOR OF OPERATIONAL SAFETY AND ONSITE REVIEW COMMITTEE. THE INSPECTION INVOLVED A TOTAL OF 25 INSPECTOR-HOURS ONSITE BY 1 NRC INSPECTOR INCLUDING 4 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JANUARY 11 - FEBRUARY 1, (84-01): A ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTOR OF OPERATIONAL SAFETY; LICENSEE EVENT REPORTS; IE BULLETINS; IE CIRCULARS; PREPARATION FOR REFUELING; ONSITE REVIEW COMMITTEE; AND PROCEDURES. THE INSPECTION INVOLVED A TOTAL OF 189 INSPECTOR-HOURS ONSITE BY 3 NRC INSPECTORS INCLUDING 30 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT IS SHUT DOWN FOR A 6 MONTH OUTAGE. THE MAJOR ACTIVITY DURING THE OUTAGE WILL BE REPLACEMENT OF THE RECIRCULATION SYSTEM PIPING, IN ADDITION TO REFUELING.

LAST IE SITE INSPECTION DATE: JANUARY 11 THROUGH FEBRUARY 1, 1984

INSPECTION REPORT NO: 84-01

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-01	01/18/84	01/30/84	INADVERENT TRIP OF UNDERVOLTAGE RELAYS.
84-03	01/09/84	02/08/84	FAILURE OF HPCI ISOLATION VALVE TIME DELAY RELAY.
84-04	01/08/84	02/09/84	E-MODE OF EFT ACTUATED BY CHLORINE MONITOR TAPE BREAK.
84-05	01/12/84	02/10/84	E-MODE OF EFT ACTUATED BY HCI MONITOR.
84-06	01/19/84	02/24/84	SBGTS INITIATION DURING A.R.M. FUNCTIONAL TEST.

1. Docket: 50-220 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: JAN HALLENBECK (315) 349-2555
4. Licensed Thermal Power (MWt): 1850
5. Nameplate Rating (Gross MWe): 755 X 0.85 = 642
6. Design Electrical Rating (Net MWe): 620
7. Maximum Dependable Capacity (Gross MWe): 630
8. Maximum Dependable Capacity (Net MWe): 610
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	125,616.0
13. Hours Reactor Critical	696.0	1,440.0	87,742.5
14. Rx Reserve Shtdwn Hrs	.0	.0	1,204.2
15. Hrs Generator On-Line	696.0	1,440.0	84,928.1
16. Unit Reserve Shtdwn Hrs	.0	.0	20.2
17. Gross Therm Ener (MWH)	1,145,253	2,473,242	140,567,599
18. Gross Elec Ener (MWH)	385,718	837,348	46,469,130
19. Net Elec Ener (MWH)	373,706	811,557	45,006,316
20. Unit Service Factor	100.0	100.0	67.6
21. Unit Avail Factor	100.0	100.0	67.6
22. Unit Cap Factor (MDC Net)	88.0	92.4	58.7
23. Unit Cap Factor (DER Net)	86.6	90.9	57.8
24. Unit Forced Outage Rate	.0	.0	17.1
25. Forced Outage Hours	.0	.0	12,940.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

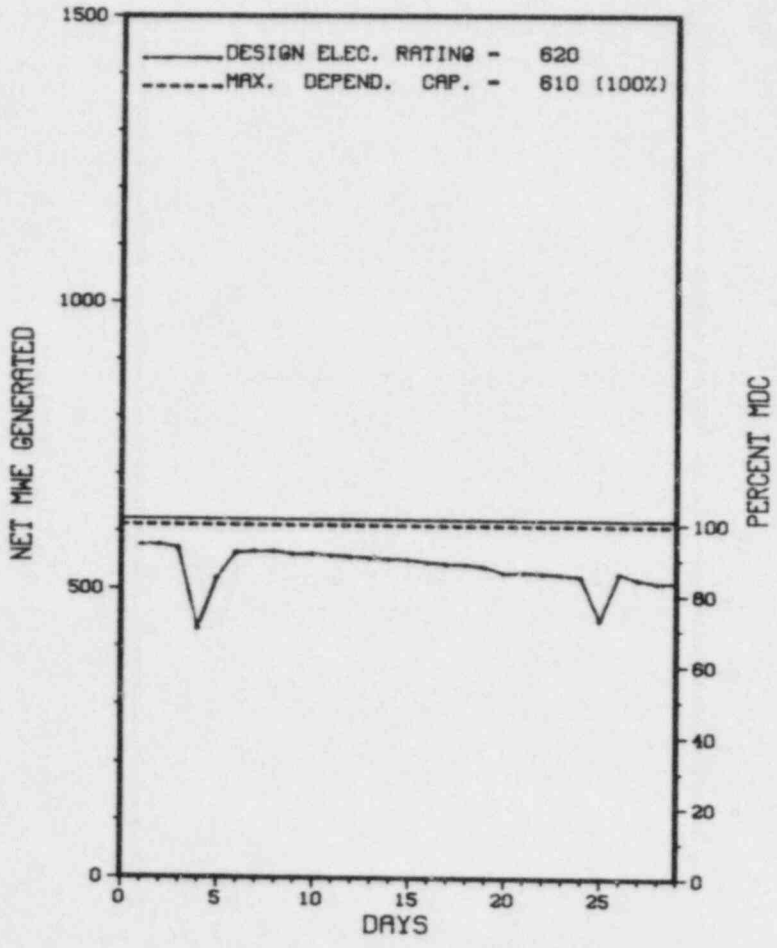
REFUEL AND OVERHAUL MARCH 16, 1984 - 8 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* NINE MILE POINT 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

NINE MILE POINT 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

NINE MILE POINT 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-4	02/04/84	S	0.0	H	5				LOAD REDUCTION TO 70.5% TO PULL CONTROL RODS.
84-5	02/25/84	S	0.0	H	5				LOAD REDUCTION TO 73.3% TO PULL CONTROL RODS.

SUMMARY

NINE MILE POINT 1 OPERATED WITH 2 REDUCTIONS AND NO OUTAGES DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* NINE MILE POINT 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....OSWEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...8 MI NE OF
OSWEGO, NY

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 5, 1969

DATE ELEC ENER 1ST GENER...NOVEMBER 9, 1969

DATE COMMERCIAL OPERATE....DECEMBER 1, 1969

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ONTARIO

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NIAGARA MOHAWK POWER

CORPORATE ADDRESS.....300 ERIE BOULEVARD WEST
SYRACUSE, NEW YORK 13202

CONTRACTOR
ARCHITECT/ENGINEER.....NIAGARA MOHAWK POWER CORP.

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....S. HUDSON

LICENSING PROJ MANAGER.....R. HERMANN
DOCKET NUMBER.....50-220

LICENSE & DATE ISSUANCE....DPR-63, DECEMBER 26, 1974

PUBLIC DOCUMENT ROOM.....STATE UNIVERSITY COLLEGE OF OSWEGO
PENFIELD LIBRARY - DOCUMENTS
OSWEGO, NY 13126
(315) 341-2323

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-338 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: JOAN N. LEE (703) 894-5151 X2527
4. Licensed Thermal Power (MWt): 2775
5. Nameplate Rating (Gross MWe): 947
6. Design Electrical Rating (Net MWe): 907
7. Maximum Dependable Capacity (Gross MWe): 930
8. Maximum Dependable Capacity (Net MWe): 883
9. If Changes Occur Above Since Last Report, Give Reasons:
MDC NET CHANGED TO REFLECT SERVICE LOADS.
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

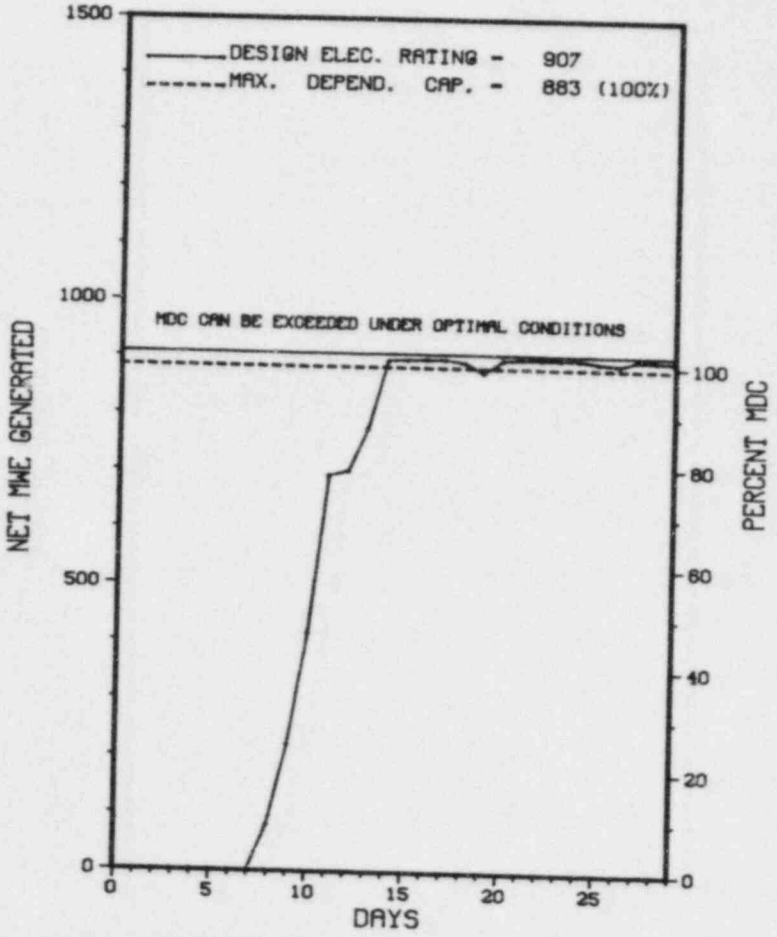
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	50,281.0
13. Hours Reactor Critical	494.6	708.6	34,295.7
14. Rx Reserve Shtdwn Hrs	7.1	7.1	2,182.8
15. Hrs Generator On-Line	490.5	687.8	33,349.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,293,539	1,833,192	86,888,969
18. Gross Elec Ener (MWH)	436,500	618,215	28,002,401
19. Net Elec Ener (MWH)	413,439	586,119	26,417,333
20. Unit Service Factor	70.5	47.8	66.3
21. Unit Avail Factor	70.5	47.8	66.3
22. Unit Cap Factor (MDC Net)	67.3	46.3	59.5
23. Unit Cap Factor (DER Net)	65.5	44.9	57.9
24. Unit Forced Outage Rate	29.5	52.2	14.0
25. Forced Outage Hours	205.5	752.2	5,320.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING OUTAGE 5/11/84; FALL MAINT. 11/23/84
27. If Currently Shutdown Estimated Startup Date: N/A

****
* NORTH ANNA 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

NORTH ANNA 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* NORTH ANNA 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-02	01/08/84	F	201.4	A	4				HIGH REACTOR COOLANT SYSTEM LEAK RATE OUTAGE CONTINUES FROM LAST MONTH. ENDED MONTH OF JANUARY WITH UNIT OFF-LINE. UNIT WAS ON-LINE FEBRUARY 8, 1984 AT 0924.
84-03	02/08/84	F	4.1	A	3				HIGH VIBRATION ON EHC PUMPS CAUSED A LOW EHC LEVEL TURBINE TRIP. REPAIRS MADE AND UNIT BACK ON-LINE AT 1410 FEBRUARY 8, 1984.
84-04	02/18/84	S	0.0	B	5				UNIT RAMPED DOWN FOR TURBINE VALVE FREEDOM TEST. UNIT RETURNED TO FULL POWER.
84-05	02/25/84	S	0.0	B	5				UNIT RAMPED DOWN FOR TURBINE VALVE FREEDOM TEST. UNIT RETURNED TO FULL POWER. ENDED THIS MONTH WITH UNIT AT 100% POWER.

****
* SUMMARY *
****

NORTH ANNA 1 RETURNED ONLINE FEBRUARY 8TH FROM AN EQUIPMENT REPAIR OUTAGE AND OPERATED ROUTINELY THE REMAINDER OF THE MONTH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* NORTH ANNA 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....LOUISA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI NW OF
RICHMOND, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 5, 1978

DATE ELEC ENER 1ST GENER...APRIL 17, 1978

DATE COMMERCIAL OPERATE....JUNE 6, 1978

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ANNA

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA ELECTRIC & POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. JOHNSON

LICENSING PROJ MANAGER.....L. ENGLE
DOCKET NUMBER.....50-338

LICENSE & DATE ISSUANCE....NPF-4, APRIL 1, 1978

PUBLIC DOCUMENT ROOM.....ALDERMAN LIBRARY/MANUSCRIPTS DEPT.
UNIV. OF VIRGINIA/CHARLOTTESVILLE VA 22901
& LOUISA COUNTY COURTHOUSE,
LOUISA, VA 23093

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 6, 1983 - JANUARY 5, 1984 (83-31): THIS ROUTINE INSPECTION INVOLVED 64 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE AND MAINTENANCE ACTIVITIES, PREVIOUS INSPECTION FINDINGS, FOLLOW-UP OF PREVIOUSLY IDENTIFIED ITEMS, SAFETY SYSTEM WALKDOWNS, LICENSEE EVENT REPORTS, COLD WEATHER PROTECTION, FIRE PROTECTION AND PLANT OPERATIONS. OF THE NINE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS. TWO APPARENT VIOLATIONS WERE IDENTIFIED IN THE REMAINING TWO AREAS (FAILURE TO MAINTAIN PROCEDURES, PARAGRAPHS 9 AND 10 AND FAILURE TO INSPECT FIRE BARRIERS, PARAGRAPH 9).

INSPECTION FEBRUARY 6-10 (84-03): THE INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A NORMAL SHIFT PERIOD; ONE AND ONE-HALF INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED SECURITY ORGANIZATION - PERSONNEL, SECURITY PROGRAM AUDIT, TESTING AND MAINTENANCE, PHYSICAL BARRIERS - PROTECTED AREA/VITAL AREAS, SECURITY SYSTEM POWER SUPPLY, ASSESSMENT AIDS, ACCESS CONTROL - PERSONNEL/PACKAGES/ VEHICLES, DETECTION AIDS - PROTECTED/VITAL AREAS, ALARM STATIONS, AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 14 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEM: ACCESS CONTROL - PERSONNEL.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 4.7.15A REQUIRES THAT EACH FIRE BARRIER PROTECTING SAFETY RELATED AREAS BE VERIFIED FUNCTIONAL BY A VISUAL

ENFORCEMENT SUMMARY

INSPECTION AT LEAST ONCE PER 18 MONTHS. CONTRARY TO THE ABOVE, THE FIRE BARRIERS BETWEEN THE ROOMS THAT HOUSE THE AUXILIARY FEEDWATER PUMPS HAVE NOT BEEN VISUALLY INSPECTED EVERY 18 MONTHS, AND WERE NOT INCLUDED AS PART OF THE 18 MONTH INSPECTION PROGRAM. TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. CONTRARY TO THE ABOVE, 1-PT-105.1.4 WAS NOT CURRENT IN THAT, THE INSPECTION OF THE WALL BETWEEN THE AUXILIARY AND FUEL BUILDINGS (PAGE 1 OF 22 OF ATTACHMENT 6.1-C OF 1-PT-105.1.4) WAS NOT DELETED AS APPROVED BY THE STATION NUCLEAR SAFETY AND OPERATING COMMITTEE. IN ADDITION, THE UNIT 2 AUXILIARY SHUTDOWN PANEL EMERGENCY AND ABNORMAL PROCEDURES WERE NOT CURRENT IN THAT 2-EP-1, 2, 3, 4 AND 5 AND 2-AP-1.4, 1.5, 4 AND 5 WERE NOT THE LATEST REVISION.

(8331 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: FEBRUARY 6-10, 1984 +

INSPECTION REPORT NO: 50-338/84-03 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-339 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: JOAN N. LEE (703) 894-5151 X2527
4. Licensed Thermal Power (MWt): 2775
5. Nameplate Rating (Gross MWe): 947
6. Design Electrical Rating (Net MWe): 907
7. Maximum Dependable Capacity (Gross MWe): 939
8. Maximum Dependable Capacity (Net MWe): 890
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	28,152.0
13. Hours Reactor Critical	621.7	1,365.7	21,012.6
14. Rx Reserve Shtdwn Hrs	3.5	3.5	2,243.5
15. Hrs Generator On-Line	584.5	1,328.5	20,1[illegible]6.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,474,396	3,431,253	53,852,283
18. Gross Elec Ener (MWH)	483,285	1,123,980	17,860,347
19. Net Elec Ener (MWH)	457,966	1,065,901	16,917,983
20. Unit Service Factor	84.0	92.3	73.2
21. Unit Avail Factor	84.0	92.3	73.2
22. Unit Cap Factor (MDC Net)	73.9	83.2	67.5
23. Unit Cap Factor (DER Net)	72.5	81.6	66.3
24. Unit Forced Outage Rate	16.0	7.7	14.7
25. Forced Outage Hours	111.5	111.5	3,559.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

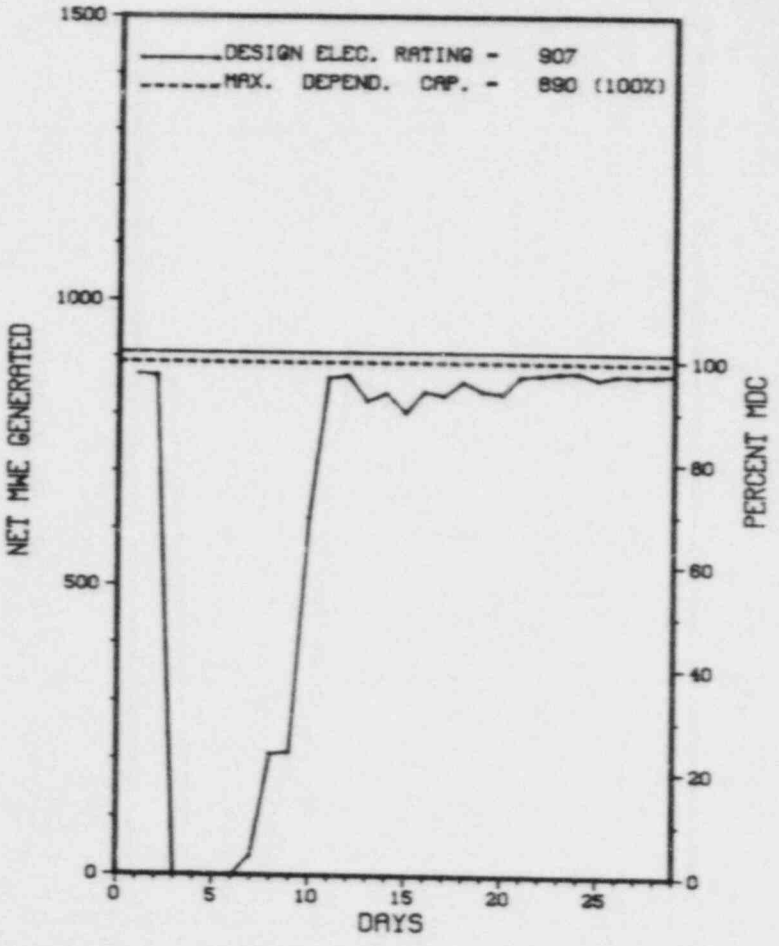
MAINTENANCE 03/30/84; REFUELING 9/21/84.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* NORTH ANNA 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

NORTH ANNA 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

NORTH ANNA 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-05	02/03/84	F	111.5	G	2				RAMPED DOWN DUE TO INJECTION OF SULFURIC ACID INTO UNIT 2 CONDENSATE STREAM AND ULTIMATELY INTO THE UNIT'S STEAM GENERATORS. FOLLOWING CHEMICAL CLEAN UP THE UNIT WAS RETURNED TO SERVICE.
84-06	02/10/84	S	0.0	B	5				RAMPED DOWN FOR TURBINE VALVE FREEDOM TEST. UNIT RETURNED TO FULL POWER.
84-07	02/12/84	S	0.0	B	5				RAMPED DOWN FOR LOAD FOLLOWING. UNIT RETURNED TO FULL POWER.
84-08	02/13/84	S	0.0	B	5				RAMPED DOWN FOR LOAD FOLLOWING. UNIT RETURNED TO FULL POWER.
84-09	02/14/84	S	0.0	B	5				RAMPED DOWN FOR LOAD FOLLOWING. UNIT RETURNED TO FULL POWER.
84-10	02/16/84	S	0.0	B	5				RAMPED DOWN FOR LOAD FOLLOWING. UNIT RETURNED TO FULL POWER.
84-11	02/19/84	S	0.0	B	5				RAMPED DOWN FOR LOAD FOLLOWING. UNIT RETURNED TO FULL POWER.
84-12	02/19/84	S	0.0	B	5				RAMPED DOWN FOR LOAD FOLLOWING. UNIT RETURNED TO FULL POWER.
84-13	02/25/84	S	0.0	B	5				RAMPED DOWN FOR TURBINE VALVE FREEDOM TEST. UNIT RETURNED TO FULL POWER. ENDED THIS MONTH WITH UNIT AT 100%.

SUMMARY

NORTH ANNA 2 OPERATED WITH 8 REDUCTIONS AND 1 OUTAGE DURING FEBRUARY.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* NORTH ANNA 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....LOUISA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI NW OF
RICHMOND, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 12, 1980

DATE ELEC ENER 1ST GENER...AUGUST 25, 1980

DATE COMMERCIAL OPERATE....DECEMBER 14, 1980

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ANNA

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA ELECTRIC & POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. JOHNSON

LICENSING PROJ MANAGER.....L. ENGLE
DOCKET NUMBER.....50-339

LICENSE & DATE ISSUANCE....NPF-7, AUGUST 21, 1980

PUBLIC DOCUMENT ROOM.....ALDERMAN LIBRARY/MANUSCRIPTS DEPT.
UNIV. OF VIRGINIA/CHARLOTTESVILLE VA 22901
& LOUISA COUNTY COURTHOUSE,
LOUISA, VA 23093

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 6, 1983 - JANUARY 5, 1984 (83-31): THIS ROUTINE INSPECTION INVOLVED 64 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE AND MAINTENANCE ACTIVITIES, PREVIOUS INSPECTION FINDINGS, FOLLOW-UP OF PREVIOUSLY IDENTIFIED ITEMS, SAFETY SYSTEM WALKDOWNS, LICENSEE EVENT REPORTS, COLD WEATHER PROTECTION, FIRE PROTECTION AND PLANT OPERATIONS. OF THE NINE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS. TWO APPARENT VIOLATIONS WERE IDENTIFIED IN THE REMAINING TWO AREAS (FAILURE TO MAINTAIN PROCEDURES, PARAGRAPHS 9 AND 10 AND FAILURE TO INSPECT FIRE BARRIERS, PARAGRAPH 9).

INSPECTION FEBRUARY 6-10 (84-03): THE INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A NORMAL SHIFT PERIOD; ONE AND ONE-HALF INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED SECURITY ORGANIZATION - PERSONNEL, SECURITY PROGRAM AUDIT, TESTING AND MAINTENANCE, PHYSICAL BARRIERS - PROTECTED AREA/VITAL AREAS, SECURITY SYSTEM POWER SUPPLY, ASSESSMENT AIDS, ACCESS CONTROL - PERSONNEL/PACKAGES/ VEHICLES, DETECTION AIDS - PROTECTED/VITAL AREAS, ALARM STATIONS, AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 14 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEM: ACCESS CONTROL - PERSONNEL.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: FEBRUARY 6-10, 1984 +

INSPECTION REPORT NO: 50-339/84-03 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-269 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: J. A. REAVIS (704) 373-7567

4. Licensed Thermal Power (MWt): 2568

5. Nameplate Rating (Gross MWe): 1038 X 0.9 = 934

6. Design Electrical Rating (Net MWe): 887

7. Maximum Dependable Capacity (Gross MWe): 899

8. Maximum Dependable Capacity (Net MWe): 860

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	93,145.0
13. Hours Reactor Critical	696.0	1,440.0	65,981.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,440.0	62,829.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,787,819	3,696,857	149,994,889
18. Gross Elec Ener (MWH)	626,220	1,297,480	52,165,710
19. Net Elec Ener (MWH)	599,828	1,242,415	49,407,966
20. Unit Service Factor	100.0	100.0	67.5
21. Unit Avail Factor	100.0	100.0	67.5
22. Unit Cap Factor (MDC Net)	100.2	100.3	61.5*
23. Unit Cap Factor (DER Net)	97.2	97.3	59.9*
24. Unit Forced Outage Rate	.0	.0	17.1
25. Forced Outage Hours	.0	.0	12,047.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

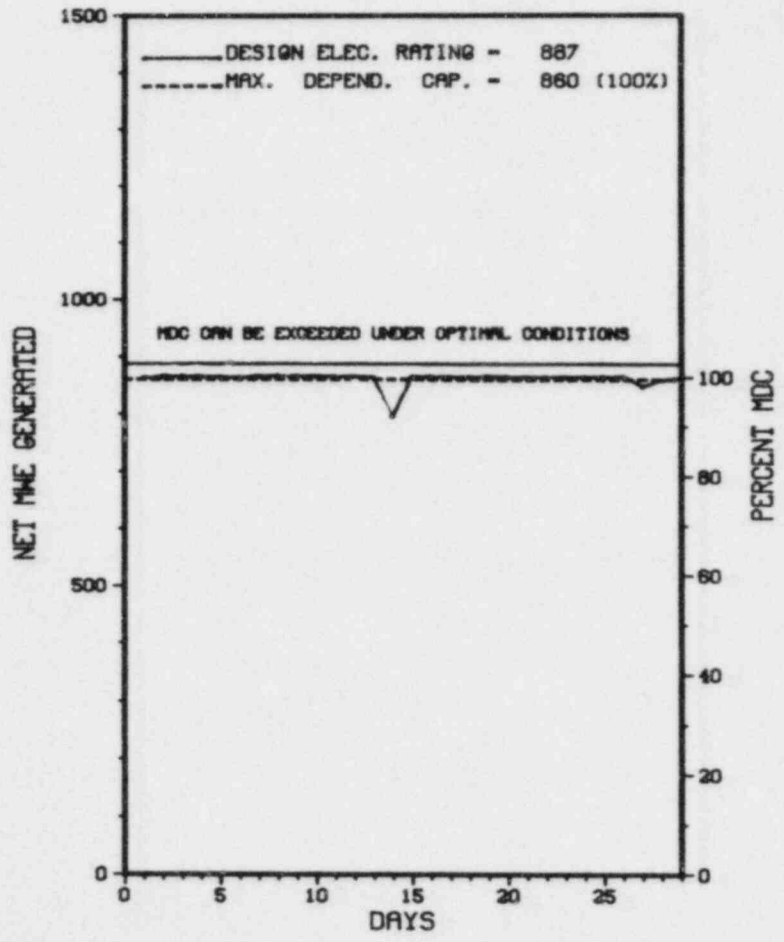
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* OCONEE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

OCONEE 1



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

OCONEE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2-P	02/13/84	S	0.0	F	5		ZZ	ZZZZZZ	DISPATCHER REDUCTION.
3-P	02/27/84	S	0.0	B	5		CC	VALVEX	PERIODIC TESTS (PT'S) - CONTROL VALVE AND STOP VALVE.

SUMMARY

OCONEE 1 OPERATED AT FULL POWER WITH 2 REDUCTIONS DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* OCONEE 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....OCONEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI W OF
GREENVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 19, 1973

DATE ELEC ENER 1ST GENER...MAY 6, 1973

DATE COMMERCIAL OPERATE....JULY 15, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE KEOWEE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE & BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. BRYANT

LICENSING PROJ MANAGER.....H. NICOLARAS
DOCKET NUMBER.....50-269

LICENSE & DATE ISSUANCE....DPR-38, FEBRUARY 6, 1973

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501 W. SOUTH BROAD ST.
WALHALLA, SOUTH CAROLINA 29691

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION FEBRUARY 6-10 (84-02): THE INSPECTION INVOLVED 11 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. ONE AND ONE-HALF INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED SITE ORIENTATION; REVIEW OF SECURITY PLAN REVISIONS; SECURITY ORGANIZATION (PERSONNEL AND RESPONSE); SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS (PROTECTED AREA); SECURITY SYSTEM POWER SUPPLY; ACCESS CONTROL (PERSONNEL, PACKAGES, AND VEHICLES); DETECTION AIDS (PROTECTED AND VITAL AREAS); ALARM STATIONS AND COMMUNICATIONS. THE INSPECTION ALSO CONSISTED OF A VISIT TO GENERAL OFFICE, DUKE POWER COMPANY. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEM: FAILURE TO MEET SPECIFIC REQUIREMENTS OF THE T&Q PLAN.

INSPECTION FEBRUARY 1-2 (84-04): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR HOURS ON SITE IN THE EMERGENCY PREPAREDNESS AREA OF PROTECTIVE ACTION DECISION MAKING. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

FAILURE TO IMPLEMENT SPECIFIC PORTIONS OF THE T & Q PLAN CONCERNING GUARD FORCE QUALIFICATIONS BY REQUIRED IMPLEMENTATION DATE OF APRIL 20, 1983.
(8402 3)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: FEBRUARY 6-10, 1984 +

INSPECTION REPORT NO: 50-269/84-02 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-270 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: J. A. REAVIS (704) 373-7567
4. Licensed Thermal Power (MWt): 2568
5. Nameplate Rating (Gross MWe): 1038 X 0.9 = 934
6. Design Electrical Rating (Net MWe): 887
7. Maximum Dependable Capacity (Gross MWe): 899
8. Maximum Dependable Capacity (Net MWe): 860
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	83,065.0
13. Hours Reactor Critical	696.0	1,440.0	58,753.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,440.0	57,600.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,790,632	3,699,486	136,190,152
18. Gross Elec Ener (MWH)	619,200	1,279,500	46,384,356
19. Net Elec Ener (MWH)	594,378	1,227,665	44,039,234
20. Unit Service Factor	100.0	100.0	69.3
21. Unit Avail Factor	100.0	100.0	69.3
22. Unit Cap Factor (MDC Net)	99.3	99.1	61.5*
23. Unit Cap Factor (DER Net)	96.3	96.1	59.8*
24. Unit Forced Outage Rate	.0	.0	16.1
25. Forced Outage Hours	.0	.0	10,256.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

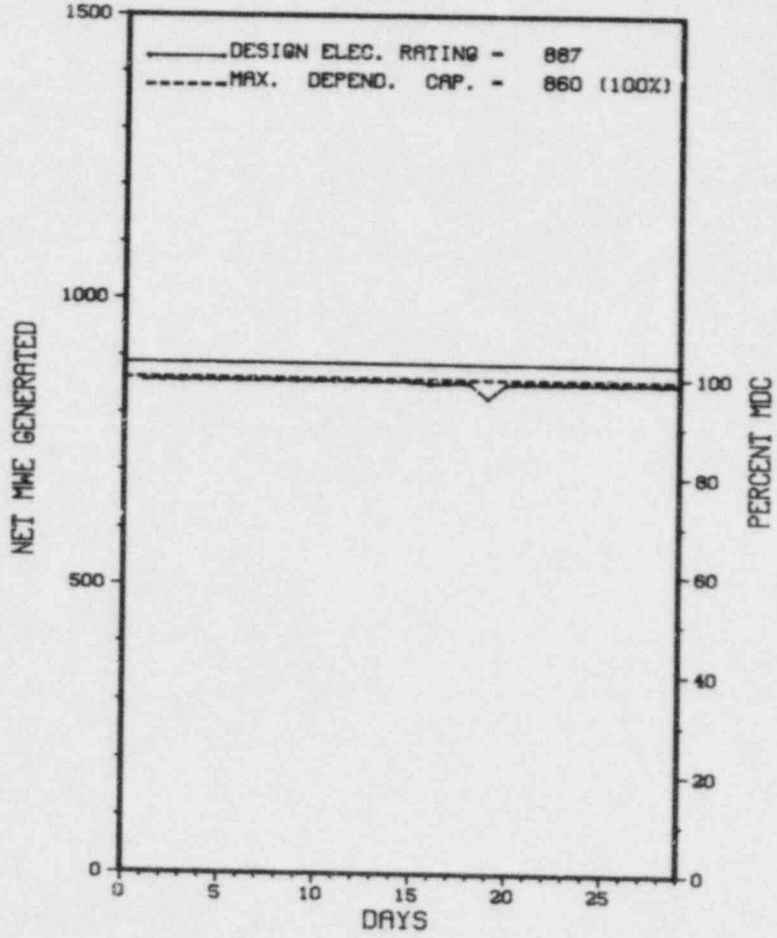
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* OCONEE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

OCONEE 2



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

OCONEE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
3-P	02/18/84	S	0.0	B	5		CC	VALVEX	PERIODIC TESTS (PT'S) - TURBINE VALVE.

*** SUMMARY ***

OCONEE 2 OPERATED AT FULL POWER WITH 1 REDUCTION DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* OCONEE 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....OCONEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI W OF
GREENVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 11, 1973

DATE ELEC ENER 1ST GENER...DECEMBER 5, 1973

DATE COMMERCIAL OPERATE....SEPTEMBER 9, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE KEOWEE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE & BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. BRYANT

LICENSING PROJ MANAGER.....H. NICOLARAS
DOCKET NUMBER.....50-270

LICENSE & DATE ISSUANCE....DPR-47, OCTOBER 6, 1973

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501 W. SOUTH BROAD ST.
WALHALLA, SOUTH CAROLINA 29691

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION FEBRUARY 6-10 (84-02): THE INSPECTION INVOLVED 12 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. ONE AND ONE-HALF INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED SITE ORIENTATION; REVIEW OF SECURITY PLAN REVISIONS; SECURITY ORGANIZATION (PERSONNEL AND RESPONSE); SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS (PROTECTED AREA); SECURITY SYSTEM POWER SUPPLY; ACCESS CONTROL (PERSONNEL, PACKAGES, AND VEHICLES); DETECTION AIDS (PROTECTED AND VITAL AREAS); ALARM STATIONS AND COMMUNICATIONS. THE INSPECTION ALSO CONSISTED OF A VISIT TO GENERAL OFFICE, DUKE POWER COMPANY. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEM: FAILURE TO MEET SPECIFIC REQUIREMENTS OF THE T&Q PLAN.

INSPECTION FEBRUARY 1-2 (84-04): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR HOURS ON SITE IN THE EMERGENCY PREPAREDNESS AREA OF PROTECTIVE ACTION DECISION MAKING. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, FAILURE TO FOLLOW MAINTENANCE PROCEDURE FOR THE INSTALLATION OF SEAL STAGING COILS RESULTING IN NO RCP SEAL LEAKOFF FLOW.
(8337 5)

ENFORCEMENT SUMMARY

FAILURE TO IMPLEMENT SPECIFIC PORTIONS OF THE T & Q PLAN CONCERNING GUARD FORCE QUALIFICATIONS BY REQUIRED IMPLEMENTATION DATE OF APRIL 20, 1983. FAILURE TO IMPLEMENT SPECIFIC PORTIONS OF THE T & Q PLAN CONCERNING GUARD FORCE QUALIFICATIONS BY REQUIRED IMPLEMENTAITON DATE OF APRIL 20, 1983.
(8402 3)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: FEBRUARY 6-10, 1984 +

INSPECTION REPORT NO: 50-270/84-02 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-287 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: J. A. REAVIS (704) 373-7567
4. Licensed Thermal Power (MWt): 2568
5. Nameplate Rating (Gross MWe): 1038 X 0.9 = 934
6. Design Electrical Rating (Net MWe): 887
7. Maximum Dependable Capacity (Gross MWe): 899
8. Maximum Dependable Capacity (Net MWe): 860
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	80,712.0
13. Hours Reactor Critical	694.9	1,438.9	58,148.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	693.0	1,437.0	57,020.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,760,898	3,670,368	139,162,931
18. Gross Elec Ener (MWH)	607,200	1,267,370	48,081,964
19. Net Elec Ener (MWH)	582,478	1,215,846	45,782,964
20. Unit Service Factor	99.6	99.8	70.6
21. Unit Avail Factor	99.6	99.8	70.6
22. Unit Cap Factor (MDC Net)	97.3	98.2	65.8*
23. Unit Cap Factor (DER Net)	94.4	95.2	64.0*
24. Unit Forced Outage Rate	.4	.2	14.9
25. Forced Outage Hours	3.0	3.0	10,145.0

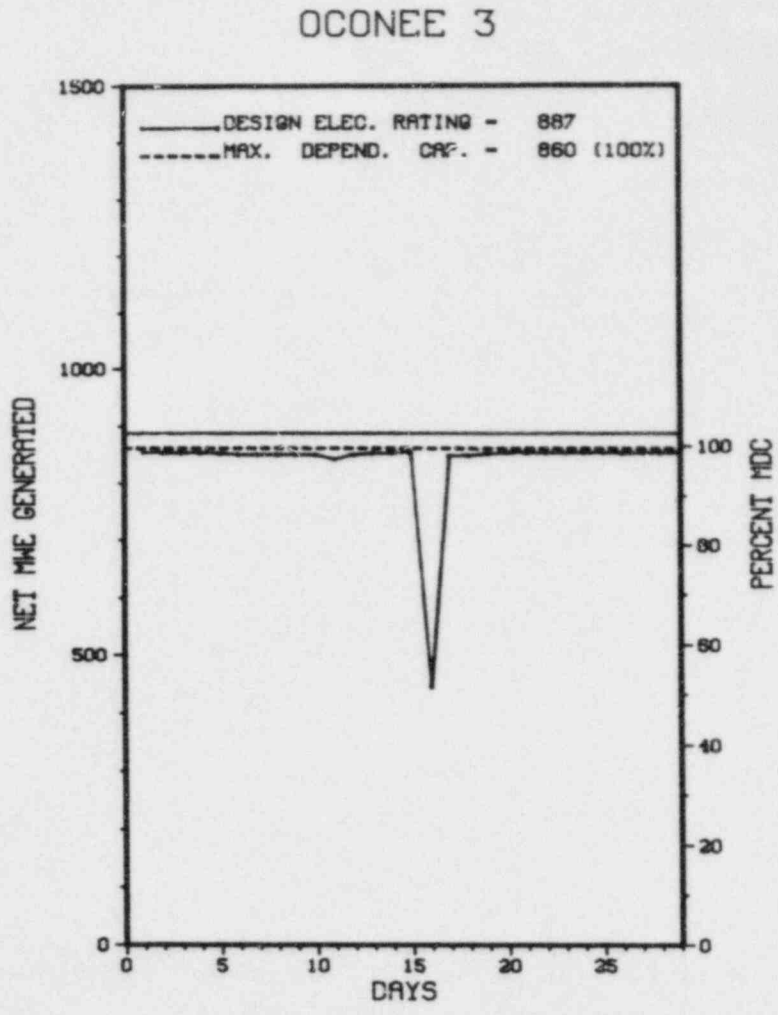
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

REFUELING - MARCH 8, 1984 - 10 WEEKS

27. If Currently Shutdown Estimated Startup Date: N/A

****
* OCONEE 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

OCONEE 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	02/16/84	F	3.0	A	3		CB	INSTRU	REACTOR COOLANT FLOW LOOP "A" FAILURE.

SUMMARY

OCONEE 3 OPERATED AT FULL POWER WITH 1 SHORT DURATION OUTAGE FOR EQUIPMENT FAILURE.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	2-Manual Scram	Instructions for
	C-Refueling	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
	F-Admin	9-Other	(LER) File (NUREG-0161)
	G-Oper Error		
	H-Other		

****
* OCONEE 3 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....OCONEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI W OF
GREENVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...SEPTEMBER 5, 1974

DATE ELEC ENER 1ST GENER...SEPTEMBER 18, 1974

DATE COMMERCIAL OPERATE....DECEMBER 16, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE KEOWEE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE & BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. BRYANT

LICENSING PROJ MANAGER.....H. NICOLARAS
DOCKET NUMBER.....50-287

LICENSE & DATE ISSUANCE....DPR-55, JULY 19, 1974

PUBLIC DOCUMENT ROOM.....OCONEE COUNTY LIBRARY
501 W. SOUTH BROAD ST.
WALHALLA, SOUTH CAROLINA 29691

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION FEBRUARY 6-10 (84-02): THE INSPECTION INVOLVED 12 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. ONE AND ONE-HALF INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED SITE ORIENTATION; REVIEW OF SECURITY PLAN REVISIONS; SECURITY ORGANIZATION (PERSONNEL AND RESPONSE); SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS (PROTECTED AREA); SECURITY SYSTEM POWER SUPPLY; ACCESS CONTROL (PERSONNEL, PACKAGES, AND VEHICLES); DETECTION AIDS (PROTECTED AND VITAL AREAS); ALARM STATIONS AND COMMUNICATIONS. THE INSPECTION ALSO CONSISTED OF A VISIT TO GENERAL OFFICE, DUKE POWER COMPANY. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEM: FAILURE TO MEET SPECIFIC REQUIREMENTS OF THE T&Q PLAN.

INSPECTION FEBRUARY 1-2 (84-04): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR HOURS ON SITE IN THE EMERGENCY PREPAREDNESS AREA OF PROTECTIVE ACTION DECISION MAKING. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: FEBRUARY 6-10, 1984 +

INSPECTION REPORT NO: 50-287/84-02 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-219 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: JOSEPH R. MOLNAR (609) 971-4699

4. Licensed Thermal Power (MWt): 1930

5. Nameplate Rating (Gross MWe): 722 X .9 = 650

6. Design Electrical Rating (Net MWe): 650

7. Maximum Dependable Capacity (Gross MWe): 650

8. Maximum Dependable Capacity (Net MWe): 620

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	124,368.0
13. Hours Reactor Critical	696.0	696.0	85,319.9
14. Rx Reserve Shtdwn Hrs	.0	.0	468.2
15. Hrs Generator On-Line	.0	.0	82,693.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	136,301,260
18. Gross Elec Ener (MWH)	0	0	46,056,905
19. Net Elec Ener (MWH)	0	-1,991	44,283,692
20. Unit Service Factor	.0	.0	66.5
21. Unit Avail Factor	.0	.0	66.5
22. Unit Cap Factor (MDC Net)	.0	.0	57.4*
23. Unit Cap Factor (DER Net)	.0	.0	54.8
24. Unit Forced Outage Rate	.0	.0	11.6
25. Forced Outage Hours	.0	.0	8,916.8

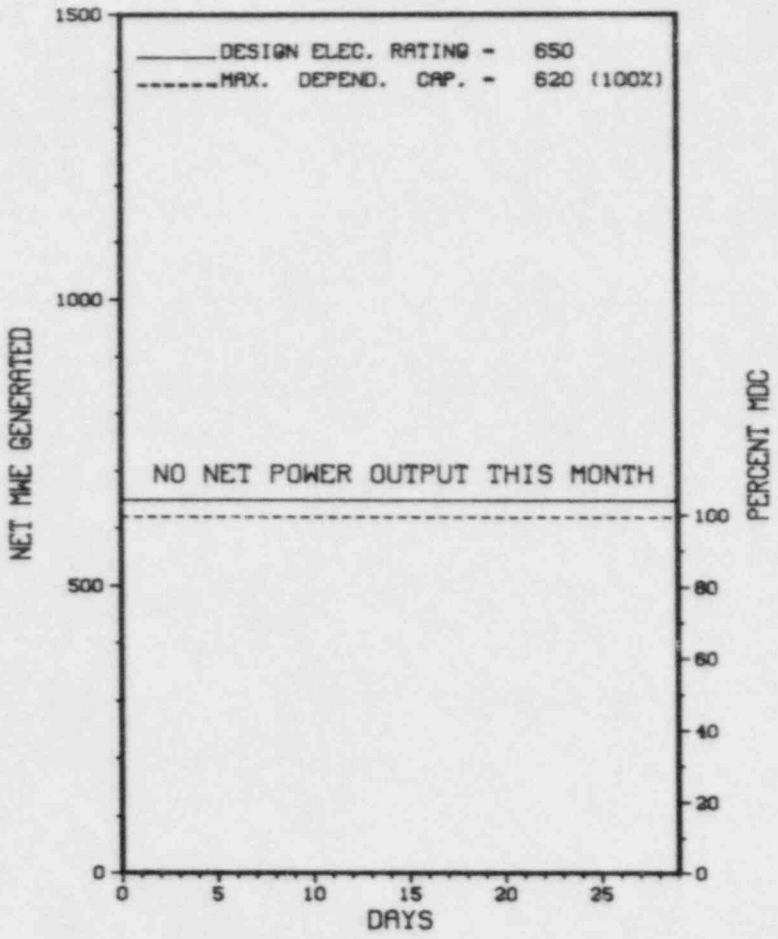
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: 06/11/84

****
* OYSTER CREEK 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

OYSTER CREEK 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
31	02/11/83	S	696.0	C	4		RC	FUELXX	REFUELING AND MAINTENANCE OUTAGE CONTINUES.

SUMMARY

OYSTER CREEK 1 REMAINS SHUTDOWN IN A CONTINUING REFUELING/ MAINTENANCE OUTAGE.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training	5-Reduced Load	Licensee Event Report
	& License Examination	9-Other	(LER) File (NUREG-0161)

****
* OYSTER CREEK 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NEW JERSEY

COUNTY.....OCEAN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9 MI S OF
TOMS RIVER, NJ

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MAY 3, 1969

DATE ELEC ENER 1ST GENER...SEPTEMBER 23, 1969

DATE COMMERCIAL OPERATE....DECEMBER 1, 1969

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....BARNEGAT BAY

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GPU NUCLEAR CORPORATION

CORPORATE ADDRESS.....100 INTERPACE PARKWAY
PARSIPPANY, NEW JERSEY 07054

CONTRACTOR
ARCHITECT/ENGINEER.....BURNS & ROE

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BURNS & ROE

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....C. COWGILL

LICENSING PROJ MANAGER.....J. LOMBARDO
DOCKET NUMBER.....50-219

LICENSE & DATE ISSUANCE....DPR-16, AUGUST 1, 1969

PUBLIC DOCUMENT ROOM.....OCEAN COUNTY LIBRARY
101 WASHINGTON STREET
TOMS RIVER, NEW JERSEY 08753

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* OYSTER CREEK 1 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-255 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: K. J. CAVADAS (616) 764-8913

4. Licensed Thermal Power (MWt): 2530

5. Nameplate Rating (Gross MWe): 955 X 0.85 = 812

6. Design Electrical Rating (Net MWe): 805

7. Maximum Dependable Capacity (Gross MWe): 675

8. Maximum Dependable Capacity (Net MWe): 635

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	106,935.0
13. Hours Reactor Critical	.0	.0	59,259.7
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	56,278.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	115,360,224
18. Gross Elec Ener (MWH)	0	0	35,750,440
19. Net Elec Ener (MWH)	0	0	33,628,014
20. Unit Service Factor	.0	.0	52.6
21. Unit Avail Factor	.0	.0	52.6
22. Unit Cap Factor (MDC Net)	.0	.0	49.5
23. Unit Cap Factor (DER Net)	.0	.0	39.1
24. Unit Forced Outage Rate	.0	.0	32.1
25. Forced Outage Hours	.0	.0	12,525.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

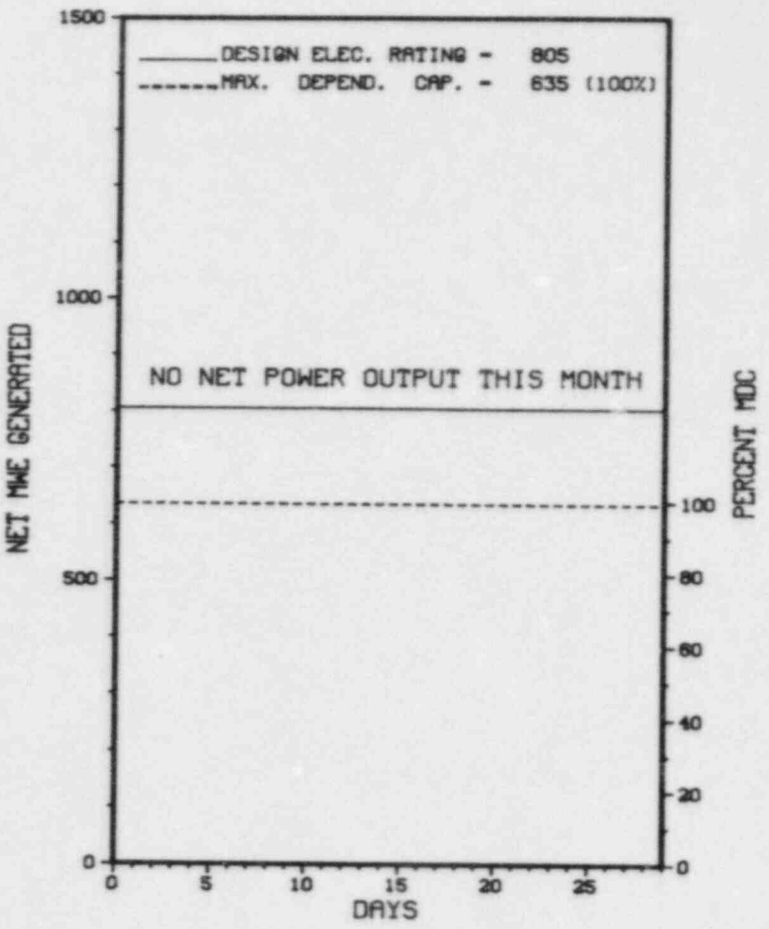
NONE

27. If Currently Shutdown Estimated Startup Date: 06/15/84

****
* PALISADES *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PALISADES



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

PALISADES

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	08/12/83	S	696.0	C	4		RC	FUELXX	REFUELING/MAINTENANCE OUTAGE CONTINUES.

SUMMARY

POWER SITES REMAIN SHUTDOWN IN A CONTINUING REFUELING/MAINTENANCE OUTAGE.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* PALISADES *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....MICHIGAN

COUNTY.....VANBUREN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SOUTH HAVEN, MI

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 24, 1971

DATE ELEC ENER 1ST GENER...DECEMBER 31, 1971

DATE COMMERCIAL OPERATE....DECEMBER 31, 1971

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONSUMERS POWER

CORPORATE ADDRESS.....212 WEST MICHIGAN AVENUE
JACKSON, MICHIGAN 49201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....B. JORGENSON

LICENSING PROJ MANAGER.....W. PAULSON
DOCKET NUMBER.....50-255

LICENSE & DATE ISSUANCE....DPR-20, OCTOBER 16, 1972

PUBLIC DOCUMENT ROOM.....KALAMAZOO PUBLIC LIBRARY
315 SOUTH ROSE STREET
REFERENCE DEPARTMENT
KALAMAZOO, MICHIGAN 49007

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON NOVEMBER 11, AND JANUARY 17, (83-27): ROUTINE ANNOUNCED INSPECTION BY REGION III INSPECTOR OF CALCULATIONS CONCERNING ADEQUACY OF FAN COOLERS FOR THE ENGINEERED SAFEGUARDS ROOM AS REFERENED IN LER 83-007. THIS INSPECTION INVOLVED A TOTAL OF 12 INSPECTOR-HOURS AT THE LICENSEE'S OFFICE BY ONE NRC INSPECTOR. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JANUARY 18-20, (84-01): SPECIAL, ANNOUNCED INSPECTION OF THE RADIATION PROTECTION PROGRAM INCLUDING: AN UNEXPLAINED HIGH WHOLE BODY EXPOSURE; A LEAKING CONTAINER DURING TRANSPORT; TMI ACTION PLAN ITEMS; EMPLOYEE CONCERNS RELATED TO IE BULLETIN NO. 80-10; IMPLEMENTATION OF 10 CFR 61; STEAM GENERATOR REPAIR ACTIVITIES; OPEN ITEMS; AND HOUSEKEEPING. THE INSPECTION INVOLVED 24 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN EACH OF THREE AREAS (PLANT ORGANIZATION NOT IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS, LEAKING RADWASTE DRUM DURING TRANSPORT, AND NOBLE GAS EFFLUENT MONITOR NOT INSTALLED IN ACCORDANCE WITH MARCH 14, 1983, NRC ORDER).

INSPECTION DURING JANUARY 3 THROUGH JANUARY 31 (84-02): ROUTINE, UNANNOUCED INSPECTION BY RESIDENT INSPECTOR OF ACTION ON PREVIOUSLY IDENTIFIED ITEMS; PLANT SAFETY; MAINTENANCE; REPORTABLE EVENTS; AND MISCELLANEOUS ACTIVITIES. NON-ROUTINE INSPECTION COVERED REVIEW OF A POTENTIALLY SIGNIFICANT OPERATING EVENT INVOLVING LOSS OF BOTH OFFSITE AND ONSITE AC POWER ON JANUARY 8, 1984, WITH THE REACTOR COMPLETELY DEFUELED. THE INSPECTION INVOLVED A TOTAL OF 114 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR INCLUDING 37 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS ARE REPORTED HEREIN, FOR THE

INSPECTION SUMMARY

INSPECTED AREAS. APPARENT NONCOMPLIANCE WAS ASSOCIATED WITH A PLANT BLACKOUT OF JANUARY 8, 1984. APPROPRIATE ENFORCEMENT ACTION FOR THOSE MATTERS WILL BE DECIDED UPON FOLLOWING A SCHEDULED MEETING WITH THE LICENSEE TO DISCUSS ALL THE ISSUES.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

INDICATIONS OF MAJOR STEAM GENERATOR TUBE DEGENERATION ARE BEING INVESTIGATED AND EVALUATED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT SHUTDOWN ON 8/13/83 TO START A REFUELING AND MAINTENANCE OUTAGE. RESTART SCHEDULED FOR MAY, 1984.

LAST IE SITE INSPECTION DATE: JANUARY 3-31, 1984

INSPECTION REPORT NO: 84-02

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-79/ 03L-0	12/30/83	01/25/84	AS FOUND TRIP SETTINGS FOR LOW PCS FLOW WERE FOUND BELOW THE LIMIT SPECIFIED IN TS TABLE.
84-01	01/08/84	02/07/84	LOSS OF COMMUNICATIONS.

1. Docket: 50-277 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: W. M. Alden (215) 841-5022

4. Licensed Thermal Power (MWt): 3293

5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152

6. Design Electrical Rating (Net MWe): 1065

7. Maximum Dependable Capacity (Gross MWe): 1098

8. Maximum Dependable Capacity (Net MWe): 1051

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	84,648.0
13. Hours Reactor Critical	495.1	1,187.9	60,887.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	481.3	1,150.6	59,162.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,467,302	3,611,784	174,166,394
18. Gross Elec Ener (MWH)	462,430	1,178,770	57,349,860
19. Net Elec Ener (MWH)	447,367	1,142,694	54,979,124
20. Unit Service Factor	69.2	79.9	69.9
21. Unit Avail Factor	69.2	79.9	69.9
22. Unit Cap Factor (MDC Net)	61.2	75.5	61.8
23. Unit Cap Factor (DER Net)	60.4	74.5	61.0
24. Unit Forced Outage Rate	8.0	9.2	11.9
25. Forced Outage Hours	41.7	116.4	7,975.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

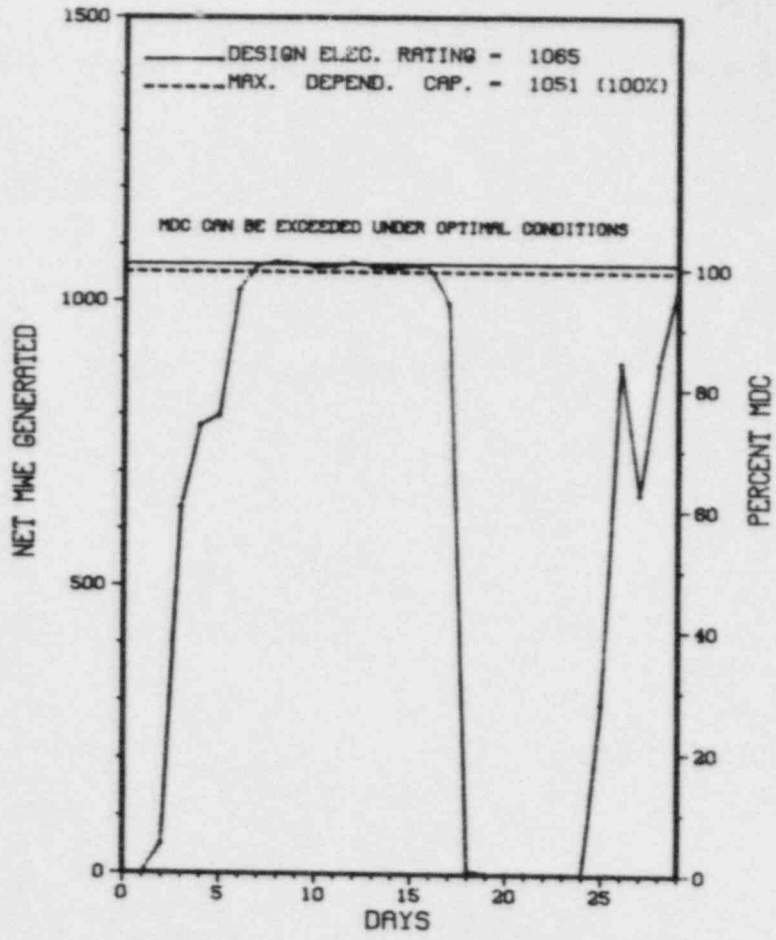
REFUELING & PIPE MAINTENANCE: 04/27/84

27. If Currently Shutdown Estimated Startup Date: N/A

****
* PEACH BOTTOM 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PEACH BOTTOM 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

PEACH BOTTOM 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	01/28/84	F	41.7	A	4		SA	VESSEL	SHUTDOWN CONTINUED DUE TO EXCESSIVE LEAKAGE THROUGH DRYWELL AIRLOCK DURING STARTUP PRESSURIZATION TEST.
2	02/18/84	S	173.0	B	1		CD	VALVEX	TEST ON MAIN STEAM ISOLATION VALVES AND FEEDWATER CHECK VALVES.
3	02/27/84	S	0.0	H	5		RC	ZZZZZZ	CONTROL ROD PATTERN ADJUSTMENT.

SUMMARY

PEACH BOTTOM 2 RETURNED ONLINE FROM A REPAIR OUTAGE ON FEBRUARY 2ND AND OPERATED ROUTINELY WITH 1 OUTAGE AND 1 REDUCTION DURING THE REMAINDER OF FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PEACH BOTTOM 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....YORK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI S OF
LANCASTER, PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 16, 1973

DATE ELEC ENER 1ST GENER...FEBRUARY 18, 1974

DATE COMMERCIAL OPERATE....JULY 5, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PHILADELPHIA ELECTRIC

CORPORATE ADDRESS.....2301 MARKET STREET
PHILADELPHIA, PENNSYLVANIA 19105

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....A. BLOUGH

LICENSING PROJ MANAGER.....G. GEARS
DOCKET NUMBER.....50-277

LICENSE & DATE ISSUANCE....DPR-44, DECEMBER 14, 1973

PUBLIC DOCUMENT ROOM.....GOVERNMENT PUBLICATIONS SECTION
STATE LIBRARY OF PENNSYLVANIA
FORUM BUILDING
COMMONWEALTH AND WALNUT STREET
HARRISBURG, PENNSYLVANIA 17105

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPU ROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-278 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: W. M. Alden (215) 841-5022

4. Licensed Thermal Power (MWt): 3293

5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152

6. Design Electrical Rating (Net MWe): 1065

7. Maximum Dependable Capacity (Gross MWe): 1098

8. Maximum Dependable Capacity (Net MWe): 1035

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	80,544.0
13. Hours Reactor Critical	677.4	1,137.3	57,937.4
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	661.4	1,105.5	56,421.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,079,002	3,459,338	164,497,643
18. Gross Elec Ener (MWH)	689,900	1,144,170	53,959,290
19. Net Elec Ener (MWH)	670,622	1,110,918	51,774,703
20. Unit Service Factor	95.0	76.8	70.1
21. Unit Avail Factor	95.0	76.8	70.1
22. Unit Cap Factor (MDC Net)	93.1	74.5	62.1
23. Unit Cap Factor (DER Net)	90.5	72.4	60.4
24. Unit Forced Outage Rate	5.0	23.2	7.7
25. Forced Outage Hours	34.6	334.5	4,665.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

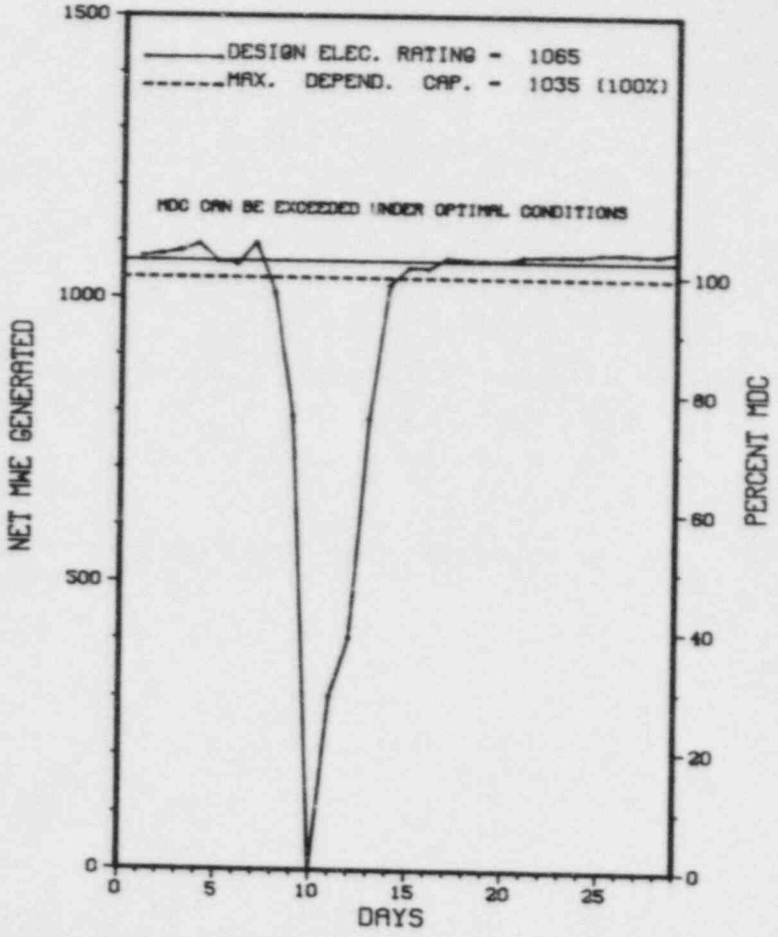
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* PEACH BOTTOM 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PEACH BOTTOM 3



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

PEACH BOTTOM 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2	02/09/84	F	34.6	A	3		CB	PUMPXX	AUTOMATIC SCRAM CAUSED BY POWER SPIKE RESULTING FROM PRESSURE SURGE ASSOCIATED WITH THE MALFUNCTIONING CONTROL CLOSING MAIN TURBINE VALVES.

SUMMARY

PEACH BOTTOM 3 OPERATED ROUTINELY WITH 1 OUTAGE FOR EQUIPMENT REPAIR DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* PEACH BOTTOM 3 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....YORK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI S OF
LANCASTER, PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...AUGUST 7, 1974

DATE ELEC ENER 1ST GENER...SEPTEMBER 1, 1974

DATE COMMERCIAL OPERATE....DECEMBER 23, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PHILADELPHIA ELECTRIC

CORPORATE ADDRESS.....2301 MARKET STREET
PHILADELPHIA, PENNSYLVANIA 19105

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....A. BLOUGH

LICENSING PROJ MANAGER.....G. GEARS
DOCKET NUMBER.....50-278

LICENSE & DATE ISSUANCE....DPR-56, JULY 2, 1974

PUBLIC DOCUMENT ROOM.....GOVERNMENT PUBLICATIONS SECTION
STATE LIBRARY OF PENNSYLVANIA
FORUM BUILDING
COMMONWEALTH AND WALNUT STREET
HARRISBURG, PENNSYLVANIA 17105

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-293 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: P. HAMILTON (617) 746-7905

4. Licensed Thermal Power (MWt): 1998

5. Nameplate Rating (Gross MWe): 780 X 0.87 = 678

6. Design Electrical Rating (Net MWe): 655

7. Maximum Dependable Capacity (Gross MWe): 690

8. Maximum Dependable Capacity (Net MWe): 670

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	98,400.0
13. Hours Reactor Critical	.0	.0	69,733.9
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	67,521.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	116,932,632
18. Gross Elec Ener (MWH)	0	0	39,228,314
19. Net Elec Ener (MWH)	0	0	37,693,409
20. Unit Service Factor	.0	.0	68.6
21. Unit Avail Factor	.0	.0	68.6
22. Unit Cap Factor (MDC Net)	.0	.0	57.2
23. Unit Cap Factor (DER Net)	.0	.0	58.5
24. Unit Forced Outage Rate	.0	.0	9.2
25. Forced Outage Hours	.0	.0	6,842.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

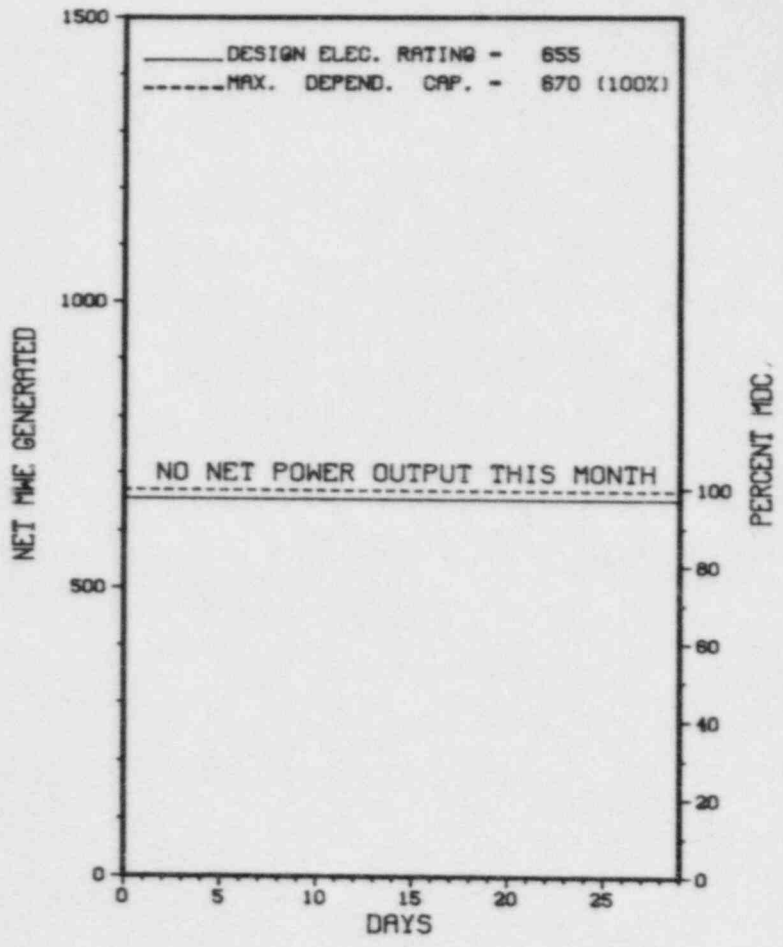
NONE

27. If Currently Shutdown Estimated Startup Date: 09/09/84

****
* PILGRIM 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PILGRIM 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

PILGRIM 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
16	12/10/83	S	695.0	C	4				SHUTDOWN FOR REFUELING AND RECIRCULATION PIPE REPAIR CONTINUES.

SUMMARY

PILGRIM 1 REMAINS SHUTDOWN IN A CONTINUING REFUELING/REPAIR OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PILGRIM 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MASSACHUSETTS

COUNTY.....PLYMOUTH

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...4 MI SE OF
PLYMOUTH, MASS

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JUNE 16, 1972

DATE ELEC ENER 1ST GENER...JULY 19, 1972

DATE COMMERCIAL OPERATE....DECEMBER 1, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CAPE COD BAY

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....BOSTON EDISON

CORPORATE ADDRESS.....800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....J. JOHNSON

LICENSING PROJ MANAGER.....P. LEECH
DOCKET NUMBER.....50-293

LICENSE & DATE ISSUANCE....DPR-35, SEPTEMBER 15, 1972

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11 NORTH STREET
PLYMOUTH, MASSACHUSETTS 02360

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-266 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: C.W. FAY (414) 277-2811

4. Licensed Thermal Power (MWt): 1518

5. Nameplate Rating (Gross MWe): 582 X 0.9 = 524

6. Design Electrical Rating (Net MWe): 497

7. Maximum Dependable Capacity (Gross MWe): 519

8. Maximum Dependable Capacity (Net MWe): 485

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	116,736.0
13. Hours Reactor Critical	.0	.0	94,078.5
14. Rx Reserve Shtdwn Hrs	.0	.0	625.4
15. Hrs Generator On-Line	.0	.0	91,607.5
16. Unit Reserve Shtdwn Hrs	.0	.0	793.5
17. Gross Therm Ener (MWH)	0	0	123,535,312
18. Gross Elec Ener (MWH)	0	0	41,395,980
19. Net Elec Ener (MWH)	0	0	39,367,882
20. Unit Service Factor	.0	.0	78.5
21. Unit Avail Factor	.0	.0	79.2
22. Unit Cap Factor (MDC Net)	.0	.0	69.0*
23. Unit Cap Factor (DER Net)	.0	.0	67.9
24. Unit Forced Outage Rate	.0	.0	2.7
25. Forced Outage Hours	.0	.0	2,406.3

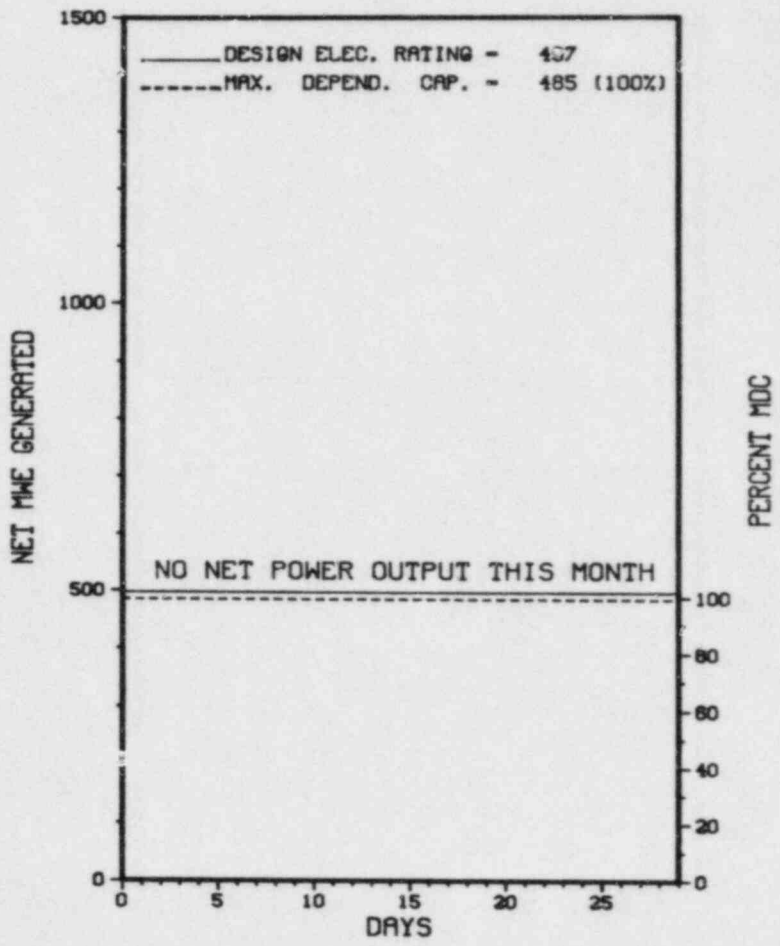
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: 03/19/84

****
* POINT BEACH 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

POINT BEACH 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
3	10/01/83	S	696.0	C	4		ZZ	ZZZZZZ	CONTINUATION OF 26-WEEK REFUELING AND STEAM GENERATOR REPLACEMENT OUTAGE.

SUMMARY

POINT BEACH 1 REMAINS SHUTDOWN IN A CONTINUING REFUELING AND STEAM GENERATOR REPLACEMENT OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* POINT BEACH 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....MANITOWOC

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...15 MI N OF
MANITOWOC, WISC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 2, 1970

DATE ELEC ENER 1ST GENER...NOVEMBER 6, 1970

DATE COMMERCIAL OPERATE....DECEMBER 21, 1970

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....WISCONSIN ELECTRIC POWER COMPANY

CORPORATE ADDRESS.....231 WEST MICHIGAN STREET
MILWAUKEE, WISCONSIN 53201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....R. HAGUE

LICENSING PROJ MANAGER.....T. COLBURN
DOCKET NUMBER.....50-266

LICENSE & DATE ISSUANCE....DPR-24, OCTOBER 5, 1970

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TWO RIVERS, WISCONSIN 54241

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON OCTOBER 11-14, 25-28, NOVEMBER 1-4 & 16, JANUARY 11, 1984 AND MANAGEMENT MEETING ON JANUARY 4 (83-21). SPECIAL ANNOUNCED INSPECTION BY REGIONAL INSPECTORS OF QA PROGRAM ADMINISTRATION, MAINTENANCE PROGRAM AND IMPLEMENTATION, DESIGN CHANGE AND MODIFICATION PROGRAM AND IMPLEMENTATION, PROCUREMENT, OFFSITE REVIEW COMMITTEE, DOCUMENT CONTROL, CALIBRATION AND CONTROL OF MEASURING AND TEST EQUIPMENT, SURVEILLANCE AND INSERVICE TESTING, CLEANLINESS CONTROL, AUDIT PROGRAM, STEAM GENERATOR REPLACEMENT PROGRAM. THE INSPECTION INVOLVED 269 INSPECTOR-HOURS ONSITE BY FOUR INSPECTORS, INCLUDING 66 INSPECTOR-HOURS AT CORPORATE HEADQUARTERS BY FOUR INSPECTORS. A MANAGEMENT MEETING WAS HELD WHICH INVOLVED 48 STAFF-HOURS. OF THE 14 AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; NINE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THE REMAINING NINE AREAS (FAILURE TO MAINTAIN CLEANLINESS CONTROL, FAILURE TO CONTROL DOCUMENTS, FAILURE TO PROPERLY STORE RECORDS, FAILURE TO CONTROL STORED ITEMS, FAILURE TO PROPERLY CONDUCT AN AUDIT PROGRAM, FAILURE TO PERFORM 10 CFR 50.59 REVIEWS, AND FAILURE TO PROVIDE APPROPRIATE TRAINING.

SAFETY EVALUATION REPORT, DECEMBER - JANUARY (83-25): ROUTINE ANNOUNCED SAFETY EVALUATION REPORT OF SITE EMERGENCY PLAN OF NOVEMBER 11, 1983. THE INSPECTION INVOLVED 128 INSPECTOR-HOURS BY TWO NRC INSPECTORS. THE PLAN APPEARS TO MEET THE PLANNING STANDARDS OF 10 CFR PART 50.47 (B) AND THE REQUIREMENTS OF 10 CFR PART 50, APPENDIX E; HOWEVER, CLARIFICATIONS ARE NEEDED TO CLOSE OUT SECTIONS A-D, G-J, L, AND N. ONE DEVIATION FROM A COMMITMENT WAS IDENTIFIED.

INSPECTION ON DECEMBER 1, TO JANUARY 31, (83-26): ROUTINE, UNANNOUNCED INSPECTION BY RESIDENT INSPECTORS OF RECEIPT OF NEW FUEL; OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; IE BULLETINS; INDEPENDENT INSPECTION; REGIONAL REQUESTS; TMI ACTION ITEMS; STEAM GENERATOR REPLACEMENT ACTIVITIES. THE INSPECTION INVOLVED A TOTAL OF 434 INSPECTOR-HOURS ONSITE BY TWO INSPECTORS INCLUDING 120

INSPECTION SUMMARY

HOURS ON OFFSHIFTS. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN EIGHT AREAS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE REMAINING AREA (FAILURE TO FOLLOW PROCEDURES).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS SHUTDOWN FOR REFUELING AND STEAM GENERATOR REPLACEMENT OUTAGE.

LAST IE SITE INSPECTION DATE: DECEMBER 1 THROUGH JANUARY 31, 1984

INSPECTION REPORT NO: 83-26

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-36 03L-0	12/31/83	01/30/84	DURING SURV. TEST, 1 PRESSURIZER PRESSURE TRANSMITTER ACTUATED ABOVE REACTOR TRIP SETPOINT REQUIRED BY T.S.
84-01	01/03/84	02/02/84	REACTOR TRIP.

1. Docket: 50-301 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: C.W. FAY (414) 277-2811
4. Licensed Thermal Power (MWt): 1518
5. Nameplate Rating (Gross MWe): 582 X 0.9 = 524
6. Design Electrical Rating (Net MWe): 497
7. Maximum Dependable Capacity (Gross MWe): 519
8. Maximum Dependable Capacity (Net MWe): 495
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	101,521.0
13. Hours Reactor Critical	696.0	1,440.0	89,868.2
14. Rx Reserve Shtdwn Hrs	.0	.0	198.3
15. Hrs Generator On-Line	696.0	1,440.0	88,342.8
16. Unit Reserve Shtdwn Hrs	.0	.0	182.7
17. Gross Therm Ener (MWH)	1,052,908	2,162,660	123,057,437
18. Gross Elec Ener (MWH)	355,470	729,650	41,689,480
19. Net Elec Ener (MWH)	340,482	698,630	39,703,895
20. Unit Service Factor	100.0	100.0	87.0
21. Unit Avail Factor	100.0	100.0	87.2
22. Unit Cap Factor (MDC Net)	98.8	98.0	79.5*
23. Unit Cap Factor (DER Net)	98.4	97.6	78.7
24. Unit Forced Outage Rate	.0	.0	1.4
25. Forced Outage Hours	.0	.0	692.2

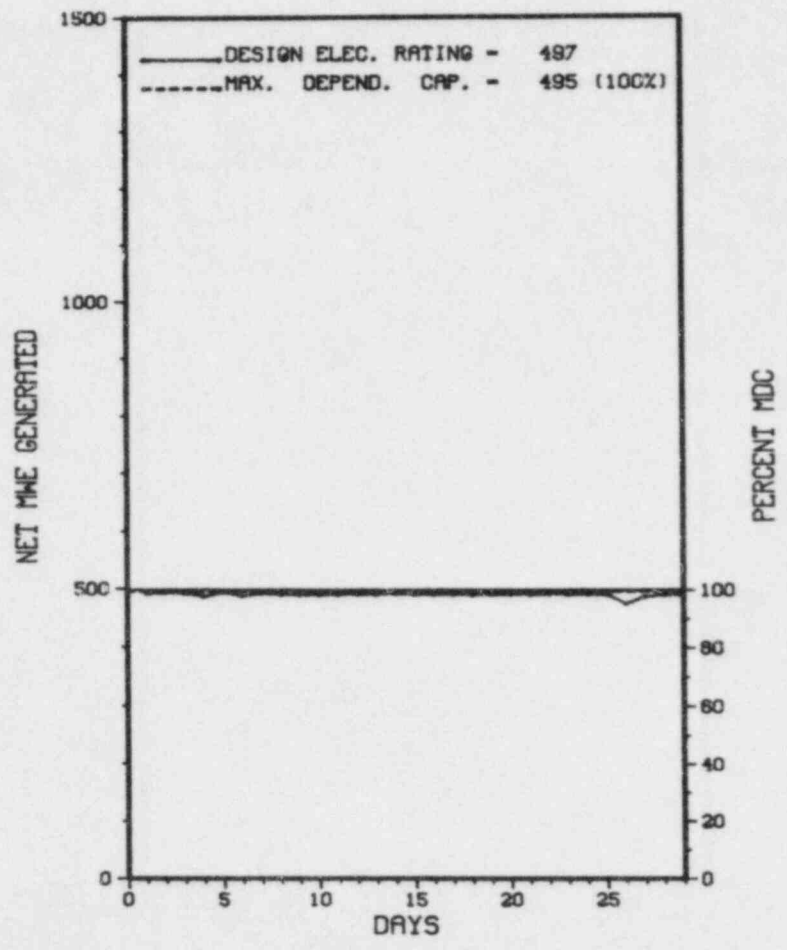
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* POINT BEACH 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

POINT BEACH 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

POINT BEACH 2 OPERATED AT FULL POWER WITH NO OUTAGES DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* POINT BEACH 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....MANITOWOC

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...15 MI N OF
MANITOWOC, WISC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 30, 1972

DATE ELEC ENER 1ST GENER...AUGUST 2, 1972

DATE COMMERCIAL OPERATE....OCTOBER 1, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....WISCONSIN ELECTRIC POWER COMPANY

CORPORATE ADDRESS.....231 WEST MICHIGAN STREET
MILWAUKEE, WISCONSIN 53201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....R. HAGUE

LICENSING PROJ MANAGER.....T. COLBURN
DOCKET NUMBER.....50-301

LICENSE & DATE ISSUANCE....DPR-27, MARCH 8, 1973

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TWO RIVERS, WISCONSIN 54241

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON OCTOBER 11-14, 25-28, NOVEMBER 1-4 & 16, JANUARY 11, 1984 AND MANAGEMENT MEETING ON JANUARY 4 (83-20). SPECIAL ANNOUNCED INSPECTION BY REGIONAL INSPECTORS OF QA PROGRAM ADMINISTRATION, MAINTENANCE PROGRAM AND IMPLEMENTATION, DESIGN CHANGE AND MODIFICATION PROGRAM AND IMPLEMENTATION, PROCUREMENT, OFFSITE REVIEW COMMITTEE, DOCUMENT CONTROL, CALIBRATION AND CONTROL OF MEASURING AND TEST EQUIPMENT, SURVEILLANCE AND INSERVICE TESTING, CLEANLINESS CONTROL, AUDIT PROGRAM, STEAM GENERATOR REPLACEMENT PROGRAM. THE INSPECTION INVOLVED 269 INSPECTOR-HOURS ONSITE BY FOUR INSPECTORS, INCLUDING 66 INSPECTOR-HOURS AT CORPORATE HEADQUARTERS BY FOUR INSPECTORS. A MANAGEMENT MEETING WAS HELD WHICH INVOLVED 48 STAFF-HOURS. OF THE 14 AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; NINE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THE REMAINING NINE AREAS (FAILURE TO MAINTAIN CLEANLINESS CONTROL, FAILURE TO CONTROL DOCUMENTS, FAILURE TO PROPERLY STORE RECORDS, FAILURE TO CONTROL STORED ITEMS, FAILURE TO PROPERLY CONDUCT AN AUDIT PROGRAM, FAILURE TO PERFORM 10 CFR 50.59 REVIEWS, AND FAILURE TO PROVIDE APPROPRIATE TRAINING.

SAFETY EVALUATION REPORT, DECEMBER - JANUARY (83-23): ROUTINE ANNOUNCED SAFETY EVALUATION REPORT OF SITE EMERGENCY PLAN OF NOVEMBER 11, 1983. THE INSPECTION INVOLVED 128 INSPECTOR-HOURS BY TWO NRC INSPECTORS. THE PLAN APPEARS TO MEET THE PLANNING STANDARDS OF 10 CFR PART 50.47 (B) AND THE REQUIREMENTS OF 10 CFR PART 50, APPENDIX E; HOWEVER, CLARIFICATIONS ARE NEEDED TO CLOSE OUT SECTIONS A-D, G-J, L, AND N. ONE DEVIATION FROM A COMMITMENT WAS IDENTIFIED.

INSPECTION ON DECEMBER 1, TO JANUARY 31, (83-24): ROUTINE, UNANNOUNCED INSPECTION BY RESIDENT INSPECTORS OF RECEIPT OF NEW FUEL; OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; IE BULLETINS; INDEPENDENT INSPECTION; REGIONAL REQUESTS; TMI ACTION ITEMS; STEAM GENERATOR REPLACEMENT ACTIVITIES. THE INSPECTION INVOLVED A TOTAL OF 434 INSPECTOR-HOURS ONSITE BY TWO INSPECTORS INCLUDING 120

INSPECTION SUMMARY

HOURS ON OFFSHIFTS. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN EIGHT AREAS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE REMAINING AREA (FAILURE TO FOLLOW PROCEDURES).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: DECEMBER 1 THROUGH JANUARY 31, 1984

INSPECTION REPORT NO: 83-24

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-282 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: DALE DUGSTAD (612) 388-1121

4. Licensed Thermal Power (MWt): 1650

5. Nameplate Rating (Gross MWe): 659 X 0.9 = 593

6. Design Electrical Rating (Net MWe): 530

7. Maximum Dependable Capacity (Gross MWe): 534

8. Maximum Dependable Capacity (Net MWe): 503

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	89,472.0
13. Hours Reactor Critical	696.0	1,390.4	73,063.4
14. Rx Reserve Shtdwn Hrs	.0	.0	5,571.1
15. Hrs Generator On-Line	696.0	1,369.0	71,750.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,104,603	2,158,322	112,469,484
18. Gross Elec Ener (MWH)	371,370	723,560	36,603,360
19. Net Elec Ener (MWH)	352,843	686,003	34,277,432
20. Unit Service Factor	100.0	95.1	80.2
21. Unit Avail Factor	100.0	95.1	80.2
22. Unit Cap Factor (MDC Net)	100.8	94.7	76.2
23. Unit Cap Factor (DER Net)	95.7	89.9	72.3
24. Unit Forced Outage Rate	.0	.0	8.4
25. Forced Outage Hours	.0	.0	2,920.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

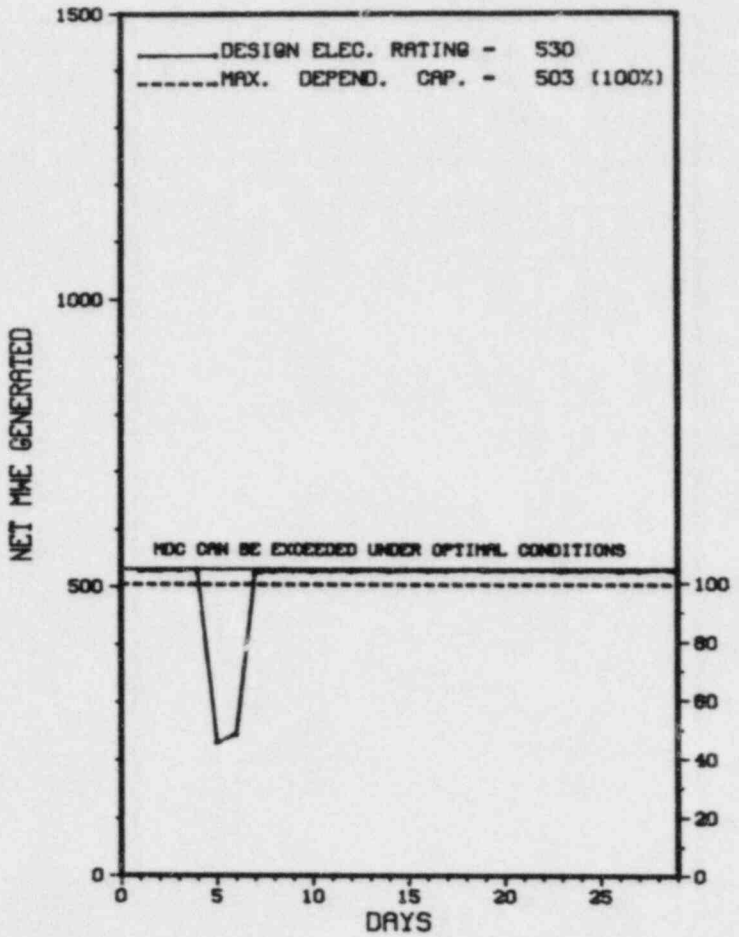
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* PRAIRIE ISLAND 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PRAIRIE ISLAND 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

PRAIRIE ISLAND 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
	02/05/84	S	0.0	B	5				TURBINE VALVES TEST, I OUT OF BAND.

SUMMARY

PRAIRIE ISLAND 1 OPERATED AT FULL POWER WITH 1 REDUCTION DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training	5-Reduced Load	Licensee Event Report
	& License Examination	9-Other	(LER) File (NUREG-0161)

****
* PRAIRIE ISLAND 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....MINNESOTA

COUNTY.....GOODHUE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...28 MI SE OF
MINNEAPOLIS, MINN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 1, 1973

DATE ELEC ENER 1ST GENER...DECEMBER 4, 1973

DATE COMMERCIAL OPERATE....DECEMBER 16, 1973

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHERN STATES POWER

CORPORATE ADDRESS.....414 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

CONTRACTOR
ARCHITECT/ENGINEER.....FLUOR PIONEER, INC.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....NORTHERN STATES POWER COMPANY

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. HARD

LICENSING PROJ MANAGER.....D. DIIANNI
DOCKET NUMBER.....50-282

LICENSE & DATE ISSUANCE....DPR-42, APRIL 5, 1974

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300 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JANUARY 17-20, (84-01): ROUTINE UNANNOUNCED INSPECTION OF RADIOACTIVE WASTE SYSTEMS, INCLUDING: SOLID WASTE PROCESSING AND STORAGE; DISPOSAL OF SOLID LOW-LEVEL WASTES; LIQUID EFFLUENTS, LIQUID EFFLUENT INSTRUMENTATION, REACTOR COOLANT WATER QUALITY; GASEOUS EFFLUENTS; GASEOUS EFFLUENT INSTRUMENTATION; AIR CLEANING SYSTEMS; AUDITS AND APPRAISALS; AND AUDITS OF TRANSPORTATION. THE INSPECTION INVOLVED 60 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

OTHER ITEMS

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: JANUARY 17-20, 1984

INSPECTION REPORT NO: 84-01

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-306 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: DALE DUGSTAD (612) 388-1121

4. Licensed Thermal Power (MWt): 1650

5. Nameplate Rating (Gross MWe): 659 X 0.9 = 593

6. Design Electrical Rating (Net MWe): 530

7. Maximum Dependable Capacity (Gross MWe): 531

8. Maximum Dependable Capacity (Net MWe): 500

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	80,590.0
13. Hours Reactor Critical	696.0	1,440.0	69,690.3
14. Rx Reserve Shtdwn Hrs	.0	.0	1,516.1
15. Hrs Generator On-Line	696.0	1,440.0	68,733.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,130,849	2,324,880	108,056,738
18. Gross Elec Ener (MWH)	380,160	782,020	34,889,420
19. Net Elec Ener (MWH)	362,593	745,601	32,720,484
20. Unit Service Factor	100.0	100.0	85.3
21. Unit Avail Factor	100.0	100.0	85.3
22. Unit Cap Factor (MDC Net)	104.2	103.6	81.2
23. Unit Cap Factor (DER Net)	98.3	97.7	76.6
24. Unit Forced Outage Rate	.0	.0	4.4
25. Forced Outage Hours	.0	.0	3,315.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

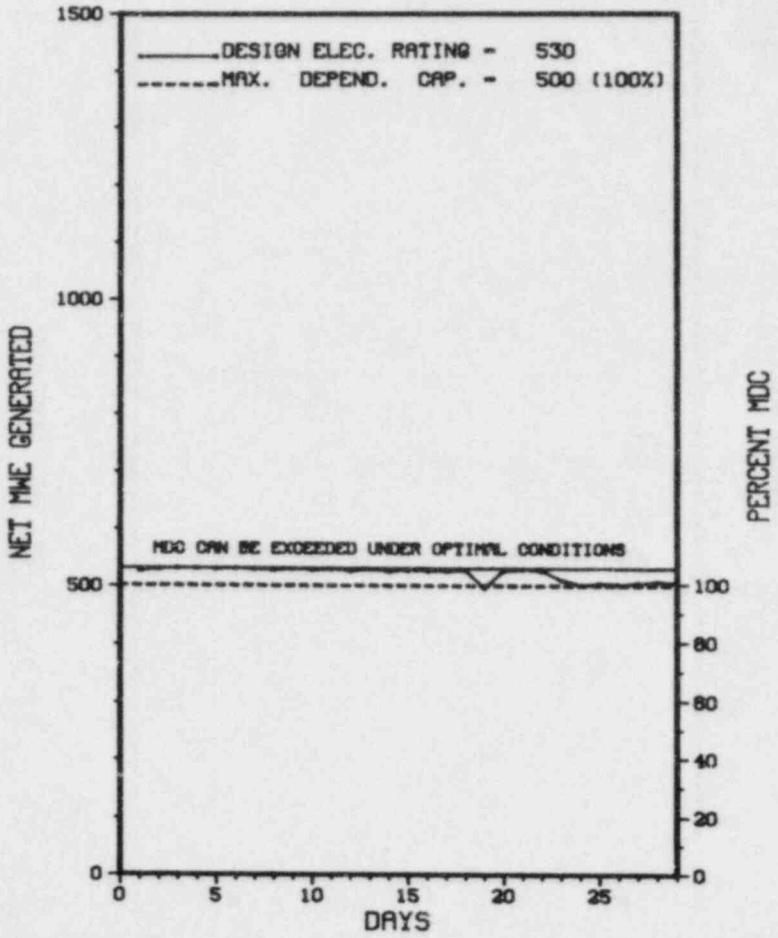
REFUELING OUTAGE IN AUGUST OF 1984.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* PRAIRIE ISLAND 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PRAIRIE ISLAND 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

PRAIRIE ISLAND 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
	02/19/84	S	0.0	B	5				TURBINE VALVES TEST.
	02/23/84	S	0.0	F	5				POWER REDUCED TO 96% DUE TO F DELTA H APPROACHING TECH SPEC LIMIT. THIS WAS ANTICIPATED IN CORE LOAD DESIGN.

SUMMARY

PRAIRIE ISLAND 2 OPERATED AT FULL POWER WITH 2 REDUCTIONS DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PRAIRIE ISLAND 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MINNESOTA

COUNTY.....GOODHUE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...28 MI SE OF
MINNEAPOLIS, MINN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 17, 1974

DATE ELEC ENER 1ST GENER...DECEMBER 21, 1974

DATE COMMERCIAL OPERATE....DECEMBER 21, 1974

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHERN STATES POWER

CORPORATE ADDRESS.....414 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

CONTRACTOR
ARCHITECT/ENGINEER.....FLUOR PIONEER, INC.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....NORTHERN STATES POWER COMPANY

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. HARD

LICENSING PROJ MANAGER.....D. DIIANNI
DOCKET NUMBER.....50-306

LICENSE & DATE ISSUANCE....DPR-60, OCTOBER 29, 1974

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MINNEAPOLIS PUBLIC LIBRARY
300 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JANUARY 17-20, (84-01): ROUTINE UNANNOUNCED INSPECTION OF RADIOACTIVE WASTE SYSTEMS, INCLUDING: SOLID WASTE PROCESSING AND STORAGE; DISPOSAL OF SOLID LOW-LEVEL WASTES; LIQUID EFFLUENTS, LIQUID EFFLUENT INSTRUMENTATION, REACTOR COOLANT WATER QUALITY; GASEOUS EFFLUENTS; GASEOUS EFFLUENT INSTRUMENTATION; AIR CLEANING SYSTEMS; AUDITS AND APPRAISALS; AND AUDITS OF TRANSPORTATION. THE INSPECTION INVOLVED 60 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

OTHER ITEMS

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: JANUARY 17-20, 1984

INSPECTION REPORT NO: 84-01

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-254 — O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: ALEX MISAK (309) 654-2241 X194

4. Licensed Thermal Power (MWt): 2511

5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828

6. Design Electrical Rating (Net MWe): 789

7. Maximum Dependable Capacity (Gross MWe): 813

8. Maximum Dependable Capacity (Net MWe): 769

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	103,464.0
13. Hours Reactor Critical	696.0	1,440.0	84,995.6
14. Rx Reserve Shtdwn Hrs	.0	.0	3,421.9
15. Hrs Generator On-Line	696.0	1,440.0	81,787.1
16. Unit Reserve Shtdwn Hrs	.0	.0	909.2
17. Gross Therm Ener (MWH)	1,643,681	3,406,884	168,513,590
18. Gross Elec Ener (MWH)	545,050	1,129,512	54,388,240
19. Net Elec Ener (MWH)	517,997	1,073,371	50,678,631
20. Unit Service Factor	100.0	100.0	79.0
21. Unit Avail Factor	100.0	100.0	79.9
22. Unit Cap Factor (MDC Net)	96.8	96.9	63.7
23. Unit Cap Factor (DER Net)	94.3	94.5	62.1
24. Unit Forced Outage Rate	.0	.0	5.9
25. Forced Outage Hours	.0	.0	2,728.0

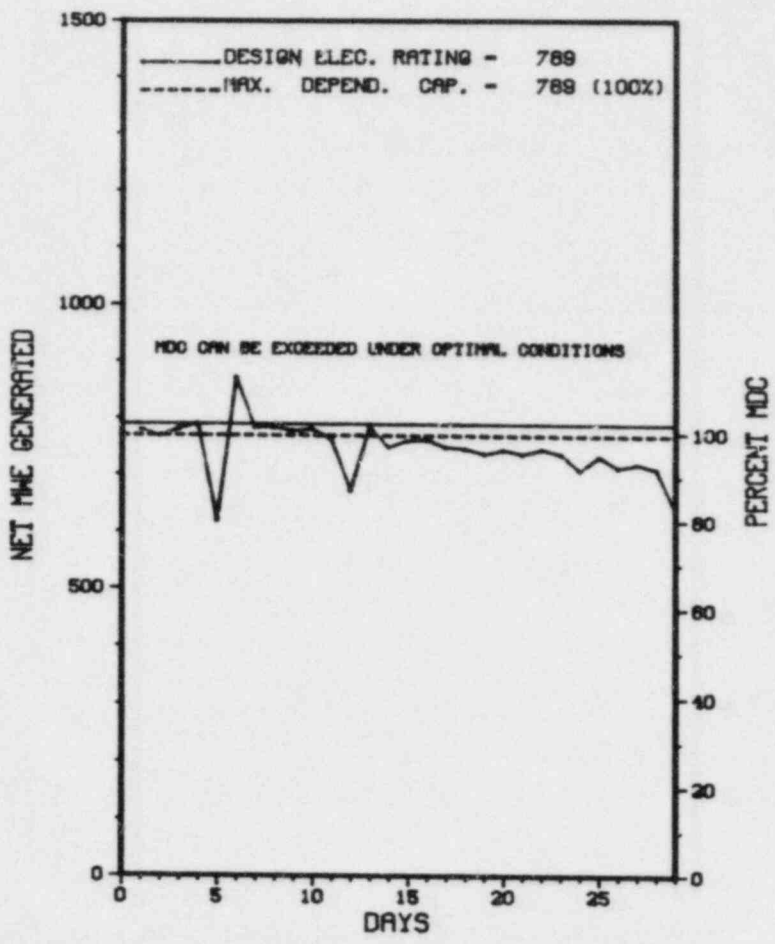
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* QUAD CITIES 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

QUAD CITIES 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-6	02/05/84	S	0.0	B	5		HA	XXXXXX	REDUCED LOAD TO PERFORM WEEKLY TURBINE TESTS.
84-7	02/12/84	S	0.0	B	5		RB	CONROD	REDUCED LOAD FOR CONTROL ROD PATTERN ADJUSTMENTS.
84-8	02/19/84	S	0.0	B	5		HA	XXXXXX	REDUCED LOAD TO PERFORM WEEKLY TURBINE TESTS.
84-9	02/26/84	S	0.0	B	5		HA	XXXXXX	REDUCED LOAD TO PERFORM WEEKLY TURBINE TESTS.
84-10	02/28/84	S	0.0	B	5		CB	INSTRU	REDUCED LOAD IN PREPARATION FOR AN ECONOMIC GENERATION CONTROL SYSTEM TEST.
84-11	02/29/84	S	0.0	B	5		CB	INSTRU	REDUCED LOAD IN PREPARATION FOR AN ECONOMIC GENERATION CONTROL SYSTEM TEST.

SUMMARY

QUAD CITIES 1 OPERATED WITH 6 REDUCTIONS DURING THE REPORT PERIOD.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* QUAD CITIES 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....ROCK ISLAND

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI NE OF
MOLINE, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...OCTOBER 18, 1971

DATE ELEC ENER 1ST GENER...APRIL 12, 1972

DATE COMMERCIAL OPERATE....FEBRUARY 18, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....A. MADISON

LICENSING PROJ MANAGER.....R. BEVAN
DOCKET NUMBER.....50-254

LICENSE & DATE ISSUANCE....DPR-29, DECEMBER 14, 1972

PUBLIC DOCUMENT ROOM.....MOLINE PUBLIC LIBRARY
504 17TH STREET
MOLINE, ILLINOIS 61265

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON DECEMBER 5, THROUGH JANUARY 31, (83-30): ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTORS OF PREVIOUS INSPECTION FINDINGS; OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; LICENSEE EVENT REPORTS; IE BULLETIN FOLLOWUP; REACTOR SCRAMS; DESIGN CHANGES AND MODIFICATIONS; REVIEW OF LICENSEE'S MONTHLY PERFORMANCE REPORT; PROCEDURES; REFUELING; REGIONAL REQUESTS; ACRS MEETING; SITE VISIT; AND INFORMATION NOTICES. THE INSPECTION INVOLVED A TOTAL OF 454 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS, INCLUDING 61 REGIONAL STAFF HOURS AND 22 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

SPECIAL INSPECTION ON FEBRUARY 3 - 14,(84-01): INSPECTION FOLLOWUP ON COMMONWEALTH EDISON COMPANY REMARKS TO SPECIAL INSPECTION REPORT CONCERNING ALLEGATIONS OF IMPROPER OPERATION AT DRESDEN, QUAD CITIES, AND ZION NUCLEAR POWER PLANTS. THE INSPECTION INVOLVED 14 INSPECTION-HOURS BY THREE NRC INSPECTORS. MEASURES TO CORRECT IDENTIFIED WEAKNESSES WERE TAKEN AS DESCRIBED IN THE REPONSE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS IN COASTDOWN ANTICIPATING SHUTDOWN FOR A 5 MONTH OUTAGE ON 3/11/84.

LAST IE SITE INSPECTION DATE: JANUARY 18-31, 1984

INSPECTION REPORT NO: 84-01

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-265 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: ALEX MISAK (309) 654-2241 X194
4. Licensed Thermal Power (MWt): 2511
5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828
6. Design Electrical Rating (Net MWe): 789
7. Maximum Dependable Capacity (Gross MWe): 813
8. Maximum Dependable Capacity (Net MWe): 769
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	102,574.0
13. Hours Reactor Critical	245.5	245.5	78,163.1
14. Rx Reserve Shtdwn Hrs	.0	.0	2,985.8
15. Hrs Generator On-Line	160.3	160.3	75,370.1
16. Unit Reserve Shtdwn Hrs	.0	.0	702.9
17. Gross Therm Ener (MWH)	193,783	193,783	155,575,871
18. Gross Elec Ener (MWH)	58,218	58,218	49,493,976
19. Net Elec Ener (MWH)	53,862	53,338	46,388,212
20. Unit Service Factor	23.0	11.1	73.5
21. Unit Avail Factor	23.0	11.1	74.2
22. Unit Cap Factor (MDC Net)	10.1	4.8	58.8
23. Unit Cap Factor (DER Net)	9.8	4.7	57.3
24. Unit Forced Outage Rate	40.2	40.2	8.7
25. Forced Outage Hours	107.7	107.7	3,297.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

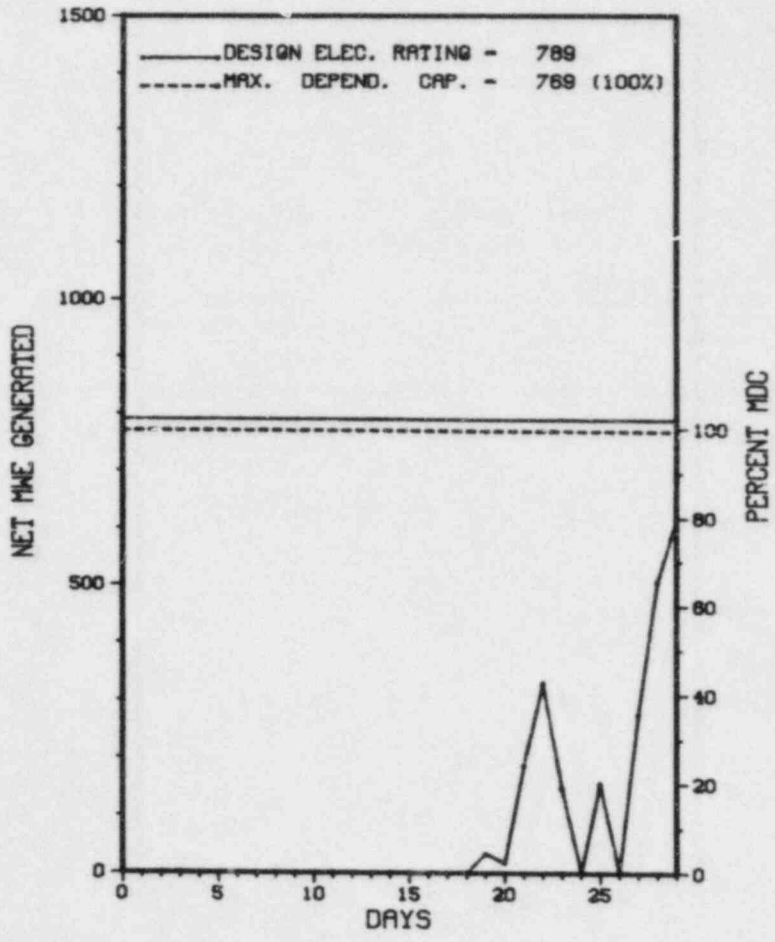
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* QUAD CITIES 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

QUAD CITIES 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* QUAD CITIES 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
83-66	09/04/83	S	428.0	C	4		RC	FUELXX	UNIT TWO REMAINS SHUTDOWN FOR END OF CYCLE SIX REFUELING AND MAINTENANCE.
84-1	02/18/84	F	7.2	B	1		HA	ZZZZZZ	UNIT TWO PLACED IN HOT STANDBY DUE TO HIGH TURBINE VIBRATION.
84-2	02/19/84	F	32.3	B	1		ZZ	VALVEX	UNIT TWO SHUTDOWN TO REPAIR MISCELLANEOUS VALVE PACKING LEAKS.
84-3	02/23/84	F	36.1	B	1		HA	ZZZZZZ	UNIT TWO PLACED IN HOT STANDBY DUE TO HIGH TURBINE VIBRATION.
84-4	02/25/84	F	32.1	B	1		CB	MOTORX	UNIT TWO SHUTDOWN TO REPLACE THE MOTOR ON THE 'B' RECIRCULATION PUMP SUCTION VALVE.

****
* SUMMARY *
****

QUAD CITIES 2 RETURNED ONLINE FEBRUARY 19TH AND OPERATED ROUTINELY THE REMAINDER OF FEBRUARY.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H
Instructions for
Preparation of
Data Entry Sheet
Licensee Event Report
(LER) File (NUREG-0161)

****
* QUAD CITIES 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....ROCK ISLAND

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI NE OF
MOLINE, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...APRIL 26, 1972

DATE ELEC ENER 1ST GENER...MAY 23, 1972

DATE COMMERCIAL OPERATE....MARCH 10, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....A. MADISON

LICENSING PROJ MANAGER.....R. BEVAN
DOCKET NUMBER.....50-265

LICENSE & DATE ISSUANCE....DPR-30, DECEMBER 14, 1972

PUBLIC DOCUMENT ROOM.....MOLINE PUBLIC LIBRARY
504 17TH STREET
MOLINE, ILLINOIS 61265

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON DECEMBER 5, THROUGH JANUARY 31, (83-31): ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTORS OF PREVIOUS INSPECTION FINDINGS; OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; LICENSEE EVENT REPORTS; IE BULLETIN FOLLOWUP; REACTOR SCRAMS; DESIGN CHANGES AND MODIFICATIONS; REVIEW OF LICENSEE'S MONTHLY PERFORMANCE REPORT; PROCEDURES; REFUELING; REGIONAL REQUESTS; ACRS MEETING; SITE VISIT; AND INFORMATION NOTICES. THE INSPECTION INVOLVED A TOTAL OF 454 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS, INCLUDING 61 REGIONAL STAFF HOURS AND 22 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

SPECIAL INSPECTION ON FEBRUARY 3 - 14,(84-01): INSPECTION FOLLOWUP ON COMMONWEALTH EDISON COMPANY REMARKS TO SPECIAL INSPECTION REPORT CONCERNING ALLEGATIONS OF IMPROPER OPERATION AT DRESDEN, QUAD CITIES, AND ZION NUCLEAR POWER PLANTS. THE INSPECTION INVOLVED 14 INSPECTION-HOURS BY THREE NRC INSPECTORS. MEASURES TO CORRECT IDENTIFIED WEAKNESSES WERE TAKEN AS DESCRIBED IN THE REPONSE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

UNIT WAS RETURNED TO SERVICE ON 2/20/84 FOLLOWING EXTENDED REFUELING AND MAINTENANCE OUTAGE, AND IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: JANUARY 18-31, 1984

INSPECTION REPORT NO: 84-01

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-14/ 03L-0	09/06/83	01/30/84	DURING LLRT, MSIV FOUND TO LEAK 34.56 SCFH.
83-18/ 01T-1	10/11/83	02/02/84	LINEAR INDICATION FOUND DURING UT EXAM.
84-01	01/03/84	01/30/84	UNIT SCRAM WHILE 'A' RPS BUS WAS OUT OF SERVICE.
84-02	01/06/84	01/27/84	UNIT 2 REACTOR SCRAM ON LOSS OF 48 VOLT BATTERY SYSTEM.

1. Docket: 50-312 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: RON COLOMBO (916) 452-3211
4. Licensed Thermal Power (MWt): 2772
5. Nameplate Rating (Gross MWe): 1070 X 0.9 = 963
6. Design Electrical Rating (Net MWe): 918
7. Maximum Dependable Capacity (Gross MWe): 917
8. Maximum Dependable Capacity (Net MWe): 873
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	77,761.0
13. Hours Reactor Critical	689.8	1,433.8	45,785.4
14. Rx Reserve Shtdwn Hrs	.0	.0	9,313.8
15. Hrs Generator On-Line	689.8	1,433.8	43,976.0
16. Unit Reserve Shtdwn Hrs	.0	.0	1,210.2
17. Gross Therm Ener (MWH)	1,721,487	3,462,831	109,374,173
18. Gross Elec Ener (MWH)	577,349	1,163,572	36,559,644
19. Net Elec Ener (MWH)	547,078	1,096,211	34,470,535
20. Unit Service Factor	99.1	99.6	56.6
21. Unit Avail Factor	99.1	99.6	58.1
22. Unit Cap Factor (MDC Net)	90.0	87.2	50.8
23. Unit Cap Factor (DER Net)	85.6	82.9	48.3
24. Unit Forced Outage Rate	.9	.4	27.2
25. Forced Outage Hours	6.2	6.2	16,416.2

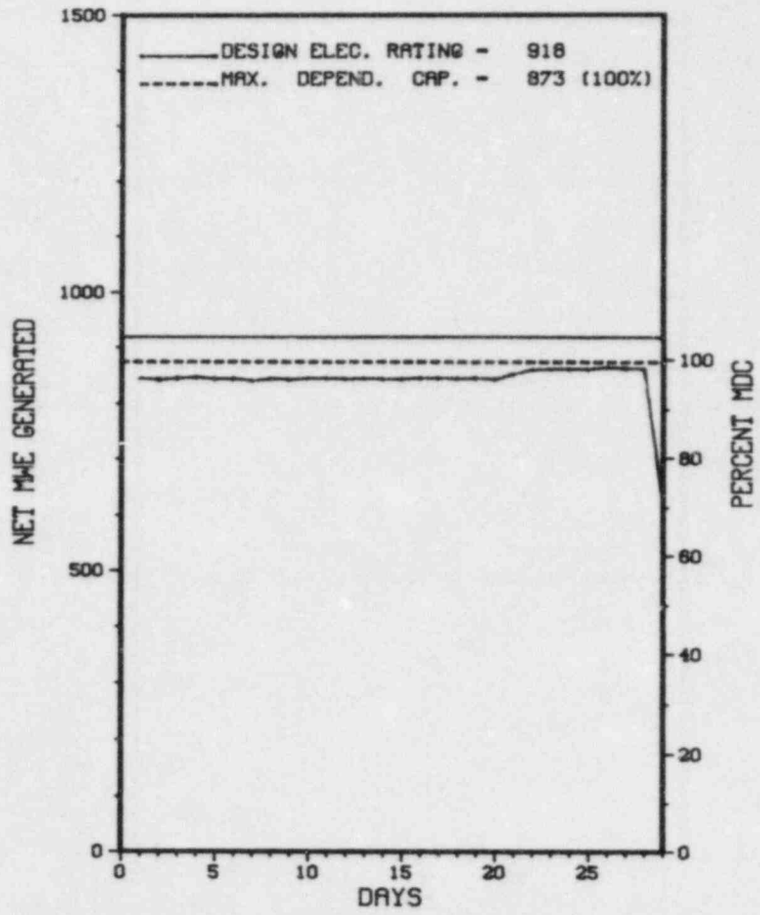
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: N/A

RANCHO SECO 1

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

RANCHO SECO 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
3	02/29/84	F	0.0	A	5		CB	PUMPXX	"A" REACTOR COOLANT PUMP UPPER AND LOWER L.O. RES. HI-LOW ALARM, REDUCED TO 65%.
4	02/29/84	F	6.2	A	3		EA	XXXXXX	PG&E LOSS OF TRANSMISSION LINES CAUSED LOW VOLTAGE FREQUENCY. PLANT COULD NOT HANDLE LOAD DEMAND ON 3 RCP OPERATION.

SUMMARY

RANCO SECO 1 OPERATED AT NEAR FULL POWER WITH 1 OUTAGE AND 1 REDUCTION DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* RANCHO SECO 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SACRAMENTO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI SE OF
SACRAMENTO, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...SEPTEMBER 16, 1974

DATE ELEC ENER 1ST GENER...OCTOBER 13, 1974

DATE COMMERCIAL OPERATE....APRIL 17, 1975

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....FOLSOM CANAL

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SACRAMENTO MUN. UTIL. DISTRICT

CORPORATE ADDRESS.....6201 S STREET P.O. BOX 15830
SACRAMENTO, CALIFORNIA 95813

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....H. CANTER

LICENSING PROJ MANAGER.....S. MINER
DOCKET NUMBER.....50-312

LICENSE & DATE ISSUANCE....DPR-54, AUGUST 16, 1974

PUBLIC DOCUMENT ROOM.....BUSINESS AND MUNICIPAL DEPARTMENT
SACRAMENTO CITY - COUNTY LIBRARY
828 I STREET
SACRAMENTO, CALIFORNIA 95814

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON NOVEMBER 7-18, 1983 (REPORT NO. 50-312/83-35) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON DECEMBER 5, 1983 - JANUARY 10, 1984 (REPORT NO. 50-312/83-36) AREAS INSPECTED: OPERATIONAL SAFETY VERIFICATION; MAINTENANCE OBSERVATIONS; SURVEILLANCE OBSERVATIONS; LER FOLLOWUP; FOLLOWUP ON REGIONAL REQUEST; ONSITE FOLLOWUP OF EVENTS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 154 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS.

RESULTS: OF THE SEVEN AREAS INSPECTED, THERE WERE TWO ITEMS OF NONCOMPLIANCE: ONE IN THE FAILURE TO FOLLOW THE TEMPORARY CHANGE PROCEDURE AND ONE FOR THE FAILURE TO COMPLY WITH SURVEILLANCE PROCEDURE SPECIFICATION FOR OPERABILITY OF THE PRESSURIZER LEVEL INSTRUMENTS.

+ INSPECTION ON DECEMBER 27-29, 1983 (REPORT NO. 50-312/83-37) AREAS INSPECTED: A SPECIAL INSPECTION BY A REGIONAL INSPECTOR OF ALLEGATIONS CONCERNING ELECTRICAL CABLE PULL CARD INADEQUACIES. THE INSPECTION INVOLVED 21 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: THE ALLEGATIONS WERE SUBSTANTIATED IN PART BY THIS INSPECTION.

+ INSPECTION ON JANUARY 16-20 AND 26, AND TELEPHONE CONVERSATIONS ON JANUARY 31 AND FEBRUARY 2 AND 20, 1984 (REPORT NO. 50-312/84-01) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION INCLUDING FOLLOWUP ON ITEMS OF NONCOMPLIANCE AND RADIOACTIVE WASTE

****
* RANCHO SECO 1 *
****

INSPECTION SUMMARY

AUDITS; SOLID, LIQUID AND GASEOUS RADIOACTIVE WASTE PROCESSING AND DISPOSAL; EFFLUENT INSTRUMENTATION; TESTS OF REACTOR COOLANT AND SECONDARY WATER; TESTS OF AIR CLEANING SYSTEMS; INDEPENDENT INSPECTION EFFORT; AND A FACILITY TOUR. THE INSPECTION INCLUDED FOLLOWUP ON A LICENSEE REPORT CONCERNING OFFSITE DOSE CALCULATIONS AND METEOROLOGICAL DATA. THE INSPECTION INVOLVED 41 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON FEBRUARY 7-10, 1984 (REPORT NO. 50-312/84-02) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON FEBRUARY 13-17, 1984 (REPORT NO. 50-312/84-03) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JANUARY 2 - FEBRUARY 24, 1984 (REPORT NO. 50-312/84-04) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT OPERATED THROUGHOUT THE MONTH EXCEPT FOR TWO BRIEF OUTAGES. ONE OCCURRED ON DECEMBER 13 FOLLOWING A REACTOR TRIP CAUSED BY TROUBLESHOOTING OF THE "A" INVERTER. THE OTHER OUTAGE, ON DECEMBER 15-19, RESULTED FROM INOPERABLE SERVICE AIR COMPRESSORS. PLANT POWER WAS LIMITED TO 64 PERCENT DURING PORTIONS OF THE MONTH BECAUSE OF EXCESS HYDRO-POWER.

LAST IE SITE INSPECTION DATE: 01/02-02/24/84+

INSPECTION REPORT NO: 50-312/84-04+

REPORTS FROM LICENSEE

****
* RANCHO SECO 1 *
****

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-08 01X-0	09-09-83	01-13-84	ERRORS IN METEOROLOGICAL DATA IMPACT OFFSITE DOSE CALCULATIONS (SPECIAL REPORT)

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1. Docket: 50-261 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: H. RAY NORRIS (803) 383-4524
4. Licensed Thermal Power (MWt): 2300
5. Nameplate Rating (Gross MWe): 854 X 0.9 = 769
6. Design Electrical Rating (Net MWe): 700
7. Maximum Dependable Capacity (Gross MWe): 700
8. Maximum Dependable Capacity (Net MWe): 665
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

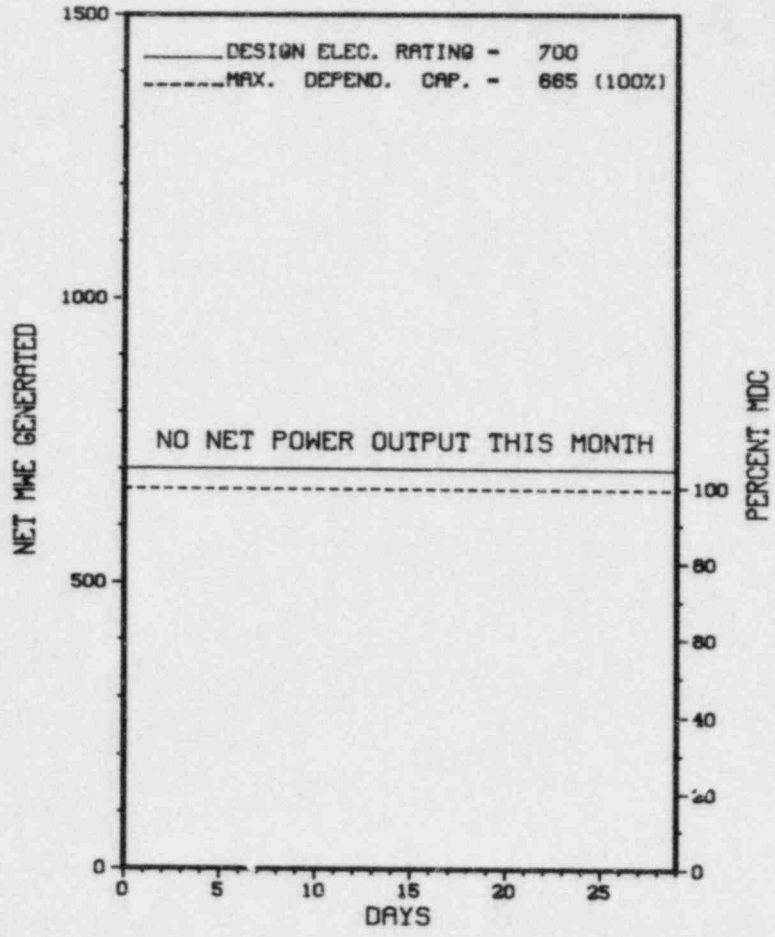
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	113,886.0
13. Hours Reactor Critical	.0	616.1	84,196.8
14. Rx Reserve Shtdwn Hrs	.0	38.9	1,675.5
15. Hrs Generator On-Line	.0	615.8	82,065.9
16. Unit Reserve Shtdwn Hrs	.0	.0	23.2
17. Gross Therm Ener (MWH)	0	783,895	162,875,180
18. Gross Elec Ener (MWH)	0	246,010	52,344,876
19. Net Elec Ener (MWH)	-2,766	221,513	49,441,137
20. Unit Service Factor	.0	42.8	72.1
21. Unit Avail Factor	.0	42.8	72.1
22. Unit Cap Factor (MDC Net)	.0	23.1	65.3
23. Unit Cap Factor (DER Net)	.0	22.0	62.0
24. Unit Forced Outage Rate	.0	17.2	14.6
25. Forced Outage Hours	.0	128.2	8,233.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: 01/17/85

****
* ROBINSON 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ROBINSON 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

ROBINSON 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
0201	01/26/84	S	696.0	C	4		RC	FUELXX	CONTINUATION OF REFUELING AND STEAM GENERATOR REPLACEMENT OUTAGE.

SUMMARY

ROBINSON 2 REMAINS SHUTDOWN IN A CONTINUING REFUELING MAINTENANCE OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ROBINSON 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....DARLINGTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI NW OF
HARTSVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...SEPTEMBER 20, 1970

DATE ELEC ENER 1ST GENER...SEPTEMBER 26, 1970

DATE COMMERCIAL OPERATE....MARCH 7, 1971

CONDENSER COOLING METHOD...RECIRCULATION

CONDENSER COOLING WATER....ROBINSON IMPOUNDMENT

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CAROLINA POWER & LIGHT

CORPORATE ADDRESS.....411 FAYETTEVILLE STREET
RALEIGH, NORTH CAROLINA 27601

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....S. WEISE

LICENSING PROJ MANAGER.....G. REQUA
DOCKET NUMBER.....50-261

LICENSE & DATE ISSUANCE....DPR-23, SEPTEMBER 23, 1970

PUBLIC DOCUMENT ROOM.....HARTSVILLE MEMORIAL LIBRARY
220 N. FIFTH ST.
HARTSVILLE, SOUTH CAROLINA 29550

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JANUARY 11 - FEBRUARY 10 (84-02): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 213 INSPECTOR HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, PLANT TOUR, OPERATIONS PERFORMANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, SITE SECURITY, SURVEILLANCE ACTIVITIES, MAINTENANCE ACTIVITIES, QUALITY ASSURANCE PRACTICES, RADIATION CONTROL ACTIVITIES, OUTSTANDING ITEMS REVIEW, IE BULLETIN AND NOTICE FOLLOWUP, ENFORCEMENT ACTION FOLLOWUP, ORGANIZATION AND ADMINISTRATION, STEAM GENERATOR REPAIR PREPARATIONS, AND REACTOR TRIP FOLLOWUP. OF THE 16 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 16 AREAS.

ENFORCEMENT SUMMARY

10 CFR 50.54(9) REQUIRES THAT NUCLEAR POWER REACTOR LICENSEES FOLLOW AND MAINTAIN IN EFFECT EMERGENCY PLANS WHICH MEET THE REQUIREMENTS OF APPENDIX E TO 10 CFR PART 50 AND THE PLANNING STANDARDS OF 50.47(B). SECTION (B)(10) OF 10 CFR 50.47 REQUIRES THAT THE LICENSEE'S EMERGENCY PLANS SHALL INCLUDE INFORMATION TO DEMONSTRATE COMPLIANCE WITH THE FOLLOWING: A RANGE OF PROTECTIVE ACTIONS HAVE BEEN DEVELOPED FOR THE PLUME EXPOSURE PATHWAY EPZ FOR EMERGENCY WORKERS AND THE PUBLIC. GUIDELINES FOR THE CHOICE OF PROTECTIVE ACTIONS DURING AN EMERGENCY, CONSISTENT WITH FEDERAL GUIDANCE, ARE DEVELOPED AND IN PLACE, AND PROTECTIVE ACTIONS FOR THE INGESTION EXPOSURE PATHWAY EPZ APPROPRIATE TO THE LOCALE HAVE BEEN DEVELOPED. THE FEDERAL GUIDANCE ON THE PROTECTIVE ACTIONS TO BE RECOMMENDED TO OFFSITE OFFICIALS FOR GENERAL EMERGENCIES IS ADDRESSED IN APPENDIX 1 OF NUREG 0654/FEMA-REP-1, REVISION 1, ENTITLED "CRITERIA FOR PREPARATION AND EVALUATION OF RADIOLOGICAL EMERGENCY PREPAREDNESS IN SUPPORT OF NUCLEAR POWER PLANTS." THIS

ENFORCEMENT SUMMARY

GUIDANCE IS CLARIFIED BY IE INFORMATION NOTICE NO. 83-28: "CRITERIA FOR PROTECTIVE ACTION RECOMMENDATIONS FOR GENERAL EMERGENCIES." CONTRARY TO THE ABOVE, THE LICENSEE HAS FAILED TO INCORPORATE THE ABOVE GUIDANCE IN THEIR PLAN OR IMPLEMENTING PROCEDURES IN THAT THERE IS NO MECHANISM OR REQUIREMENT FOR DETERMINING DIRECTLY FROM CONTAINMENT AND CORE STATUS WHAT OFFSITE PROTECTIVE ACTION RECOMMENDATIONS WILL BE MADE, OTHER THAN SHELTERING IN THE PLUME EPZ.
(8335 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: JANUARY 11 - FEBRUARY 10, 1984 +

INSPECTION REPORT NO: 50-261/84-02 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-272 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: L. K. MILLER (609) 935-6000 X4455

4. Licensed Thermal Power (MWt): 3338

5. Nameplate Rating (Gross MWe): 1300 X 0.9 = 1170

6. Design Electrical Rating (Net MWe): 1090

7. Maximum Dependable Capacity (Gross MWe): 1124

8. Maximum Dependable Capacity (Net MWe): 1079

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	58,465.0
13. Hours Reactor Critical	569.4	1,237.6	34,388.8
14. Rx Reserve Shtdwn Hrs	54.5	54.5	3,088.4
15. Hrs Generator On-Line	569.4	1 197.8	32,975.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,901,969	3,800,023	99,621,600
18. Gross Elec Ener (MWH)	645,390	1,281,380	32,894,278
19. Net Elec Ener (MWH)	618,884	1,223,764	31,195,076
20. Unit Service Factor	81.8	83.2	56.4
21. Unit Avail Factor	81.8	83.2	56.4
22. Unit Cap Factor (MDC Net)	82.4	78.8	49.5
23. Unit Cap Factor (DER Net)	81.6	78.0	49.0
24. Unit Forced Outage Rate	18.2	16.8	30.1
25. Forced Outage Hours	126.6	242.2	14,465.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

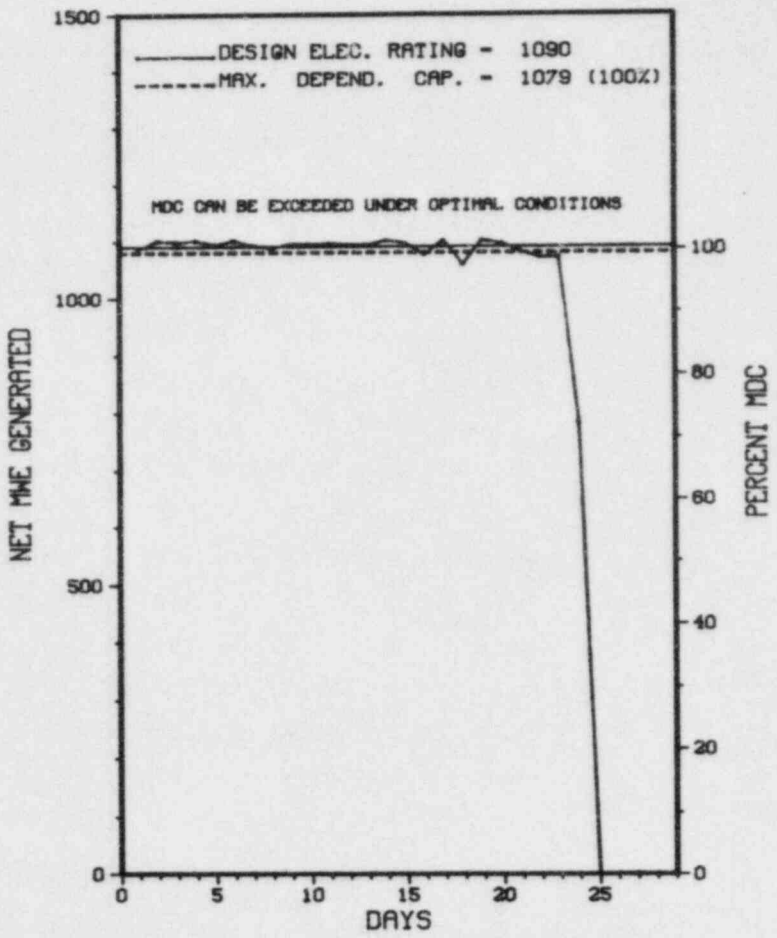
REFUELING: 3-1-84.

27. If Currently Shutdown Estimated Startup Date: 09/01/84

****
* SALEM 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SALEM 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

SALEM 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-144	02/18/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-146	02/18/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-148	02/18/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-150	02/18/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-152	02/19/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-154	02/19/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-156	02/20/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-158	02/21/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-160	02/22/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATE[illegible]OX CLEANING.
84-162	02/23/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-164	02/23/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-166	02/23/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-168	02/23/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-170	02/24/84	S	0.0	B	5		HC	HTEXCH	CONDENSER TUBE AND WATER BOX CLEANING.
84-172	02/24/84	F	126.6	A	3		HA	GENERA	SHORT IN GENERATOR WINDING.

SUMMARY

SALEM 1 OPERATED ROUTINELY, SHUTTING DOWN ON FEBRUARY 24TH FOR A REPAIR OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SALEM 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NEW JERSEY

COUNTY.....SALEM

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI S OF
WILMINGTON, DEL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY..DECEMBER 11, 1976

DATE ELEC ENER 1ST GENER...DECEMBER 25, 1976

DATE COMMERCIAL OPERATE....JUNE 30, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DELAWARE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PUBLIC SERVICE ELECTRIC & GAS

CORPORATE ADDRESS.....80 PARK PLACE
NEWARK, NEW JERSEY 07101

CONTRACTOR
ARCHITECT/ENGINEER.....PUBLIC SERVICES & GAS CO.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. LINVILLE

LICENSING PROJ MANAGER.....D. FISCHER
DOCKET NUMBER.....50-272

LICENSE & DATE ISSUANCE....DPR-70, DECEMBER 1, 1976

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112 WEST BROADWAY
SALEM, NEW JERSEY 08079

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

INSPECTION STATUS - (CONTINUED)

****
* SALEM 1 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-311 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: L. K. MILLER (609) 935-6000 X4455
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1162
6. Design Electrical Rating (Net MWe): 1115
7. Maximum Dependable Capacity (Gross MWe): 1149
8. Maximum Dependable Capacity (Net MWe): 1106
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	20,881.0
13. Hours Reactor Critical	18.6	18.6	11,727.1
14. Rx Reserve Shtdwn Hrs	677.4	1,421.4	3,512.0
15. Hrs Generator On-Line	.0	.0	11,417.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	4,879	4,879	33,475,951
18. Gross Elec Ener (MWH)	0	0	10,868,290
19. Net Elec Ener (MWH)	-13,922	-18,698	10,298,553
20. Unit Service Factor	.0	.0	54.7
21. Unit Avail Factor	.0	.0	54.7
22. Unit Cap Factor (MDC Net)	.0	.0	44.6
23. Unit Cap Factor (DER Net)	.0	.0	44.2
24. Unit Forced Outage Rate	100.0	100.0	33.0
25. Forced Outage Hours	696.0	1,440.0	5,623.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

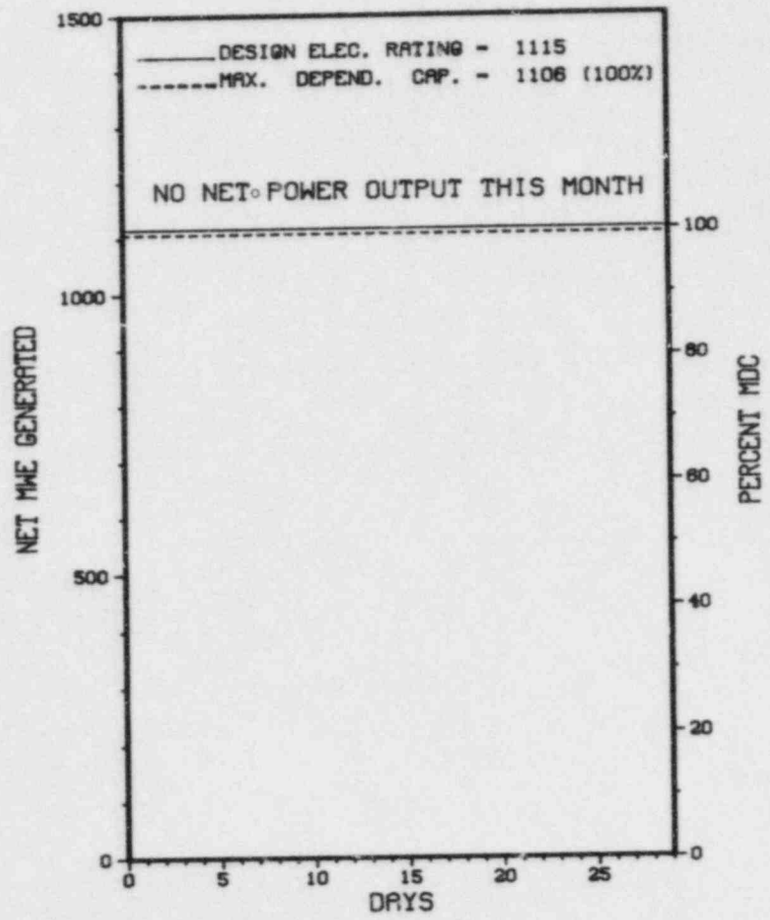
NONE

27. If Currently Shutdown Estimated Startup Date: 04/15/84

****
* SALEM 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SALEM 2



FEBRUARY 1984

Report Period FEB 1984 UNIT SHUTDOWNS / REDUCTIONS

****
* SALEM 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-010	11/17/83	F	696.0	A	4		HA	GENERA	STATOR CORE IRON GENERATOR.

****
* SUMMARY *
****

SALEM 2 REMAINS SHUTDOWN IN A CONTINUING REPAIR OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SALEM 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....NEW JERSEY

COUNTY.....SALEM

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI S OF
WILMINGTON, DEL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 8, 1980

DATE ELEC ENER 1ST GENER...JUNE 3, 1981

DATE COMMERCIAL OPERATE....OCTOBER 13, 1981

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DELAWARE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PUBLIC SERVICE ELECTRIC & GAS

CORPORATE ADDRESS.....80 PARK PLACE
NEWARK, NEW JERSEY 07101

CONTRACTOR
ARCHITECT/ENGINEER.....PUBLIC SERVICES & GAS CO.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. LINVILLE

LICENSING PROJ MANAGER.....D. FISCHER
DOCKET NUMBER.....50-311

LICENSE & DATE ISSUANCE....DPR-75, MAY 20, 1981

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112 WEST BROADWAY
SALEM, NEW JERSEY 08079

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-206 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: C. A. MORRIS (714) 492-7700 X56264
4. Licensed Thermal Power (MWt): 1347
5. Nameplate Rating (Gross MWe): 500 X 0.9 = 450
6. Design Electrical Rating (Net MWe): 436
7. Maximum Dependable Capacity (Gross MWe): 456
8. Maximum Dependable Capacity (Net MWe): 436
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	146,480.0
13. Hours Reactor Critical	.0	.0	88,440.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	84,821.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	108,263,946
18. Gross Elec Ener (MWH)	0	0	36,906,434
19. Net Elec Ener (MWH)	-2,330	-4,860	34,936,899
20. Unit Service Factor	.0	.0	55.8
21. Unit Avail Factor	.0	.0	55.8
22. Unit Cap Factor (MDC Net)	.0	.0	52.6
23. Unit Cap Factor (DER Net)	.0	.0	52.6
24. Unit Forced Outage Rate	.0	.0	21.9
25. Forced Outage Hours	.0	.0	11,178.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

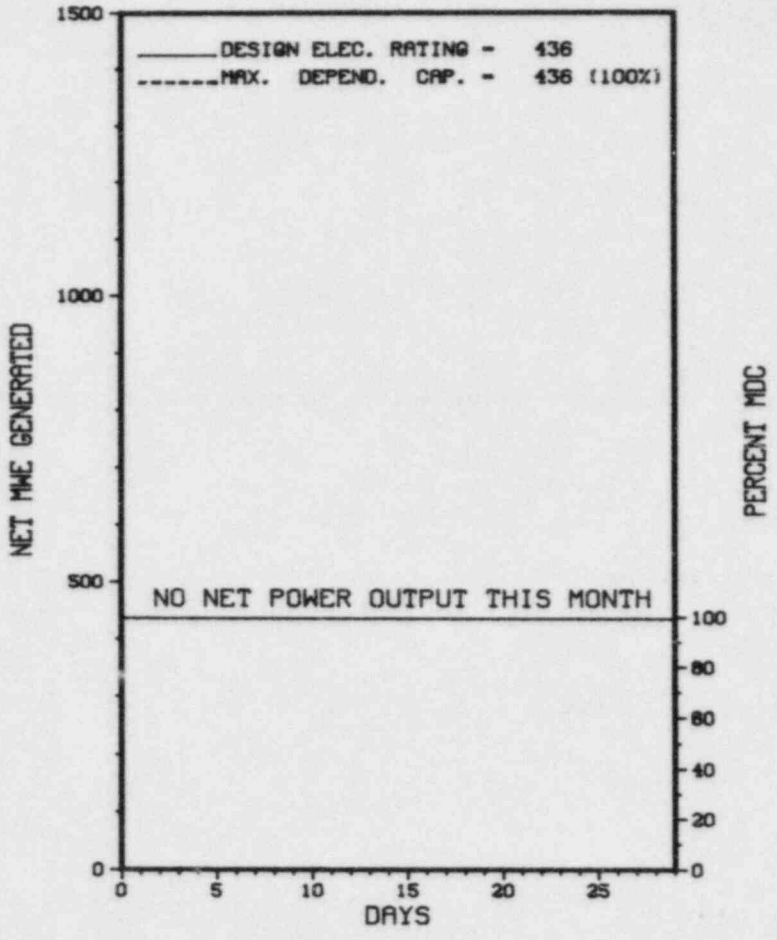
NONE

27. If Currently Shutdown Estimated Startup Date: 06/15/84

****
* SAN ONOFRE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SAN ONOFRE 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

SAN ONOFRE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
78	02/27/82	S	696.0	B	4				EXTENDED OUTAGE TO ACCOMPLISH SEISMIC BACKFIT AND MISCELLANEOUS MAINTENANCE ITEMS.

SUMMARY

SAN ONOFRE 1 REMAINS SHUTDOWN IN A CONTINUING MAINTENANCE OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SAN ONOFRE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN DIEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SAN CLEMENTE, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 14, 1967

DATE ELEC ENER 1ST GENER...JULY 16, 1967

DATE COMMERCIAL OPERATE....JANUARY 1, 1968

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTHERN CALIFORNIA EDISON

CORPORATE ADDRESS.....2244 WALNUT GROVE AVENUE
ROSEMEAD, CALIFORNIA 91770

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....A. DANGELO

LICENSING PROJ MANAGER.....W. PAULSON
DOCKET NUMBER.....50-206

LICENSE & DATE ISSUANCE....DPR-13, MARCH 27, 1967

PUBLIC DOCUMENT ROOM.....SAN CLEMENTE BRANCH LIBRARY
242 AVENIDA DEL MAR
SAN CLEMENTE, CALIFORNIA 92672

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON DECEMBER 12-16, 1983 (REPORT NO. 50-206/83-26) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JANUARY 15-20 AND 26, 1984 (REPORT NO. 50-206/84-02) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF RADIATION PROTECTION PROGRAM, HEALTH PHYSICS AND CHEMISTRY TRAINING, GENERAL EMPLOYEE TRAINING, AND PERSONNEL DOSIMETRY DURING OUTAGE CONDITIONS. THE INSPECTION INCLUDED A TOUR OF UNIT 1. THE INSPECTION INVOLVED 42 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 24-26, 1984 (REPORT NO. 50-206/84-03) AREAS INSPECTED: SECURITY PROGRAM AUDIT; FOLLOWUP ON ALLEGATIONS; SECURITY EVENT FOLLOWUP; INDEPENDENT INSPECTION EFFORT; AND FOLLOWUP ITEMS FROM PREVIOUS SECURITY INSPECTIONS. THE INSPECTION INVOLVED 25 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 23-27, 1984 (REPORT NO. 50-206/84-04) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF THE RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM AND FOLLOWUP ON LICENSEE ACTION REGARDING PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 22 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

INSPECTION SUMMARY

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 22 - FEBRUARY 23, 1984 (REPORT NO. 50-206/84-05) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON FEBRUARY 28 - MARCH 9, 1984 (REPORT NO. 50-206/84-06) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

THE PLANT HAS REMAINED SHUTDOWN SINCE LATE FEBRUARY 1982, FOR SEISMIC UP-GRADING, TMI MODIFICATIONS, STEAM GENERATOR TUBE RE-EVALUATION, EMERGENCY CORE COOLING SYSTEM TESTS, AND ROUTINE MAINTENANCE. THESE PROJECTS ARE NEARING COMPLETION AND RESTART IS AWAITING NRC RESOLUTION OF THE SEISMIC ISSUE.

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT HAS BEEN IN COLD SHUTDOWN SINCE FEBRUARY 1982, FOR EXTENSIVE SEISMIC REWORK. IN JUNE 1983, THE LICENSEE SUBMITTED TO NRR DESCRIPTIONS OF TWO PLANS FOR SONGS UNIT 1 AS WELL AS AN INDICATION OF THE CONTINUING ACTIONS WHICH WILL BE TAKEN UNTIL THE RESUMPTION OF POWER OPERATION. GENERALLY, THE LICENSEE WILL CONTINUE WORK TO COMPLETE NECESSARY MODIFICATIONS TO MAINTAIN THE PLANT IN MODE 5. AT THIS POINT, MODIFICATION EFFORTS WILL CEASE, AND A HOLD ON UNIT 1 ACTIVITIES WILL BE ASSUMED UNTIL AN ACCORD IS REACHED WITH NRR ON THE PLAN FOR RETURNING UNIT 1 TO POWER.

LAST IE SITE INSPECTION DATE: 02/28-03/09/84+

INSPECTION REPORT NO: 50-206/84-06+

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-361 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: C. A. MORRIS (717) 492-7700 X56264
4. Licensed Thermal Power (MWt): 3410
5. Nameplate Rating (Gross MWe): 1127
6. Design Electrical Rating (Net MWe): 1070
7. Maximum Dependable Capacity (Gross MWe): 1127
8. Maximum Dependable Capacity (Net MWe): 1070
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	4,945.0
13. Hours Reactor Critical	363.7	662.3	3,275.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	330.5	610.4	3,172.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,031,289	1,910,666	10,404,201
18. Gross Elec Ener (MWH)	348,887	651,468	3,563,432
19. Net Elec Ener (MWH)	323,201	605,432	3,381,076
20. Unit Service Factor	47.5	42.4	64.1
21. Unit Avail Factor	47.5	42.4	64.1
22. Unit Cap Factor (MDC Net)	43.4	39.0	63.9
23. Unit Cap Factor (DER Net)	43.4	39.0	63.9
24. Unit Forced Outage Rate	.0	4.9	4.0
25. Forced Outage Hours	.0	31.6	132.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

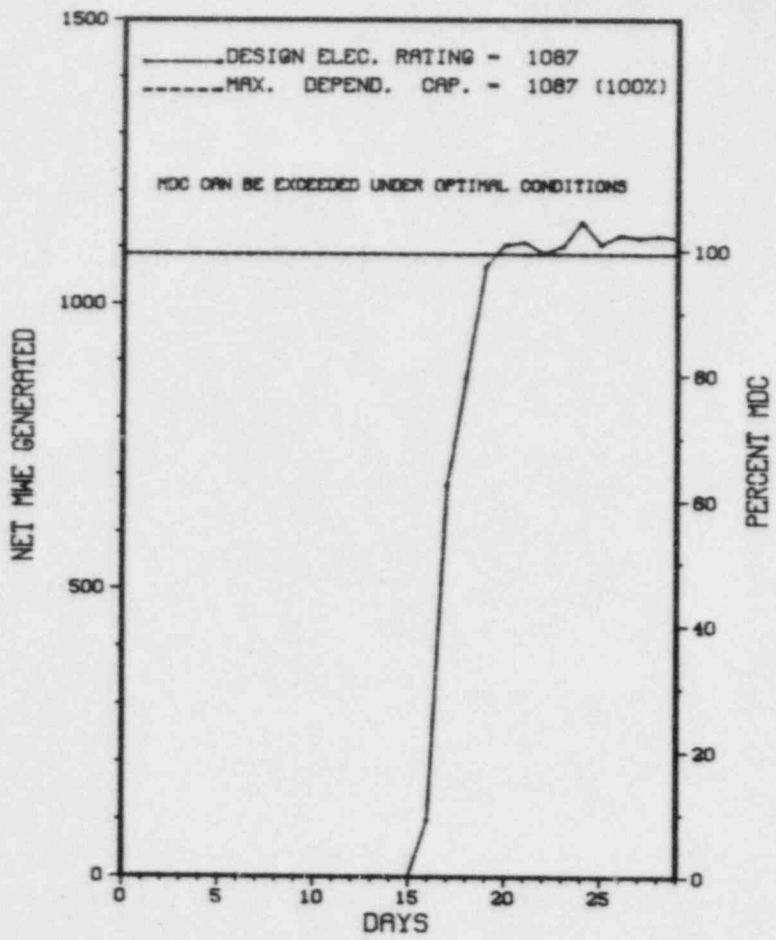
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SAN ONOFRE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SAN ONOFRE 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* SAN ONOFRE 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2	01/13/84	S	365.5	B	4				UNIT WAS SHUT DOWN FOR REACTOR COOLANT PUMP SEAL REPLACEMENT.

****
* SUMMARY *
****

SAN ONOFRE 2 RETURNED ONLINE FROM A REPAIR OUTAGE ON FEBRUARY 16TH AND OPERATED ROUTINELY THE REMAINDER OF THE MONTH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SAN ONOFRE 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN DIEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SAN CLEMENTE, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 26, 1982

DATE ELEC ENER 1ST GENER...SEPTEMBER 20, 1982

DATE COMMERCIAL OPERATE....AUGUST 8, 1983

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTHERN CALIFORNIA EDISON

CORPORATE ADDRESS.....P.O. BOX 800
ROSEMEAD, CALIFORNIA 91770

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC COM (ENG VERSION)

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....A. CHAFFEE

LICENSING PROJ MANAGER.....H. ROOD
DOCKET NUMBER.....50-361

LICENSE & DATE ISSUANCE....., SEPTEMBER 7, 1982

PUBLIC DOCUMENT ROOM.....SAN CLEMENTE LIBRARY
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SAN CLEMENTE, CALIFORNIA

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 22 - SEPTEMBER 16, 1983 (REPORT NO. 50-361/83-30) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON DECEMBER 12-16, 1983 (REPORT NO. 50-361/83-41) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JANUARY 15-20 AND 26, 1984 (REPORT NO. 50-361/84-02) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF RADIATION PROTECTION PROGRAM, HEALTH PHYSICS AND CHEMISTRY TRAINING, GENERAL EMPLOYEE TRAINING, AND PERSONNEL DOSIMETRY DURING OUTAGE CONDITIONS. THE INSPECTION INCLUDED A TOUR OF UNIT 2. THE INSPECTION INVOLVED 42 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 24-26, 1984 (REPORT NO. 50-361/84-03) AREAS INSPECTED: FACILITY ORGANIZATION, FACILITY OPERATION, SHIPPING AND RECEIVING, RECORDS AND REPORTS, FOLLOWUP ON A NOTICE OF DEVIATION. THE INSPECTION INVOLVED 14 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 24-26, 1984 (REPORT NO. 50-361/84-04) AREAS INSPECTED: SECURITY PROGRAM AUDIT; FOLLOWUP ON ALLEGATIONS; SECURITY EVENT FOLLOWUP; INDEPENDENT INSPECTION EFFORT; AND FOLLOWUP ITEMS FROM PREVIOUS SECURITY INSPECTIONS. THE INSPECTION

INSPECTION SUMMARY

INVOLVED 25 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 23-27, 1984 (REPORT NO. 50-361/84-05) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF THE RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM AND FOLLOWUP ON LICENSEE ACTION REGARDING PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 22 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 22 - FEBRUARY 23, 1984 (REPORT NO. 50-361/84-06) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON FEBRUARY 27 - MARCH 1, 1984 (REPORT NO. 50-361/84-07) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON FEBRUARY 28 - MARCH 9, 1984 (REPORT NO. 50-361/84-08) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

LOW POWER FACILITY OPERATING LICENSE WAS ISSUED FEBRUARY 16, 1982. THE FULL POWER FACILITY OPERATING LICENSE WAS ISSUED SEPTEMBER 7, 1982, AS AMENDMENT 7 TO THE LOW POWER LICENSE. THE PLANT COMMENCED COMMERCIAL OPERATION ON AUGUST 7, 1983.

PLANT STATUS:

STEADY OPERATION AT FULL POWER; PROBLEMS BEING EXPERIENCED WITH SEALS ON ONE REACTOR COOLANT PUMP.

LAST IE SITE INSPECTION DATE: 02/28-3/09/84+

INSPECTION REPORT NO: 50-361/84-08+

Report Period FEB 1984

REPORTS FROM LICENSEE

****
* SAN ONOFRE 2 *
****

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

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1. Docket: 50-362 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: C.A. MORRIS (714) 492-7700 EXT 56264
4. Licensed Thermal Power (MWt): 3390
5. Nameplate Rating (Gross MWe): 1127
6. Design Electrical Rating (Net MWe): 1070
7. Maximum Dependable Capacity (Gross MWe): 1127
8. Maximum Dependable Capacity (Net MWe): 1070
9. If Changes Occur Above Since Last Report, Give Reasons:

---

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	3,774.0
13. Hours Reactor Critical	.0	142.8	1,924.2
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	123.0	1,766.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	415,741	3,964,779
18. Gross Elec Ener (MWH)	0	140,226	1,238,523
19. Net Elec Ener (MWH)	-5,210	121,446	1,118,513
20. Unit Service Factor			
21. Unit Avail Factor		NOT IN	
22. Unit Cap Factor (MDC Net)		COMMERCIAL	
23. Unit Cap Factor (DER Net)		OPERATION	
24. Unit Forced Outage Rate			
25. Forced Outage Hours	.0	.0	556.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

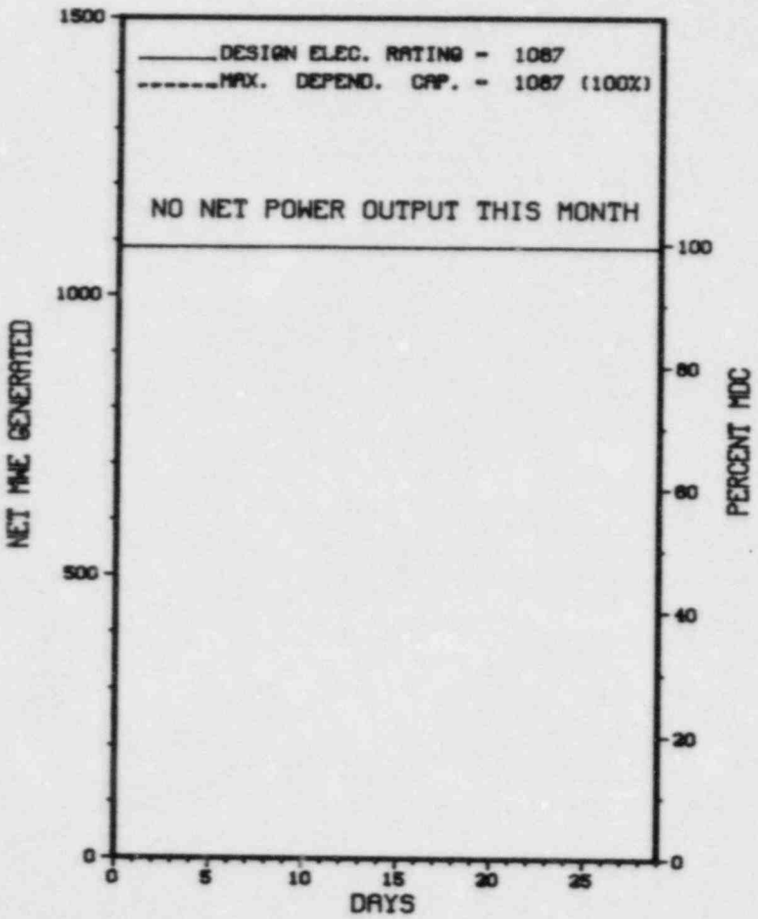
NONE

27. If Currently Shutdown Estimated Startup Date: 03/08/84

****
* SAN ONOFRE 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SAN ONOFRE 3



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

SAN ONOFRE 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2	01/13/84	S	365.5	B	4				UNIT WAS SHUT DOWN FOR REACTOR COOLANT PUMP SEAL REPLACEMENT.
1	01/06/84	S	696.0	B	4				UNIT IS SHUTDOWN FOR A SURVEILLANCE AND REACTOR COOLANT PUMP SEAL OUTAGE.
1	01/06/84	S	696.0	B	4				UNIT IS SHUTDOWN FOR A SURVEILLANCE AND REACTOR COOLANT PUMP SEAL OUTAGE.

SUMMARY

SAN ONOFRE 3 REMAINS SHUTDOWN IN A CONTINUING MAINTENANCE OUTAGE.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SAN ONOFRE 3 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN DIEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SAN CLEMENTE, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 29, 1983

DATE ELEC ENER 1ST GENER...SEPTEMBER 25, 1983

DATE COMMERCIAL OPERATE....**********

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTHERN CALIFORNIA EDISON

CORPORATE ADDRESS.....P.O. BOX 800
ROSEMEAD, CALIFORNIA 91770

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC COM (ENG VERSION)

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....A. CHAFFEE

LICENSING PROJ MANAGER.....H. ROOD
DOCKET NUMBER.....50-362

LICENSE & DATE ISSUANCE....., NOVEMBER 15, 1982

PUBLIC DOCUMENT ROOM.....SAN CLEMENTE LIBRARY
242 AVENIDA DEL MAR
SAN CLEMENTE, CALIFORNIA

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 22 - SEPTEMBER 16, 1983 (REPORT NO. 50-362/83-28) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON DECEMBER 12-16, 1983 (REPORT NO. 50-362/83-40) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JANUARY 15-20 AND 26, 1984 (REPORT NO. 50-362/84-02) AREAS INSEPCTED: ROUTINE, UNANNOUNCED INSPECTION OF RADIATION PROTECTION PROGRAM, HEALTH PHYSICS AND CHEMISTRY TRAINING, GENERAL EMPLOYEE TRAINING, AND PERSONNEL DOSIMETRY DURING OUTAGE CONDITIONS. THE INSPECTION INCLUDED A TOUR OF UNIT 3. THE INSPECTION INVOLVED 42 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 24-26, 1984 (REPORT NO. 50-362/84-03) AREAS INSPECTED: FACILITY ORGANIZATION, FACILITY OPERATION, SHIPPING AND RECEIVING, RECORDS AND REPORTS, FOLLOWUP ON A NOTICE OF DEVIATION. THE INSPECTION INVOLVED 14 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 24-26, 1984 (REPORT NO. 50-362/84-04) AREAS INSEPCTED: SECURITY PROGRAM AUDIT; FOLLOWUP ON ALLEGATIONS; SECURITY EVENT FOLLOWUP; INDEPENDENT INSPECTION EFFORT; AND FOLLOWUP ITEMS FROM PREVIOUS SECURITY INSPECTIONS. THE INSPECTION

INSPECTION SUMMARY

INVOLVED 25 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 23-27, 1984 (REPORT NO. 50-362/84-05) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF THE RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM AND FOLLOWUP ON LICENSEE ACTION REGARDING PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 22 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JANUARY 22 - FEBRUARY 23, 1984 (REPORT NO. 50-362/84-06) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON FEBRUARY 28 - MARCH 9, 1984 (REPORT NO. 50-362/84-07) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

ABNORMALLY HIGH RADIATION LEVELS OBSERVED IN REACTOR COOLANT SYSTEM.

FACILITY ITEMS (PLANS AND PROCEDURES):

SEVEN WEEK OUTAGE FOR REPLACEMENT OF REACTOR COOLANT PUMP SEALS AND SURVEILLANCE TESTING.

MANAGERIAL ITEMS:

LOW POWER FACILITY OPERATING LICENSE WAS ISSUED NOVEMBER 15, 1982. THE FULL POWER LICENSE WAS ISSUED SEPTEMBER 16, 1983.

PLANT STATUS:

INITIAL CRITICALITY WAS AUGUST 29, 1983. POWER ASCENSION TESTING WAS COMPLETED ON JANUARY 6, 1984. THE UNIT WILL NOW BE SHUT DOWN FOR ABOUT SEVEN WEEKS FOR REPLACEMENT OF REACTOR COOLANT PUMP SEALS AND SURVEILLANCE TESTING.

ABNORMALLY HIGH LEVELS OF RADIOACTIVITY HAVE BEEN OBSERVED, AND THE CAUSE AND NECESSARY CORRECTIVE ACTIONS ARE BEING EVALUATED.

LAST IE SITE INSPECTION DATE: 02/28-03/09/84+

INSPECTION REPORT NO: 50-362/84-07+

REPORTS FROM LICENSEE

SAN ONOFRE 3

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

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1. Docket: 50-327 O P E R A T I N G S T A T U S

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: MIKE EDDINGS (615) 870-6248

4. Licensed Thermal Power (MWt): 3411

5. Nameplate Rating (Gross MWe): 1220

6. Design Electrical Rating (Net MWe): 1148

7. Maximum Dependable Capacity (Gross MWe): 1183

8. Maximum Dependable Capacity (Net MWe): 1148

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	23,377.0
13. Hours Reactor Critical	479.5	1,014.9	15,456.3
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	479.5	961.3	15,074.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,494,485	2,870,308	48,362,108
18. Gross Elec Ener (MWH)	501,810	956,150	16,337,286
19. Net Elec Ener (MWH)	480,444	915,067	15,691,995
20. Unit Service Factor	68.9	66.8	64.5
21. Unit Avail Factor	68.9	66.8	64.5
22. Unit Cap Factor (MDC Net)	60.1	55.4	58.5
23. Unit Cap Factor (DER Net)	60.1	55.4	58.5
24. Unit Forced Outage Rate	.0	21.4	19.5
25. Forced Outage Hours	.0	262.2	3,642.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

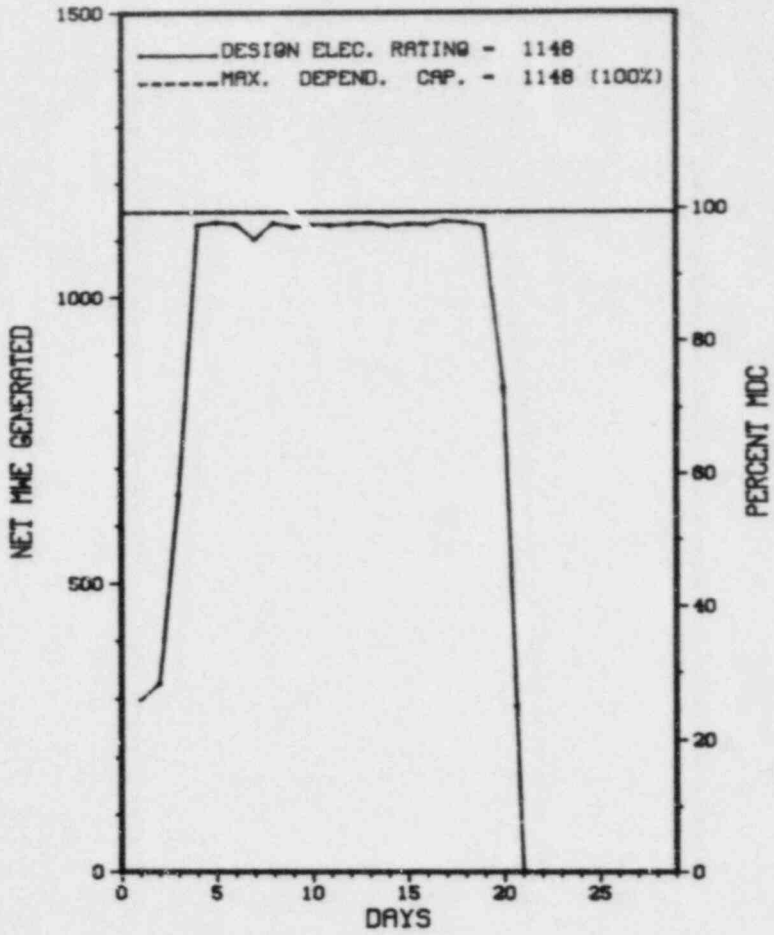
NONE

27. If Currently Shutdown Estimated Startup Date: 04/13/84

****
* SEQUOYAH 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SEQUOYAH 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

SEQUOYAH 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
5	02/20/84	S	216.5	C	1		RC	FUELXX	REFUELING OUTAGE COMMENCES.

SUMMARY

SEUOYAH 1 ENTERED A REFUELING OUTAGE ON FEBRUARY 20TH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SEQUOYAH 1 *
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FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....TENNESSEE

COUNTY.....HAMILTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9.5 MI NE OF
CHATTANOOGA, TN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 5, 1980

DATE ELEC ENER 1ST GENER...JULY 22, 1980

DATE COMMERCIAL OPERATE....JULY 1, 1981

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHICKAMAUGA LAKE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....500A CHESTNUT STREET TOWER II
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....E. FORD

LICENSING PROJ MANAGER.....C. STAHLE
DOCKET NUMBER.....50-327

LICENSE & DATE ISSUANCE....DPR-77, SEPTEMBER 17, 1980

PUBLIC DOCUMENT ROOM.....CHATTANOOGA - HAMILTON BICENTENNIAL LIBRARY
1001 BROAD STREET
CHATTANOOGA, TENNESSEE 37402

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 6, 1983 - JANUARY 5 AND JANUARY 16, 1984 (83-31): THIS ROUTINE INSPECTION INVOLVED 79 INSPECTOR HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY VERIFICATION, ESF SYSTEM OPERABILITY VERIFICATION, PROCEDURAL INDEPENDENT VERIFICATION REVIEW AND INDEPENDENT INSPECTION EFFORT. AN ENFORCEMENT CONFERENCE WAS ALSO CONDUCTED AT THE END OF THE INSPECTION PERIOD. OF THE FOUR AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; FIVE VIOLATIONS WERE FOUND IN ONE AREA (FAILURE TO FOLLOW SI-166.3, PARAGRAPH 6.C, FAILURE TO FOLLOW SOI-63.1, PARAGRAPH 6.A, FAILURE TO MEET TS 3.6.1.1, PARAGRAPH 6.D, FAILURE TO FOLLOW RCI-14, PARAGRAPH 6.B, AND FAILURE TO MEET TS 3.8.2.1 FOR THE 1-I INVERTER, PARAGRAPH 6.E).

INSPECTION JANUARY 16-20 (84-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 51 INSPECTOR HOURS ON SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS ENFORCEMENT MATTERS; QA PROGRAM REVIEW; NON-LICENSED PERSONNEL TRAINING; LICENSED OPERATOR REQUALIFICATION TRAINING; PROCUREMENT PROGRAM; RECEIPT, STORAGE, AND HANDLING OF EQUIPMENT AND MATERIALS; RECORDS; AND DOCUMENT CONTROL. OF THE EIGHT AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO APPARENT VIOLATIONS WERE FOUND IN TWO AREAS (FAILURE TO PERFORM REQUIRED ABNORMAL AND EMERGENCY PROCEDURE REVIEW, PARAGRAPH 7.A, AND FAILURE TO PROVIDE PROPER MATERIAL STORAGE, PARAGRAPH 9).

INSPECTION JANUARY 6 - FEBRUARY 5 (84-03): THIS ROUTINE, INSPECTION INVOLVED 82 INSPECTOR HOURS ON SITE IN THE AREAS OF OPERATION SAFETY VERIFICATION, ESF SYSTEM OPERABILITY VERIFICATION, INSPECTION OF TMI ACTION PLAN REQUIREMENTS, LICENSEE EVENT REPORT REVIEW AND INDEPENDENT INSPECTION EFFORT. OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERIA I & XVI AS IMPLEMENTED BY THE LICENSEES QA PROGRAM (TVA-TR75-1A) OQA IS NOT ENSURING EFFECTIVE EXECUTION OF THE QA PROGRAM IN THAT ALL CONDITIONS ADVERSE TO QUALITY HAVE NOT BEEN PROMPTLY CORRECTED. THE CURRENT COMPOSITE OPEN ITEM LIST REVIEW SUMMARY CONTAINS 1 OUTSTANDING ITEM FROM 1979, 16 OUTSTANDING ITEMS FROM 1981 AND 65 OUTSTANDING ITEMS FROM 1982. REGION II HAS ISSUED FOUR VIOLATIONS SINCE FEBRUARY 1981 FOR FAILURE TO TAKE PROMPT CORRECTIVE ACTION BY MECHANISMS DEFINED WITHIN THE QA PROGRAM. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION III AS IMPLEMENTED BY THE LICENSEE'S QA PROGRAM (TVA-TR75-1A), THE CONSTRUCTION ENGINEERING BRANCH HAS NOT ESTABLISHED MEASURES TO CONTROL DESIGN INTERFACES BETWEEN PARTICIPATING ORGANIZATIONS. THIS WAS IDENTIFIED IN THE LICENSEE'S AUDIT 83V-26 AND AGAIN IN AUDIT 83V-73. BOTH OF THESE AUDITS WERE CONDUCTED ON DIFFERENT VENDORS. CONTRARY TO BROWNS FERRY TECHNICAL SPECIFICATION (TS) 6.10.C, SEQUOYAH TS 6.5.2.10.C, THE LICENSEES ACCEPTED QA PROGRAM'S ENDORSEMENT OF REGULATORY GUIDE 1.144 AND ANSI N45.2.12 - 1974 OR 1977, PARAGRAPH 4.4.6 MULTIPLE EXAMPLES WERE IDENTIFIED OF FAILURE TO ISSUE AUDITS WITHIN REQUIRED TIMEFRAMES. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII AS IMPLEMENTED BY THE LICENSEES QA PROGRAM (TVA-TR75-1A), REGULATORY GUIDE 1.144, ANSI AND N45.2.12, PARAGRAPH 4.5.1, NUMEROUS EXAMPLES WERE IDENTIFIED WHERE THE AUDITED ORGANIZATION DID NOT RESPOND TO AUDITS FINDINGS WITHIN REQUIRED TIMEFRAMES. THIS IS A REPEAT VIOLATION. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION II AS IMPLEMENTED BY THE LICENSEES QA PROGRAM, REGULATORY GUIDE 1.146, AND ANSI N45.2.23, MEASURES HAD NOT BEEN ESTABLISHED TO REQUIRE VERIFICATION OF MINIMUM CREDITS NEEDED TO BE A LEAD AUDITOR.
(8327 4)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVII AS IMPLEMENTED BY THE LICENSEE'S QA PROGRAM, REGULATORY GUIDE 1.88, AND ANSI N45.2.9 RECORDS WERE NOT MAINTAINED TO DEMONSTRATE THAT ALL AUDITORS AND LEAD AUDITORS WERE QUALIFIED TO PERFORM SAFETY-RELATED QA AUDITS. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII IMPLEMENTED BY THE LICENSEE'S QA PROGRAM, REGULATORY GUIDE 1.144, AND ANSI N45.2.12, PROCEDURE DID NOT DELINEATE THAT PERSONS CONTACTED DURING THE AUDIT BE IDENTIFIED IN THE AUDIT REPORT.
(8327 5)

10 CFR 50, APPENDIX B, CRITERION X AND THE LICENSEE'S ACCEPTED QUALITY ASSURANCE PROGRAM (TOPICAL REPORT TVA-TR75-1) SECTION 17.2.10 REQUIRE THAT INSPECTION SHALL BE PERFORMED DURING MODIFICATION AFFECTING THE QUALITY OF CRITICAL SYSTEMS, STRUCTURES AND COMPONENTS (CSSC) ITEMS AT TVA PLANTS. MODIFICATION AND ADDITION INSTRUCTION (M&AI)-12 "INTERCONNECTING CABLE TERMINATION AND INSULATION INSPECTION", SECTION 5.0, FURTHER REQUIRES THAT QA INSPECTORS SHALL BE RESPONSIBLE FOR INSPECTING PER THIS PROCEDURE ON CSSC EQUIPMENT. CONTRARY TO THE ABOVE, INSPECTION WAS NOT PERFORMED ON CSSC EQUIPMENT AS REQUIRED BY M&AI-12 SECTION 5 IN THAT DURING THE PERFORMANCE OF WORK PLAN WP10260, WHICH REROUTED SIGNAL CABLES FROM RADIATION MONITORS 1-119, 1-120, 1-121, 2-120 AND 2-121, THE INSPECTION OF THE CABLE TERMINATION PER M&AI-12 WAS PERFORMED BY THE COGNIZANT ENGINEER INSTEAD OF A QA INSPECTOR. THE AFFECTED RADIATION MONITORS ARE CSSC EQUIPMENT. TECHNICAL SPECIFICATION 6.8.1.A REQUIRES THAT WRITTEN PROCEDURES SHALL BE IMPLEMENTED COVERING THE ACTIVITIES REFERENCED IN APPENDIX A OF REGULATORY GUIDE 1.33, REVISION 2, FEBRUARY 1978, INCLUDING DISCHARGING LIQUID RADIOACTIVE WASTE AS EFFLUENTS. SYSTEM OPERATING INSTRUCTION (SOI)-14.3 "CONDENSATE DEMINERALIZER WASTE DISPOSAL" PROVIDES REQUIREMENTS, CONDITIONS, PRECAUTIONS AND INSTRUCTIONS FOR RELEASING THE HIGH CRUD TANKS (HCT). CONTRARY TO THE ABOVE, PROCEDURE SOI-14.3 WAS NOT IMPLEMENTED IN THAT ON OCTOBER 15, 1983, DURING A PLANNED RELEASE FROM HCT "B" TO COOLING TOWER BLOWDOWN, THE TANK WAS PARTIALLY RELEASED TO THE TURBINE BUILDING SUMP BECAUSE THE VALVE ALIGNMENT WAS NOT PROPERLY COMPLETED IN ACCORDANCE WITH THE PROCEDURE. WHEN THE ERROR WAS DISCOVERED, THE RELEASE WAS STOPPED, THE VALVE ALIGNMENT CORRECTED AND THE RELEASE PROPERLY COMPLETED. THE ACTIVITY LEVELS IN THE TANK WERE LESS THAN 10 CFR 20, APPENDIX B, TABLE II LIMITS, THEREFORE, TECHNICAL SPECIFICATION RELEASE LIMITS WERE NOT EXCEEDED.
(8329 4)

TECHNICAL SPECIFICATION 6.8.1.C. REQUIRES THAT WRITTEN PROCEDURES BE IMPLEMENTED FOR SURVEILLANCE AND TEST ACTIVITIES OF SAFETY-RELATED EQUIPMENT. SURVEILLANCE INSTRUCTION SI-166.3 "FULL STROKING OF CATEGORY "A & B" VALVES DURING COLD SHUTDOWN" PROVIDES PREREQUISITES, PRECAUTIONS AND INSTRUCTIONS FOR INSERVICE TESTING OF VARIOUS SAFETY RELATED VALVES INCLUDING 1-FCV-72-41. CONTRARY TO THE ABOVE, WRITTEN PROCEDURES FOR SURVEILLANCE ACTIVITIES OF SAFETY-RELATED EQUIPMENT WERE NOT PROPERLY IMPLEMENTED IN THAT ON DECEMBER 16, 1983 VALVE 1-FCV-72-41 WAS OPENED WITHOUT VALVES 1-HCV-74-37 AND 1-74-531 BEING SHUT AS REQUIRED BY SECTION 7.7.3.2 OF SI 166.3. THE IMPROPER VALVE ALIGNMENT RESULTED IN APPROXIMATELY 600 GALLONS OF PRIMARY COOLANT BEING SPRAYED INTO UNIT 1 CONTAINMENT. TECHNICAL SPECIFICATION 6.11 REQUIRES THAT PROCEDURES FOR PERONNEL RADIATION PROTECTION SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. RADIOLOGICAL CONTROL INSTRUCTION RCI-14

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* SEQUOYAH 1 *
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ENFORCEMENT SUMMARY

"RADIATION WORK PERMIT (RWP) PROGRAM" REQUIRES THAT EACH WORKER ENTERING AN RWP AREA RECORD HIS NAME, SOCIAL SECURITY NUMBER, DATE, TIME AND DOSIMETER READING EACH TIME HE ENTERS AND LEAVES THE AREA AND COMPLY WITH ANY OTHER INSTRUCTIONS OF THE RWP. CONTRARY TO THE ABOVE, PROCEDURES FOR PERSONNEL RADIATION PROTECTION WERE NOT ADHERED TO IN THAT ON DECEMBER 22, 1983 THE INSPECTOR NOTED THAT KEY CARD RECORDS FOR DOOR A-8 TO THE 2A RHR PUMP ROOM SHOWED THAT APPROXIMATELY 200 ENTRIES WERE MADE INTO THE ROOM BETWEEN NOVEMBER 19 AND DECEMBER 8, 1983. REVIEW OF RWP 02-2-00831 TIME SHEETS 34, 35, 36 AND 37 WHICH WERE IN EFFECT FOR THAT TIME PERIOD REVEALED THAT ONLY 34 ENTRIES WERE LOGGED.
(8331 4)

CONTRARY TO 10CFR55, APPENDIX A, PARAGRAPH 3D, EACH LICENSED OPERATOR AND SENIOR OPERATOR HAS NOT REVIEWED THE CONTENTS OF ALL ABNORMAL INSTRUCTIONS (AOI) OR EMERGENCY INSTRUCTIONS (EOI) ON A REGULARLY SCHEDULED BASIS.
(8401 4)

CONTRARY TO 10CFR50, APPENDIX B, CRITERION XIII, THE LICENSEE FAILED TO PROVIDE ADEQUATE SUPPORT OF REACTOR PLANT EQUIPMENT STORED IN OUTSIDE BUILDING 1. FAILURE TO SUPPORT PROPERLY RESULTED IN A 1 3/4 INCH DEFLECTION OF THE WOODEN CONTAINER AND COULD RESULT IN DEFORMATION OF PARTS WITHIN THE CONTAINER.

(8401 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

OUTAGE

LAST IE SITE INSPECTION DATE: JANUARY 6 - FEBRUARY 5, 1984 +

INSPECTION REPORT NO: 50-327/84-03 +

REPORTS FROM LICENSEE

SEQUOYAH 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-113/ 83-116/ 03L-0	09/01/83 09/24/83	09/28/83 10/20/83	STEAM GENERATOR BLOWDOWN EFFLUENT LINE RADIATION MONITOR DECLARED LOSS OF P-250 COMPUTER CAUSING REACTOR COOLANT SUBCOOLING MARGIN MONITOR INOPERABLE DUE TO POWER SPIKE TO THE DISC.
83-117/ 01T-0	09/26/83	10/07/83	MONTHLY POWER RANGE INCORE-EXCORE CHANNEL COMPARISON NOT PERFORMED IN SPECIFIED TIME INTERVAL DUE TO PERSONNEL ERROR.
83-119/ 03L-0	09/10/83	10/07/83	CONTAINMENT INTERNAL PRESSURE LIMIT OF +.3 PSIG RELATIVE TO ANNULUS PRESSURE EXCEEDED DUE TO NORMAL AIR LEAKAGE.
83-120/ 03L-0	09/10/83	10/07/83	ALL ICE CONDENSER DOORS OPENED WHEN 1-FCV-63-73 WAS STROKED DURING PERFORMANCE OF SI-166.3.
83-122/ --	- -	- -	D/G 2A-A DECLARED INOPERABLE WHEN IT FAILED TO MAINTAIN A LOAD OF 4400KW DURING TEST-CAUSE FAULTY EXCITER TRANSFORMER.
83-123/ 03L-0	09/26/83	10/21/83	CONTROL DAMPER I-FCV-30-149 FAILED TO OPERATE DUE TO FLOW CONTROLLER FC-30-149 BEING INOPERABLE.
83-126/ 03L-0	09/26/83	10/25/83	ICE BUILDUP ON SEVERAL INTERMEDIATE DECK DOORS DUE TO HUMIDITY ENTERING THE ICE CONDENSER.
03L-0			INOPERABLE DUE TO LOSS OF EFFLUENT FLOW.

1. Docket: 50-328 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: DAVID DUPREE (615) 870-6543
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1220
6. Design Electrical Rating (Net MWe): 1148
7. Maximum Dependable Capacity (Gross MWe): 1183
8. Maximum Dependable Capacity (Net MWe): 1148
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	15,337.0
13. Hours Reactor Critical	635.8	1,379.8	11,740.9
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	631.6	1,375.6	11,530.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,077,520	4,613,057	37,031,124
18. Gross Elec Ener (MWH)	716,270	1,597,750	12,629,690
19. Net Elec Ener (MWH)	691,337	1,541,099	12,158,837
20. Unit Service Factor	90.7	95.5	75.2
21. Unit Avail Factor	90.7	95.5	75.2
22. Unit Cap Factor (MDC Net)	86.5	93.2	69.1
23. Unit Cap Factor (DER Net)	86.5	93.2	69.1
24. Unit Forced Outage Rate	9.3	4.5	9.2
25. Forced Outage Hours	64.4	64.4	1,166.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Da N/A

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* SEQUOYAH 2 *
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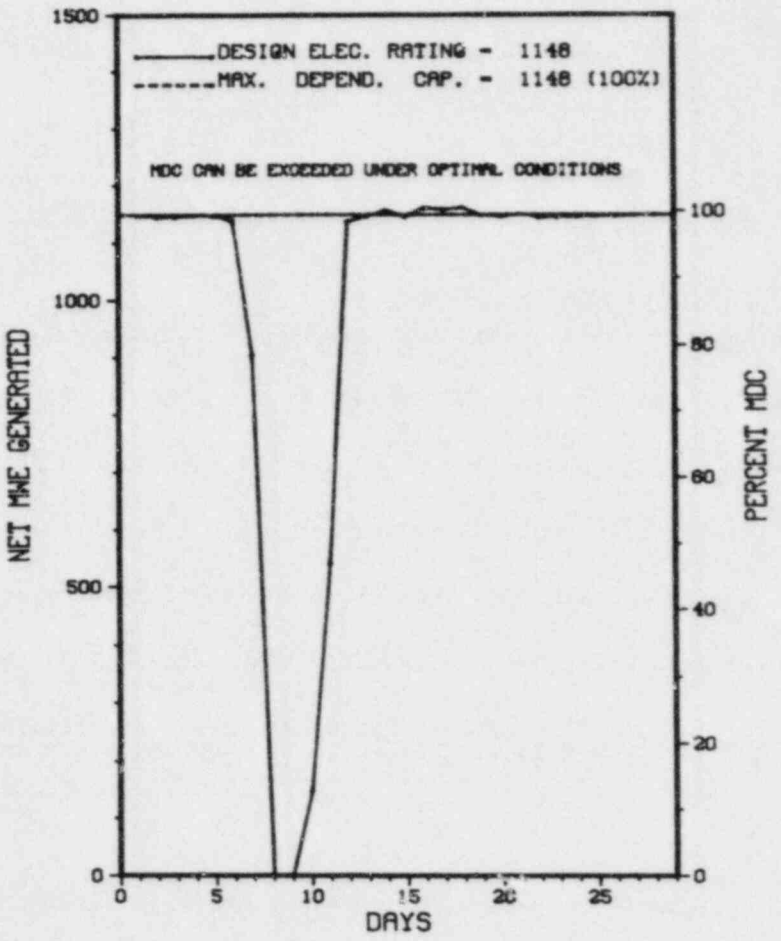
AVERAGE DAILY POWER LEVEL (MWe) PLOT

SEQUOYAH 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

SEQUOYAH 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	02/07/84	F	64.4	A	1				STEAM GENERATOR CHEMISTRY OUT OF SPECIFICATIONS.

SUMMARY

SEQUOYAH 2 OPERATED WITH 1 OUTAGE FOR EQUIPMENT REPAIR DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SEQUOYAH 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....TENNESSEE

COUNTY.....HAMILTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9.5 MI NE OF
CHATTANOOGA, TN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 5, 1981

DATE ELEC ENER 1ST GENER...DECEMBER 23, 1981

DATE COMMERCIAL OPERATE....JUNE 1, 1982

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHICKAMAUGA LAKE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....831 POWER BUILDING
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....E. FORD

LICENSING PROJ MANAGER.....C. STAHLE
DOCKET NUMBER.....50-328

LICENSE & DATE ISSUANCE....DPR-79, SEPTEMBER 15, 1981

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INSPECTION JANUARY 6 - FEBRUARY 5 (84-03): THIS ROUTINE, INSPECTION INVOLVED 83 INSPECTOR HOURS ON SITE IN THE AREAS OF OPERATION SAFETY VERIFICATION, ESF SYSTEM OPERABILITY VERIFICATION, INSPECTION OF TMI ACTION PLAN REQUIREMENTS, LICENSEE EVENT REPORT REVIEW AND INDEPENDENT INSPECTION EFFORT. OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

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CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVII AS IMPLEMENTED BY THE LICENSEE'S QA PROGRAM, REGULATORY GUIDE 1.88, AND ANSI N45.2.9 RECORDS WERE NOT MAINTAINED TO DEMONSTRATE THAT ALL AUDITORS AND LEAD AUDITORS WERE QUALIFIED TO PERFORM SAFETY-RELATED QA AUDITS. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII IMPLEMENTED BY THE LICENSEE'S QA PROGRAM, REGULATORY GUIDE 1.144, AND ANSI N45.2.12, PROCEDURE DID NOT DELINEATE THAT PERSONS CONTACTED DURING THE AUDIT BE IDENTIFIED IN THE AUDIT REPORT.
(8327 5)

10 CFR 50, APPENDIX B, CRITERION X AND THE LICENSEE'S ACCEPTED QUALITY ASSURANCE PROGRAM (TOPICAL REPORT TVA-TR75-1) SECTION 17.2.10 REQUIRE THAT INSPECTION SHALL BE PERFORMED DURING MODIFICATION AFFECTING THE QUALITY OF CRITICAL SYSTEMS, STRUCTURES AND COMPONENTS (CSSC) ITEMS AT TVA PLANTS. MODIFICATION AND ADDITION INSTRUCTION (M&AI)-12 "INTERCONNECTING CABLE TERMINATION AND INSULATION INSPECTION", SECTION 5.0, FURTHER REQUIRES THAT QA INSPECTORS SHALL BE RESPONSIBLE FOR INSPECTING PER THIS PROCEDURE ON CSSC EQUIPMENT. CONTRARY TO THE ABOVE, INSPECTION WAS NOT PERFORMED ON CSSC EQUIPMENT AS REQUIRED BY M&AI-12 SECTION 5 IN THAT DURING THE PERFORMANCE OF WORK PLAN WP10260, WHICH REROUTED SIGNAL CABLES FROM RADIATION MONITORS 1-119, 1-120, 1-121, 2-120 AND 2-121, THE INSPECTION OF THE CABLE TERMINATION PER M&AI-12 WAS PERFORMED BY THE COGNIZANT ENGINEER INSTEAD OF A QA INSPECTOR. THE AFFECTED RADIATION MONITORS ARE CSSC EQUIPMENT. TECHNICAL SPECIFICATION 6.8.1.A REQUIRES THAT WRITTEN PROCEDURES SHALL BE IMPLEMENTED COVERING THE ACTIVITIES REFERENCED IN APPENDIX A OF REGULATORY GUIDE 1.33, REVISION 2, FEBRUARY 1978, INCLUDING DISCHARGING LIQUID RADIOACTIVE WASTE AS EFFLUENTS. SYSTEM OPERATING INSTRUCTION (SOI)-14.3 "CONDENSATE DEMINERALIZER WASTE DISPOSAL" PROVIDES REQUIREMENTS, CONDITIONS, PRECAUTIONS AND INSTRUCTIONS FOR RELEASING THE HIGH CRUD TANKS (HCT). CONTRARY TO THE ABOVE, PROCEDURE SOI-14.3 WAS NOT IMPLEMENTED IN THAT ON OCTOBER 15, 1983, DURING A PLANNED RELEASE FROM HCT "B" TO COOLING TOWER BLOWDOWN, THE TANK WAS PARTIALLY RELEASED TO THE TURBINE BUILDING SUMP BECAUSE THE VALVE ALIGNMENT WAS NOT PROPERLY COMPLETED IN ACCORDANCE WITH THE PROCEDURE. WHEN THE ERROR WAS DISCOVERED, THE RELEASE WAS STOPPED, THE VALVE ALIGNMENT CORRECTED AND THE RELEASE PROPERLY COMPLETED. THE ACTIVITY LEVELS IN THE TANK WERE LESS THAN 10 CFR 20, APPENDIX B, TABLE II LIMITS, THEREFORE, TECHNICAL SPECIFICATION RELEASE LIMITS WERE NOT EXCEEDED. TECHNICAL SPECIFICATION 3.5.1.2 REQUIRES THAT THE UPPER HEAD INJECTION ACCUMULATOR SYSTEM SHALL BE OPERABLE WITH THE UNIT IN MODE 3 ABOVE 1900 PSIG. TECHNICAL SPECIFICATION 3.0.4 REQUIRES THE ENTRY INTO AN OPERATIONAL MODE OR OTHER SPECIFIED CONDITION SHALL NOT BE MADE UNLESS THE CONDITIONS FOR THE LIMITING CONDITION FOR OPERATION ARE MET WITHOUT RELIANCE ON PROVISIONS CONTAINED IN THE ACTION REQUIREMENTS. CONTRARY TO THE ABOVE, UNIT 2 WAS IN MODE 3 AND WENT ABOVE 1900 PSIG ON NOVEMBER 7, 1983, WITHOUT THE UPPER HEAD INJECTION ACCUMULATOR SYSTEM OPERABLE IN THAT REACTOR COOLANT SYSTEM PRESSURE WAS ALLOWED TO INCREASE PRIOR TO PLACING THE SYSTEM IN SERVICE. THE CONDITION FOR LCO 3.5.1.2 WAS NOT MET WITHOUT RELIANCE ON PROVISIONS CONTAINED IN THE ACTION REQUIREMENTS. REACTOR COOLANT SYSTEM PRESSURE WAS QUICKLY REDUCED TO LESS THAN 1900 PSIG WHEN THE ERROR WAS DISCOVERED. PRESSURE WAS ABOVE 1900 PSIG FOR APPROXIMATELY ONE HOUR. UNIT 2 LICENSE CONDITION 2.C.(16).C. REQUIRES THAT PROCEDURES SHALL BE AVAILABLE TO VERIFY THE ADEQUACY OF OPERATING ACTIVITIES IN ACCORDANCE WITH PARAGRAPH I.C.6 OF NUREG 0737. PARAGRAPH I.C.6 REFERENCES SECTION 5.2.6 OF ANSI STANDARD N18.7 WHICH REQUIRES THAT TEMPORARY MODIFICATIONS SUCH AS LIFTING OF LEADS BE CONTROLLED BY APPROVED PROCEDURES WHICH SHALL INCLUDE A REQUIREMENT FOR INDEPENDENT

ENFORCEMENT SUMMARY

VERIFICATION. CONTRARY TO THE ABOVE, ADEQUATE PROCEDURES WERE NOT AVAILABLE TO VERIFY THE ADEQUACY OF OPERATING ACTIVITIES IN THAT ON NOVEMBER 1, 1983, LEADS WERE LIFTED FROM UNIT 2 UPPER HEAD INJECTION LEVELS SWITCHES (L/S) 87-23 AND 87-24 TO SUPPORT CALIBRATION PER SI-196.2. THE LEAD LIFTING WAS NOT CONTROLLED BY SI-196.2 AND THERE WERE NO REQUIREMENT FOR INDEPENDENT VERIFICATION OF THE RETERMINATION. THE WIRES WERE RETERMINATED INCORRECTLY AND CAUSED EQUIPMENT INOPERABILITY. THE ERROR ON L/S 87-23 WAS IDENTIFIED AND CORRECTED ON NOVEMBER 8. THE ERROR ON L/S 87-24 WAS IDENTIFIED AND CORRECTED ON NOVEMBER 15.
(8329 4)

TECHNICAL SPECIFICATION 6.8.1.A REQUIRES THAT WRITTEN PROCEDURES BE ESTABLISHED AND IMPLEMENTED COVERING SAFETY-RELATED ACTIVITIES INCLUDING THE OPERATION OF THE EMERGENCY CORE COOLING SYSTEM (ECCS). SYSTEM OPERATING INSTRUCTION SOI-63.1 "EMERGENCY CORE COOLING SYSTEM" PROVIDES THE STANDBY MODE SYSTEM ADJUSTMENT REQUIREMENTS FOR THE RESIDUAL HEAT REMOVAL (RHR) SYSTEM. CONTRARY TO THE ABOVE, WRITTEN PROCEDURES WERE NOT PROPERLY IMPLEMENTED FOR SAFETY-RELATED ACTIVITIES IN THAT ON DECEMBER 9, 1983 THE INSPECTOR FOUND THAT VALVE 2-FCV-74-520 (2A RHR PUMP DISCHARGE VALVE) WAS OPEN BUT NOT LOCKED AS REQUIRED BY VALVE CHECKLIST 63.1D-1. THE 2A RHR PUMP WAS REQUIRED TO BE OPERABLE. TECHNICAL SPECIFICATION 3.6.1.1 REQUIRES THAT PRIMARY CONTAINMENT INTEGRITY SHALL BE MAINTAINED IN MODES 1, 2, 3 AND 4 AND THAT CONTAINMENT INTEGRITY SHALL BE DEMONSTRATED BY VERIFYING THAT ALL PENETRATIONS REQUIRED TO BE CLOSED DURING ACCIDENT CONDITIONS ARE CLOSED BY VALVES, BLIND FLANGES OR DEACTIVATED AUTOMATIC VALVES SECURED IN THEIR POSITIONS. SURVEILLANCE INSTRUCTION SI-14-2 "VERIFICATION OF CONTAINMENT INTEGRITY" DEMONSTRATES CONTAINMENT INTEGRITY BY VERIFYING THAT CERTAIN VALVES IN THE CONTAINMENT ANNULUS WHICH ISOLATE CONTAINMENT PENETRATIONS ARE LOCKED CLOSED. CONTRARY TO THE ABOVE, CONTAINMENT INTEGRITY WAS NOT PROPERLY DEMONSTRATED IN THAT ON DECEMBER 19, 1983 THE INSPECTOR CHECKED APPROXIMATELY 40 SI-14.2 VALVES IN THE UNIT 2 ANNULUS AND 12 OF THE VALVES WERE NOT LOCKED. ALL VALVES CHECKED WERE IN THEIR PROPER POSITIONS. TECHNICAL SPECIFICATION 6.11 REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. RADIOLOGICAL CONTROL INSTRUCTION RCI-14 "RADIATION WORK PERMIT (RWP) PROGRAM" REQUIRES THAT EACH WORKER ENTERING AN RWP AREA RECORD HIS NAME, SOCIAL SECURITY NUMBER, DATE, TIME AND DOSIMETER READING EACH TIME HE ENTERS AND LEAVES THE AREA AND COMPLY WITH ANY OTHER INSTRUCTIONS OF THE RWP. CONTRARY TO THE ABOVE, PROCEDURES FOR PERSONNEL RADIATION PROTECTION WERE NOT ADHERED TO IN THAT ON DECEMBER 22, 1983, THE INSPECTOR NOTED THAT KEY CARD RECORDS FOR DOOR A-8 TO THE 2A RHR PUMP ROOM SHOWED THAT APPROXIMATELY 200 ENTRIES WERE MADE INTO THE ROOM BETWEEN NOVEMBER 19 AND DECEMBER 8, 1983. REVIEW OF RWP 02-2-00831 TIME SHEETS 34, 35, 36 AND 37 WHICH WERE IN EFFECT FOR THAT TIME PERIOD REVEALED THAT ONLY 34 ENTRIES WERE LOGGED. TECHNICAL SPECIFICATION 3.8.2.1 REQUIRES THE FOUR 120 VOLT A.C. VITAL INSTRUMENT BOARDS BE ENERGIZED FROM THEIR RESPECTIVE INVERTERS IN MODES 1, 2, 3 AND 4. WITH ONE INVERTER INOPERABLE, ENERGIZE THE ASSOCIATED VITAL INSTRUMENT POWER BOARD WITHIN 8 HOURS; RESTORE THE INOPERABLE INVERTER TO OPERABLE STATUS WITHIN 24 HOURS OR BE IN AT LEAST HOT STANDBY WITHIN THE NEXT 6 HOURS AND IN COLD SHUTDOWN WITHIN THE FOLLOWING 30 HOURS. CONTRARY TO THE ABOVE, WITH UNIT 2 IN MODE 1 WHILE PERFORMING MAINTENANCE ON VITAL INVERTER 1-I, THE INVERTER WAS TAKEN OUT OF SERVICE AT 12:08 CST ON 12/20/83, AND NOT RETURNED TO SERVICE UNTIL 8:00 A.M. ON 12/22/83. THIS IS IN EXCESS OF THE 24 HOURS ALLOWED BY THE ACTION STATEMENT TO RESTORE THE INVERTER TO OPERABILITY OR BE IN HOT STANDBY WITHIN THE NEXT 6 HOURS. POWER TO THE VITAL INSTRUMENT POWER BOARD WAS MAINTAINED DURING THIS INVERTS OUTAGE.
(8331 4)

CONTRARY TO 10CFR55, APPENDIX A, PARAGRAPH 3D, EACH LICENSED OPERATOR AND SENIOR OPERATOR HAS NOT REVIEWED THE CONTENTS OF ALL ABNORMAL INSTRUCTIONS (AOI) OR EMERGENCY INSTRUCTIONS (EOI) ON A REGULARLY SCHEDULED BASIS.
(8401 4)

CONTRARY TO 10CFR50, APPENDIX B, CRITERION XIII, THE LICENSEE FAILED TO PROVIDE ADEQUATE SUPPORT OF REACTOR PLANT EQUIPMENT STORED IN OUTSIDE BUILDING 1. FAILURE TO SUPPORT PROPERLY RESULTED IN A 1 3/4 INCH DEFLECTION OF THE WOODEN CONTAINER AND COULD RESULT IN DEFORMATION OF PARTS WITHIN THE CONTAINER.

(8401 5)

OTHER ITEMS

****
* SEQUOYAH 2 *
****

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

100%

LAST IE SITE INSPECTION DATE: JANUARY 6 - FEBRUARY 5, 1984 +

INSPECTION REPORT NO: 50-328/84-03 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-335 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: N. W. GRANT (305) 552-3675

4. Licensed Thermal Power (MWt): 2700

5. Nameplate Rating (Gross MWe): 1000 X 0.89 = 890

6. Design Electrical Rating (Net MWe): 830

7. Maximum Dependable Capacity (Gross MWe): 867

8. Maximum Dependable Capacity (Net MWe): 822

9. If Changes Occur Above Since Last Report, Give Reasons:

---

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	63,048.0
13. Hours Reactor Critical	.0	.0	44,466.3
14. Rx Reserve Shtdwn Hrs	.0	.0	205.3
15. Hrs Generator On-Line	.0	.0	43,576.9
16. Unit Reserve Shtdwn Hrs	.0	.0	39.3
17. Gross Therm Ener (MWH)	0	0	108,667,938
18. Gross Elec Ener (MWH)	0	0	35,373,875
19. Net Elec Ener (MWH)	-2,759	-5,486	33,324,214
20. Unit Service Factor	.0	.0	69.1
21. Unit Avail Factor	.0	.0	69.2
22. Unit Cap Factor (MDC Net)	.0	.0	64.3
23. Unit Cap Factor (DER Net)	.0	.0	63.7
24. Unit Forced Outage Rate	.0	.0	4.6
25. Forced Outage Hours	.0	.0	2,104.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

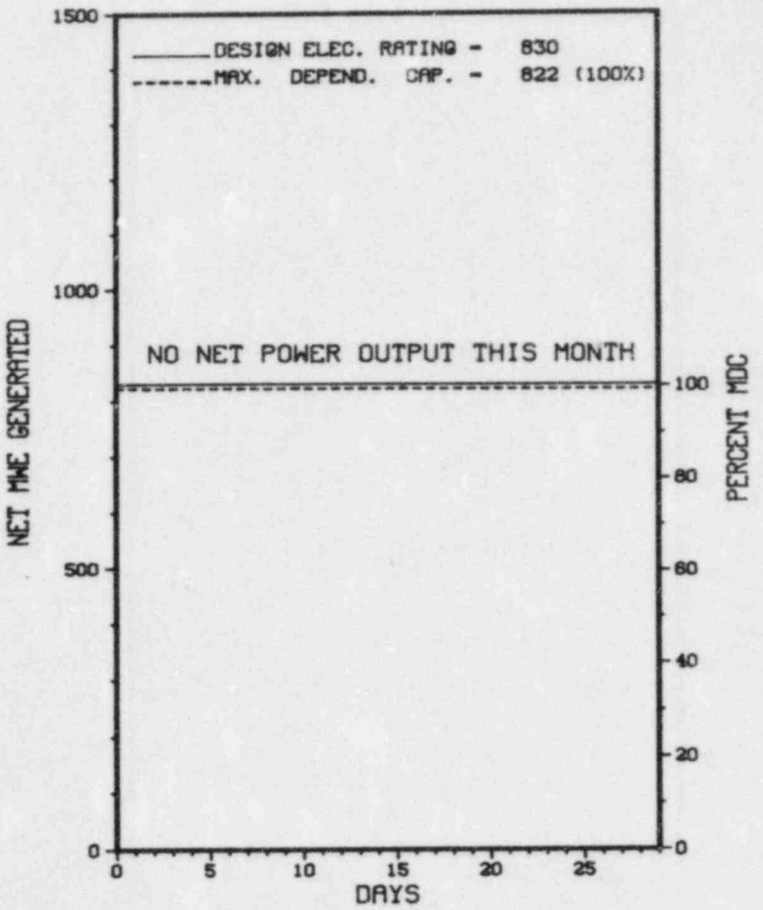
NONE

27. If Currently Shutdown Estimated Startup Date: 04/15/84

****
* ST LUCIE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ST LUCIE 1



FEBRUARY 1984

Report Period FEB 1984 UNIT SHUTDOWNS / REDUCTIONS

****
* ST LUCIE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
3	02/26/83	S	696.0	C	4		RC	FUELXX	UNIT #1 REMAINED OUT OF SERVICE FOR REFUELING AND SCHEDULED MAINTENANCE.

****
* SUMMARY *
****

ST. LUCIE 1 REMAINS OFFLINE IN A CONTINUING REFUELING/ MAINTENANCE OUTAGE.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* ST LUCIE 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....ST LUCIE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...12 MI SE OF
FT. PIERCE, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 22, 1976

DATE ELEC ENER 1ST GENER...MAY 7, 1976

DATE COMMERCIAL OPERATE....DECEMBER 21, 1976

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....ATLANTIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....9250 WEST FLAGLER STREET P.O. BOX 529100
MIAMI, FLORIDA 33174

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR..... ...EBASCO

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....S. ELROD

LICENSING PROJ MANAGER.....D. SELLS
DOCKET NUMBER.....50-335

LICENSE & DATE ISSUANCE....DPR-67, MARCH 1, 1976

PUBLIC DOCUMENT ROOM.....INDIAN RIVER COMMUNITY COLLEGE LIBRARY
3209 VIRGINIA AVENUE
FT. PIERCE, FLORIDA 33450

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 11 - DECEMBER 10 (83-42): THIS ROUTINE INSPECTION INVOLVED 47 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, PLANT OPERATIONS, LICENSEE EVENT REPORTS AND IE INFORMATION NOTICES. OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 11, 1983 - JANUARY 10, 1984 (83-43): THIS ROUTINE INSPECTION INVOLVED 59 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF MAINTENANCE, SURVEILLANCE, PLANT OPERATIONS, AND TMI ACTION ITEM I.A.1.3. OF THE FOUR AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT VIOLATION WAS IDENTIFIED IN ONE AREA, FAILURE TO MAINTAIN PROCEDURES (PARAGRAPH 5).

INSPECTION JANUARY 9-13 (84-01): THE INSPECTION INVOLVED 16 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 7 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS AND IE INFORMATION NOTICES; SECURITY ORGANIZATION - MANAGEMENT/ PERSONNEL/RESPONSE; SECURITY PROGRAM AUDIT; PHYSICAL BARRIERS - PROTECTED AREA; PHYSICAL BARRIERS - VITAL AREAS; ASSESSMENT AIDS; ACCESS CONTROL - PERSONNEL; ACCESS CONTROL - PACKAGES; ACCESS CONTROL - VEHICLES; DETECTION AIDS - PROTECTED AREA; DETECTION AIDS - VITAL AREAS; ALARM STATIONS; AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 15 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION JANUARY 18-20 (84-03): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR HOURS ON SITE IN THE AREAS OF EMERGENCY PREPAREDNESS TO ASSESS ON-SHIFT PROMPT EMERGENCY CLASSIFICATION AND PROTECTIVE ACTION RECOMMENDATIONS. OF THE AREAS

****
* ST LUCIE 1 *
****

INSPECTION SUMMARY

INSPECTED NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 1-3 (84-04): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR HOURS ON SITE IN THE AREAS OF MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, PLANT OPERATIONS, PREOPERATIONAL TEST. 'G, PROCEDURES, LOW POWER TESTING, POWER ASCENSION, PLANT MODIFICATION. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 6-10 (84-05): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 18 INSPECTOR HOURS ON SITE IN THE AREAS OF HEALTH PHYSICS INSTRUMENTATION, ACTION ON NUREG 0737 ITEMS, ALARA ACTIVITIES, AND FOLLOW-UP ON PREVIOUS INSPECTOR FOLLOWUP ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN THE AREAS INSPECTED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

DURING REFUELING OUTAGE, THE THERMAL SHIELD WITHIN THE REACTOR VESSEL WAS FOUND TO BE BROKEN. THE SHIELD IS BEING REMOVED.

FACILITY ITEMS (PLANS AND PROCEDURES):

EXTENDED OUTAGE, RESTART PLANNED IN EARLY 1984.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ REFUELING.

LAST IE SITE INSPECTION DATE: FEBRUARY 6-10, 1984 +

INSPECTION REPORT NO: 50-335/84-05 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-389 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: N. W. GRANT (305) 552-3675
4. Licensed Thermal Power (MWt): 2560
5. Nameplate Rating (Gross MWe): 0850
6. Design Electrical Rating (Net MWe): 804
7. Maximum Dependable Capacity (Gross MWe): 832
8. Maximum Dependable Capacity (Net MWe): 786
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	4,945.0
13. Hours Reactor Critical	687.5	1,421.4	4,648.4
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	684.5	1,285.6	4,416.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,721,996	3,197,631	10,855,575
18. Gross Elec Ener (MWH)	578,830	1,075,530	3,618,750
19. Net Elec Ener (MWH)	547,789	1,013,681	3,411,267
20. Unit Service Factor	98.3	89.3	89.3
21. Unit Avail Factor	98.3	89.3	89.3
22. Unit Cap Factor (MDC Net)	100.1	89.6	87.8
23. Unit Cap Factor (DER Net)	97.9	87.6	85.8
24. Unit Forced Outage Rate	1.7	8.8	10.2
25. Forced Outage Hours	11.5	124.3	498.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

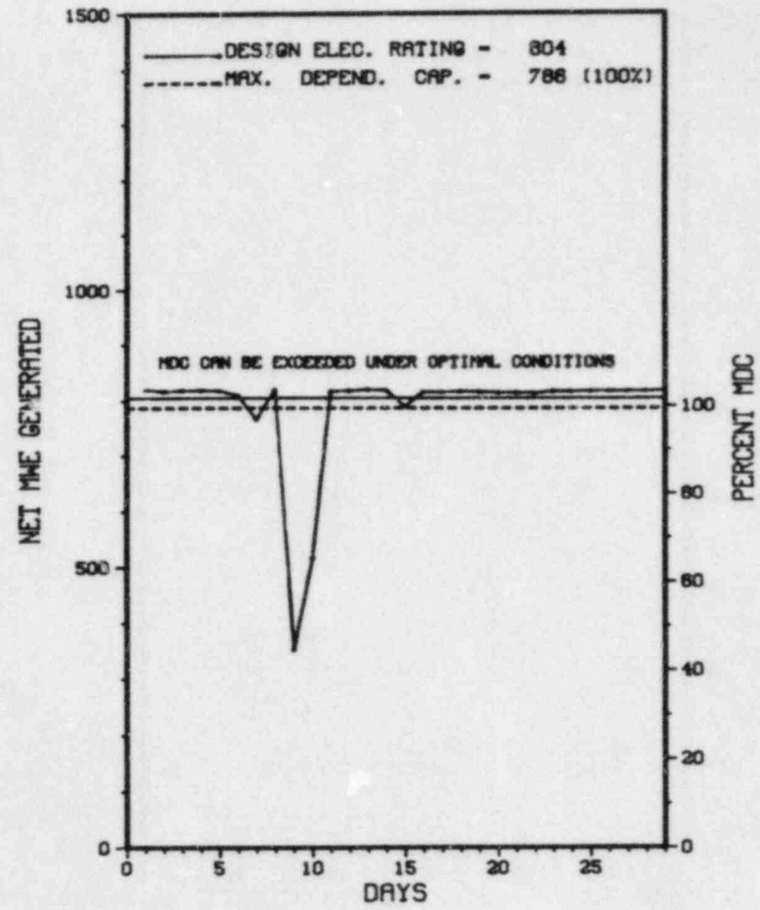
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ST LUCIE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ST LUCIE 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

ST LUCIE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
06	02/09/84	F	11.5	A	3	84-04	HH	PUMPXX	MAIN FEEDWATER PUMP TRIPPED DUE TO LOW SUCTION PRESSURE, RESULTING IN REACTOR TRIP. A VENT PROBLEM WITH THE CONDENSATE PUMPS WAS THE CAUSE. THE DESIGN IS BEING CHANGED TO PREVENT RECURRENCE.
07	02/09/84	F	0.0	H	5		HH	ZZZZZZ	HIGH STEAM GENERATOR CHLORIDE LEVEL CAUSED POWER REDUCTION.

SUMMARY

ST. LUCIE 2 OPERATED WITH 1 OUTAGE AND 1 REDUCTION DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Trainirg & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

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* ST LUCIE 2 *
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FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....ST LUCIE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...12 MI SE OF
FT. PIERCE, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 2, 1983

DATE ELEC ENER 1ST GENER...JUNE 13, 1983

DATE COMMERCIAL OPERATE....AUGUST 8, 1983

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....ATLANTIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....P.O. BOX 013100
MIAMI, FLORIDA 33101

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....S. ELROD

LICENSING PROJ MANAGER.....D. SELLS
DOCKET NUMBER.....50-389

LICENSE & DATE ISSUANCE...., JUNE 10, 1983

PUBLIC DOCUMENT ROOM.....INDIAN RIVER COMMUNITY COLLEGE LIBRARY
3209 VIRGINIA AVENUE
FT. PIERCE, FLORIDA 33450

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 11 - DECEMBER 10 (83-70): THIS ROUTINE INSPECTION INVOLVED 48 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, PLANT OPERATIONS, LICENSEE EVENT REPORTS, AND IE INFORMATION NOTICES. OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 11, 1983 - JANUARY 10, 1984 (83-71): THIS ROUTINE INSPECTION INVOLVED 60 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF MAINTENANCE, SURVEILLANCE, PLANT OPERATIONS, AND TMI ACTION ITEM I.A.1.3. OF THE FOUR AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT VIOLATION WAS IDENTIFIED IN ONE AREA, FAILURE TO MAINTAIN PROCEDURES (PARAGRAPH 5).

INSPECTION JANUARY 9-13 (84-02): THE INSPECTION INVOLVED 16 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 7 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE INSPECTION INCLUDED LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS AND IE INFORMATION NOTICES; SECURITY ORGANIZATION - MANAGEMENT/ PERSONNEL/RESPONSE; SECURITY PROGRAM AUDIT; PHYSICAL BARRIERS - PROTECTED AREA; PHYSICAL BARRIERS - VITAL AREAS; ASSESSMENT AIDS; ACCESS CONTROL - PERSONNEL; ACCESS CONTROL - PACKAGES; ACCESS CONTROL - VEHICLES; DETECTION AIDS - PROTECTED AREA; DETECTION AIDS - VITAL AREAS; ALARM STATIONS; AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 15 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION JANUARY 18-20 (84-04): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR HOURS ON SITE IN THE AREAS OF EMERGENCY PREPAREDNESS TO ASSESS ON-SHIFT PROMPT EMERGENCY CLASSIFICATION AND PROTECTIVE ACTION RECOMMENDATIONS. OF THE AREAS

INSPECTION SUMMARY

INSPECTED NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 1-3 (84-05): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR HOURS ON SITE IN THE AREAS OF MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, PLANT OPERATIONS, PREOPERATIONAL TESTING, PROCEDURES, LOW POWER TESTING, POWER ASCENSION, PLANT MODIFICATION. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 6-10 (84-07): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 18 INSPECTOR HOURS ON SITE IN THE AREAS OF HEALTH PHYSICS INSTRUMENTATION, ACTION ON NUREG 0737 ITEMS, ALARA ACTIVITIES, AND FOLLOW-UP ON PREVIOUS INSPECTOR FOLLOWUP ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN THE AREAS INSPECTED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

PERFORMING STARTUP TESTING.

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: FEBRUARY 6-10, 1984 +

INSPECTION REPORT NO: 50-389/84-07 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-395 O P E R A T I N G S T A T U S
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: G. A. LOIGNON (803) 345-5209
4. Licensed Thermal Power (MWt): 2775
5. Nameplate Rating (Gross MWe): 0900
6. Design Electrical Rating (Net MWe): 900
7. Maximum Dependable Capacity (Gross MWe): 900
8. Maximum Dependable Capacity (Net MWe): 900
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	1,440.0
13. Hours Reactor Critical	641.8	1,376.8	1,376.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	611.4	1,344.2	1,344.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,666,309	3,663,244	3,663,244
18. Gross Elec Ener (MWH)	554,195	1,222,545	1,222,545
19. Net Elec Ener (MWH)	531,912	1,173,800	1,173,800
20. Unit Service Factor	87.8	93.3	93.3
21. Unit Avail Factor	87.8	93.3	93.3
22. Unit Cap Factor (MDC Net)	84.9	90.6	90.6
23. Unit Cap Factor (DER Net)	84.9	90.6	90.6
24. Unit Forced Outage Rate	12.2	6.7	6.7
25. Forced Outage Hours	84.6	95.8	95.8

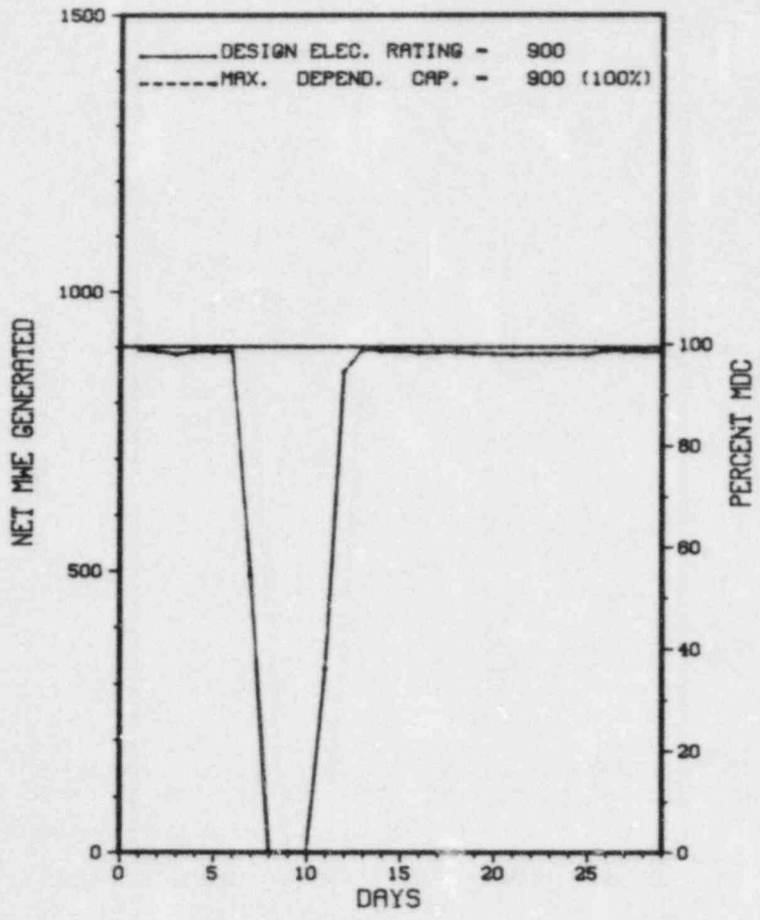
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
SPRING MAINTENANCE, MARCH 22, 1984, 29 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SUMMER 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SUMMER 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* SUMMER 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2	02/07/84	F	25.4	A	3				REACTOR TRIP FROM "B" STEAM GENERATOR LO-LO LEVEL.
3	02/08/84	F	59.2	A	1				TRIPPED TURBINE BECAUSE FEEDWATER TEMPERATURE DECREASED TO BELOW 225 DEGREES F CLOSING FEEDWATER ISOLATION VALVES.

****
* SUMMARY *
****

SUMMER 1 OPERATED AT FULL POWER WITH 2 OUTAGES DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SUMMER 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....FAIRFIELD

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...26 MI NW OF
COLUMBIA, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 22, 1982

DATE ELEC ENER 1ST GENER...NOVEMBER 16, 1982

DATE COMMERCIAL OPERATE....JANUARY 1, 1984

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MONTICELLO RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTH CAROLINA ELECTRIC & GAS CO.

CORPORATE ADDRESS.....P.O. BOX 764
COLUMBIA, SOUTH CAROLINA 29202

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DANIEL INTERNATIONAL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR..... .

LICENSING PROJ MANAGER.....J. HOPKINS
DOCKET NUMBER.....50-395

LICENSE & DATE ISSUANCE....NPF-12, NOVEMBER 12, 1982

PUBLIC DOCUMENT ROOM.....FAIRFIELD COUNTY LIBRARY
GARDEN & WASHINGTON STREETS
WINNSBORO, SOUTH CAROLINA 29180

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JANUARY 16-19 (84-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 79 INSPECTOR HOURS ON SITE IN THE AREAS OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF THE LABORATORY QUALITY CONTROL PROGRAM; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; AIRBORNE EFFLUENT SAMPLING METHODOLOGY; AND COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC REGION II MOBILE LABORATORY. OF THE FOUR AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 16-20 (84-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 69 INSPECTOR HOURS ON SITE IN THE AREAS OF EMERGENCY PREPAREDNESS. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 10-13 (84-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 32 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY VERIFICATION, MAINTENANCE AND SURVEILLANCE ACTIVITIES, PHYSICAL PROTECTION, TECHNICAL SPECIFICATION COMPLIANCE, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, AND FOLLOW-UP OF LICENSEE EVENT REPORTS. OF THE 6 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JANUARY 16-20, 1984 +

INSPECTION REPORT NO: 50-395/84-02 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-280 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: VIVIAN M. JONES (804) 357-3184

4. Licensed Thermal Power (MWt): 2441

5. Nameplate Rating (Gross MWe): 942 X 0.9 = 848

6. Design Electrical Rating (Net MWe): 788

7. Maximum Dependable Capacity (Gross MWe): 811

8. Maximum Dependable Capacity (Net MWe): 775

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	98,088.0
13. Hours Reactor Critical	571.8	1,310.7	60,409.7
14. Rx Reserve Shtdwn Hrs	4.2	9.3	3,774.5
15. Hrs Generator On-Line	566.5	1,280.0	59,146.8
16. Unit Reserve Shtdwn Hrs	.0	.0	3,736.2
17. Gross Therm Ener (MWH)	1,342,137	3,026,287	137,426,900
18. Gross Elec Ener (MWH)	433,030	972,425	44,292,268
19. Net Elec Ener (MWH)	411,451	923,876	42,001,612
20. Unit Service Factor	81.4	88.9	60.3
21. Unit Avail Factor	81.4	88.9	64.1
22. Unit Cap Factor (MDC Net)	76.3	82.8	55.3
23. Unit Cap Factor (DER Net)	75.0	81.4	54.3
24. Unit Forced Outage Rate	1.6	3.0	21.3
25. Forced Outage Hours	9.3	39.8	12,251.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

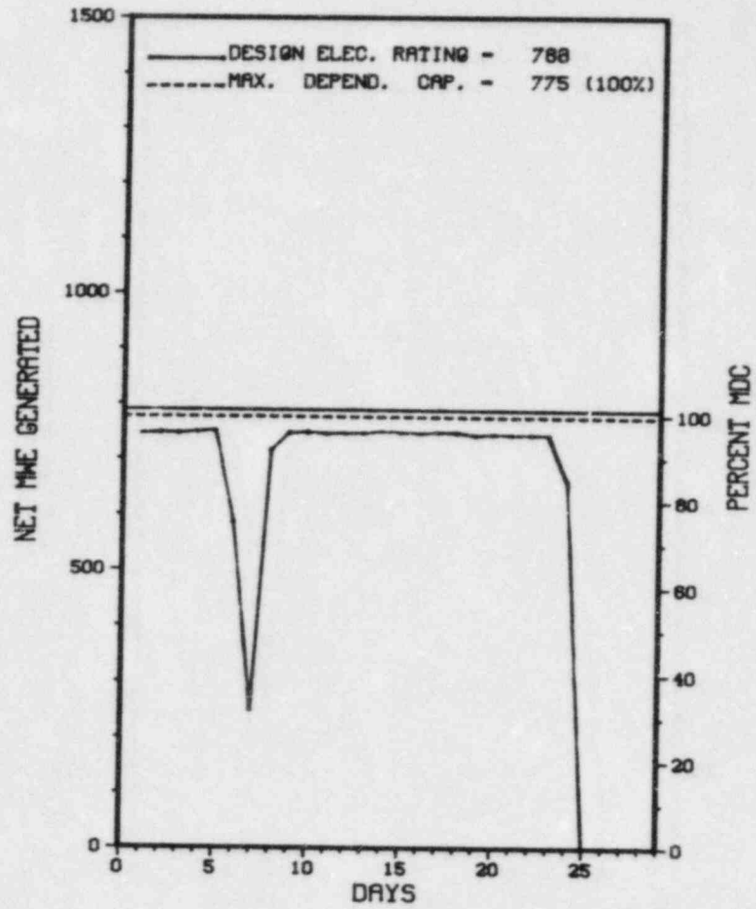
WINTER MAINTENANCE - 2-24-84 - 2 WEEKS.

27. If Currently Shutdown Estimated Startup Date: 03/09/84

****
* SURRY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SURRY 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* SURRY 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-3	02/06/84	F	9.3	A	3	84-003-00	LD	PSF	REACTOR TRIP CAUSED BY "A" S/G LOW LEVEL WITH A STEAM FLOW/FEED FLOW MISMATCH. THE INITIATING EVENT WAS A LOSS OF CONTROL POWER TO UNIT 1 POLISHERS AND THE SUBSEQUENT RE-ENERGIZING OF THE TRIPPED BREAKER. RE-ENERGIZING CONTROL POWER CAUSED THE AOV'S FOR THE POLISH BEDS TO GO CLOSED CAUSING A REDUCTION IN FEEDWATER FLOW. THE CORRECTIVE ACTION WAS TO REPLACE THE BREAKER THAT TRIPPED CAUSING A LOSS OF CONTROL POWER. ALSO A SIGN WILL BE POSTED BY THE BREAKER STATING THAT "DO NOT RE-ENERGIZE BREAKER UNTIL POLISH BLDG. IS BYPASSED."
84-4	02/24/84	S	120.2	H	1				SHUTDOWN FOR SNUBBER OUTAGE AND REPAIRS.

****
* SUMMARY *
****

SURRY 1 OPERATED ROUTINELY, SHUTTING DOWN ON FEBRUARY 24TH FOR MAINTENANCE AND REPAIRS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SURRY 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....SURRY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI NW OF
NEWPORT NEWS, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 1, 1972

DATE ELEC ENER 1ST GENER...JULY 4, 1972

DATE COMMERCIAL OPERATE....DECEMBER 22, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....JAMES RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA ELECTRIC & POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. BURKE

LICENSING PROJ MANAGER.....D. NEIGHBORS
DOCKET NUMBER.....50-280

LICENSE & DATE ISSUANCE....DPR-32, MAY 25, 1972

PUBLIC DOCUMENT ROOM.....SWEM LIBRARY
COLLEGE OF WILLIAM AND MARY
WILLIAMSBURG, VIRGINIA 23185

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 1-31 (83-37): THIS INSPECTION INVOLVED 95 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS, PLANT SECURITY, FOLLOW-UP OF EVENTS AND LICENSEE EVENT REPORTS, PERFORMANCE APPRAISAL STAFF INSPECTION FINDINGS, MAINTENANCE AND SURVEILLANCE REVIEW. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 23-25 (84-01): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR HOURS ON SITE IN THE AREAS OF EMERGENCY PREPAREDNESS. OF THE AREAS INSPECTED TWO VIOLATIONS WERE FOUND IN TWO AREAS (INADEQUATE PROCEDURES FOR PROTECTIVE ACTION DECISIONMAKING AND FAILURE TO PROPERLY TRAIN EMERGENCY PERSONNEL IN PROTECTIVE ACTION RECOMMENDATION DECISIONMAKING). THESE ARE DISCUSSED IN PARAGRAPH 5.

INSPECTION JANUARY 30 - FEBRUARY 2 (84-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, IE BULLETIN 81-01, AND FOLLOWUP ON LICENSEE IDENTIFIED ITEMS. OF THE 3 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE APPARENT VIOLATION WAS FOUND IN ONE AREA (FAILURE TO PERFORM ADDITIONAL FUNCTIONAL TESTS ON SNUBBERS PER TECHNICAL SPECIFICATION REQUIREMENTS - PARAGRAPH 3).

INSPECTION JANUARY 3-31 (84-04): THIS INSPECTION INVOLVED 100 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MAINTENANCE AND SURVEILLANCE, PLANT SECURITY, FOLLOWUP OF EVENTS AND LICENSEE EVENT REPORTS. OF THE FOUR AREAS INSPECTED, NO VIOLATIONS WERE IDENTIFIED IN THREE AREAS, AND ONE VIOLATION WAS IDENTIFIED IN ONE AREA; (FAILURE TO PROPERLY MONITOR A RADIOACTIVE GASEOUS WASTE RELEASE - PARAGRAPH 5.B).

ENFORCEMENT SUMMARY

10 CFR 20.201 REQUIRES THAT EACH LICENSEE MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS (1) MAY BE NECESSARY FOR THE LICENSEE TO COMPLY WITH THE REGULATIONS IN THIS PART, AND (2) ARE REASONABLE UNDER THE CIRCUMSTANCES TO EVALUATE THE EXTENT OF RADIATION HAZARDS THAT MAY BE PRESENT. 10 CFR 20.202 REQUIRES THAT EACH LICENSEE SUPPLY APPROPRIATE PERSONNEL MONITORING EQUIPMENT TO, AND SHALL REQUIRE THE USE OF SUCH EQUIPMENT BY EACH INDIVIDUAL WHO ENTERS A RESTRICTED AREA UNDER SUCH CIRCUMSTANCES THAT HE RECEIVES, OR IS LIKELY TO RECEIVE A DOSE IN ANY CALENDAR QUARTER IN EXCESS OF 25% OF THE APPLICABLE VALUE SPECIFIED IN PARAGRAPH (A) OF 20.101. CONTRARY TO THE ABOVE, THE REQUIREMENT TO PERFORM AN EVALUATION TO ENSURE COMPLIANCE WITH APPLICABLE REQULATIONS WAS NOT MET IN THAT ON NOVEMBER 3, 1983, A THERMOLUMINESCENT DOSIMETER WAS NOT PLACED ON A WORKER INSTALLING SHORING IN THE SHIPPING CASK FOR A LET DOWN FILTER IN A POSITION SUCH THAT IT MONITORED THE PART OF THE WHOLE BODY RECEIVING THE HIGHEST DOSE. IN ADDITION, THE REQUIREMENT TO PROVIDE APPROPRIATE MONITORING DEVICES WAS NOT MET IN THAT THE WORKER'S EXTREMITIES WERE NOT MONITORED. (8334 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JANUARY 3-31, 1984 +

INSPECTION REPORT NO: 50-280/84-04 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-281 OPERATING STATUS

2 Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: VIVIAN H. JONES (804) 357-3184

4. Licensed Thermal Power (MWt): 2441

5. Nameplate Rating (Gross MWe): 942 X 0.9 = 848

6. Design Electrical Rating (Net MWe): 788

7. Maximum Dependable Capacity (Gross MWe): 811

8. Maximum Dependable Capacity (Net MWe): 775

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	94,968.0
13. Hours Reactor Critical	696.0	1,416.2	59,986.8
14. Rx Reserve Shtdwn Hrs	.0	23.8	23.8
15. Hrs Generator On-Line	696.0	1,407.1	58,983.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,659,218	3,375,408	138,091,280
18. Gross Elec Ener (MWH)	530,760	1,083,220	44,873,079
19. Net Elec Ener (MWH)	503,388	1,027,846	42,534,906
20. Unit Service Factor	100.0	97.7	62.1
21. Unit Avail Factor	100.0	97.7	62.1
22. Unit Cap Factor (MDC Net)	93.3	92.1	57.8
23. Unit Cap Factor (DER Net)	91.8	90.6	56.8
24. Unit Forced Outage Rate	.0	2.3	13.9
25. Forced Outage Hours	.0	32.9	6,859.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

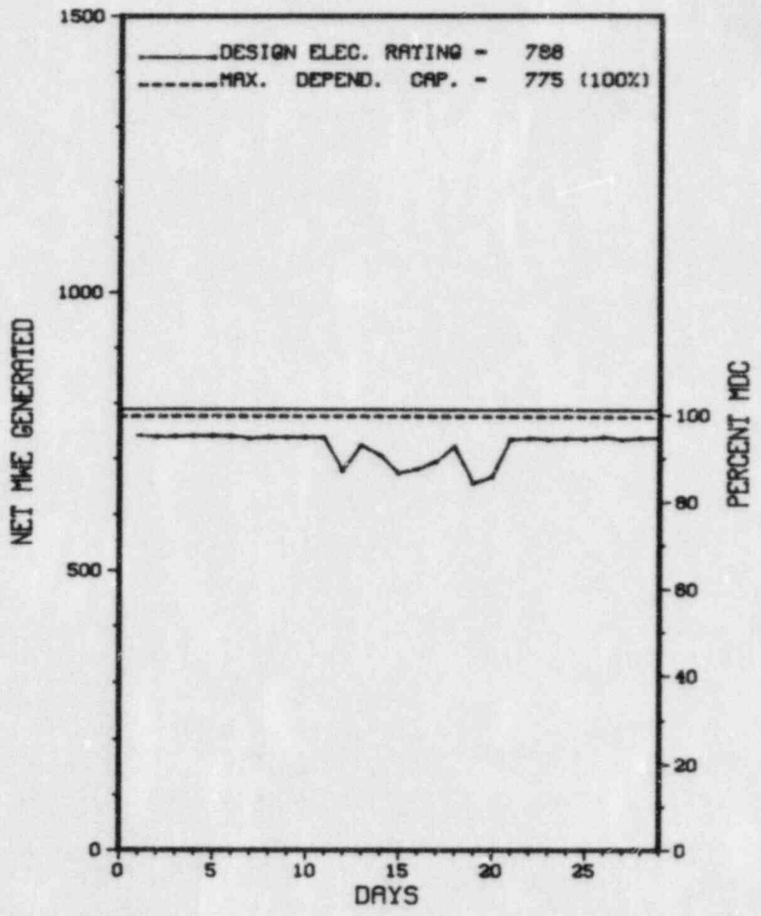
SPRING MAINTENANCE 3-16-84 - 14 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SURRY 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SURRY 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* SURRY 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-3	02/11/84	S	0.0	H	5				POWER WAS REDUCED TO 65% (505 MW'S) FOR LOAD FOLLOWING.
84-4	02/12/84	S	0.0	H	5				POWER WAS REDUCED TO 76% (615 MW'S) FOR LOAD FOLLOWING.
84-5	02/14/84	S	0.0	H	5				POWER WAS REDUCED TO 64% (500 MW'S) FOR LOAD FOLLOWING.
84-6	02/15/84	S	0.0	H	5				POWER WAS REDUCED TO 64% (500 MW'S) FOR LOAD FOLLOWING.
84-7	02/17/84	S	0.0	H	5				POWER WAS REDUCED TO 64% (500 MW'S) FOR LOAD FOLLOWING.
84-8	02/18/84	S	0.0	H	5				POWER WAS REDUCED TO 64% (500 MW'S) FOR LOAD FOLLOWING.
84-9	02/19/84	S	0.0	H	5				POWER WAS REDUCED TO 64% (500 MW'S) FOR LOAD FOLLOWING.

****
* SUMMARY *
****

SURRY 2 OPERATED WITH 7 REDUCTIONS AND NO OUTAGES DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SURRY 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....SURRY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI NW OF
NEWPORT NEWS, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MARCH 7, 1973

DATE ELEC ENER 1ST GENER...MARCH 10, 1973

DATE COMMERCIAL OPERATE....MAY 1, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....JAMES RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA ELECTRIC & POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. BURKE

LICENSING PROJ MANAGER.....D. NEIGHBORS
DOCKET NUMBER..... 50-281

LICENSE & DATE ISSUANCE....DPR-37, JANUARY 29, 1973

PUBLIC DOCUMENT ROOM.....SWEM LIBRARY
COLLEGE OF WILLIAM AND MARY
WILLIAMSBURG, VIRGINIA 23185

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION DECEMBER 1-31 (83-39): THIS INSPECTION INVOLVED 95 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS, PLANT SECURITY, FOLLOW-UP OF EVENTS AND LICENSEE EVENT REPORTS, PERFORMANCE APPRAISAL STAFF INSPECTION FINDINGS, MAINTENANCE AND SURVEILLANCE REVIEW. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 23-25 (84-01): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR HOURS ON SITE IN THE AREAS OF EMERGENCY PREPAREDNESS. OF THE AREAS INSPECTED TWO VIOLATIONS WERE FOUND IN TWO AREAS (INADEQUATE PROCEDURES FOR PROTECTIVE ACTION DECISIONMAKING AND FAILURE TO PROPERLY TRAIN EMERGENCY PERSONNEL IN PROTECTIVE ACTION RECOMMENDATION DECISIONMAKING). THESE ARE DISCUSSED IN PARAGRAPH 5.

INSPECTION JANUARY 30 - FEBRUARY 2 (84-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, IE BULLETIN 81-01, AND FOLLOWUP ON LICENSEE IDENTIFIED ITEMS. OF THE 3 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE APPARENT VIOLATION WAS FOUND IN ONE AREA (FAILURE TO PERFORM ADDITIONAL FUNCTIONAL TESTS ON SNUBBERS PER TECHNICAL SPECIFICATION REQUIREMENTS - PARAGRAPH 3).

INSPECTION JANUARY 3-31 (84-04): THIS INSPECTION INVOLVED 100 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MAINTENANCE AND SURVEILLANCE, PLANT SECURITY, FOLLOWUP OF EVENTS AND LICENSEE EVENT REPORTS. OF THE FOUR AREAS INSPECTED, NO VIOLATIONS WERE IDENTIFIED IN THREE AREAS, AND ONE VIOLATION WAS IDENTIFIED IN ONE AREA; (FAILURE TO PROPERLY MONITOR A RADIOACTIVE GASEOUS WASTE RELEASE - PARAGRAPH 5.B).

ENFORCEMENT SUMMARY

10 CFR 20.201 REQUIRES THAT EACH LICENSEE MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS (1) MAY BE NECESSARY FOR THE LICENSEE TO COMPLY WITH THE REGULATIONS IN THIS PART, AND (2) ARE REASONABLE UNDER THE CIRCUMSTANCES TO EVALUATE THE EXTENT OF RADIATION HAZARDS THAT MAY BE PRESENT. 10 CFR 20.202 REQUIRES THAT EACH LICENSEE SUPPLY APPROPRIATE PERSONNEL MONITORING EQUIPMENT TO, AND SHALL REQUIRE THE USE OF SUCH EQUIPMENT BY EACH INDIVIDUAL WHO ENTERS A RESTRICTED AREA UNDER SUCH CIRCUMSTANCES THAT HE RECEIVES, OR IS LIKELY TO RECEIVE A DOSE IN ANY CALENDAR QUARTER IN EXCESS OF 25% OF THE APPLICABLE VALUE SPECIFIED IN PARAGRAPH (A) OF 20.101. CONTRARY TO THE ABOVE, THE REQUIREMENT TO PERFORM AN EVALUATION TO ENSURE COMPLIANCE WITH APPLICABLE REQULATIONS WAS NOT MET IN THAT ON NOVEMBER 3, 1983, A THERMOLUMINESCENT DOSIMETER WAS NOT PLACED ON A WORKER INSTALLING SHORING IN THE SHIPPING CASK FOR A LET DOWN FILTER IN A POSITION SUCH THAT IT MONITORED THE PART OF THE WHOLE BODY RECEIVING THE HIGHEST DOSE. IN ADDITION, THE REQUIREMENT TO PROVIDE APPROPRIATE MONITORING DEVICES WAS NOT MET IN THAT THE WORKER'S EXTREMITIES WERE NOT MONITORED. (8336 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN FOR REFUELING AND 10 YEAR IN-SERVICE INSPECTION (ISI).

LAST IE SITE INSPECTION DATE: JANUARY 3-31, 1984 +

INSPECTION REPORT NO: 50-281/84-04 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-387 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: L. A. KUCZYNSKI (717) 542-2181
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1065
7. Maximum Dependable Capacity (Gross MWe): 1068
8. Maximum Dependable Capacity (Net MWe): 1032
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	6,409.0
13. Hours Reactor Critical	159.9	159.9	4,005.2
14. Rx Reserve Shtdwn Hrs	.0	.0	156.7
15. Hrs Generator On-Line	98.2	98.2	3,866.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	163,201	163,201	11,412,972
18. Gross Elec Ener (MWH)	43,470	43,470	3,710,020
19. Net Elec Ener (MWH)	40,180	40,180	3,576,553
20. Unit Service Factor	14.1	6.8	60.3
21. Unit Avail Factor	14.1	6.8	60.3
22. Unit Cap Factor (MDC Net)	5.6	2.7	54.1
23. Unit Cap Factor (DER Net)	5.4	2.6	52.4
24. Unit Forced Outage Rate	46.8	46.8	13.3
25. Forced Outage Hours	86.3	86.3	594.8

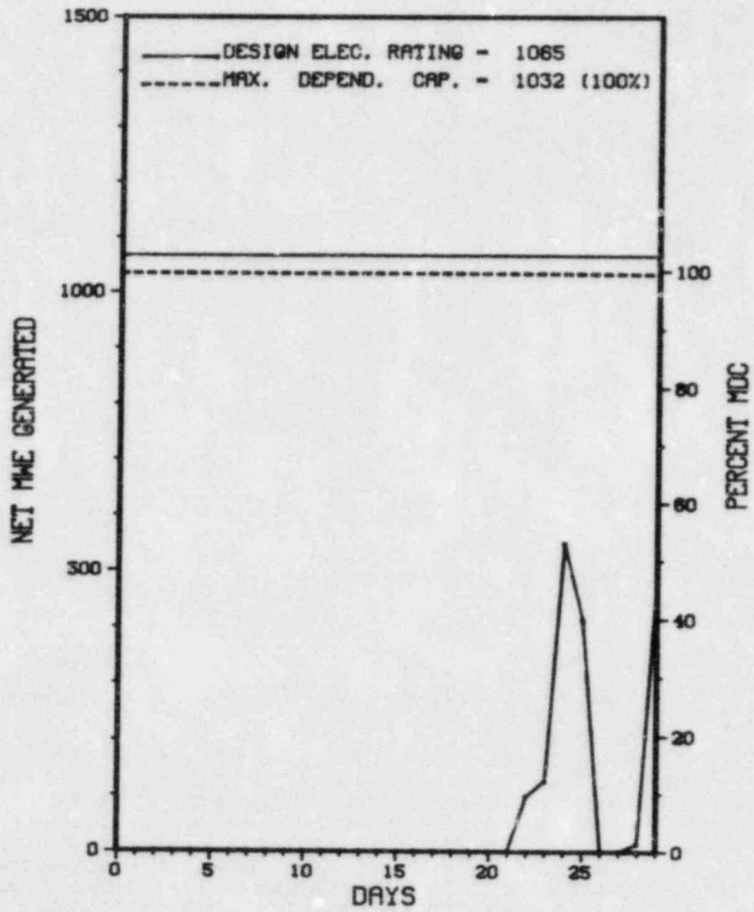
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SUSQUEHANNA 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SUSQUEHANNA 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

SUSQUEHANNA 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
20	12/03/83	S	511.5	H	2		ZZ	ZZZZZZ	MANUAL SCRAM FROM 33% REACTOR POWER TO COMMENCE UNIT 1-UNIT 2 TIE-IN OUTAGE. NO ACTION REQUIRED TO PREVENT RECURRENCE; THIS WAS A SCHEDULED EVENT. OUTAGE COMPLETED AT 0728 ON 2/22/84.
1A	02/22/84	F	12.2	A	2				GENERATOR REMOVED FROM GRID FOR 12.2. HRS. ON FEB 22-23, 1984, DUE TO GENERATOR 'C' PHASE OUTPUT DISCONNECT BREAKER SWITCH IMPROPER CLOSURE CAUSING BREAKER OVERHEATING.
1B	02/25/84	F	74.1	A	2	84-010	SF	VALVOP	MANUAL SCRAM FROM 54% POWER DUE TO A SAFETY RELIEF VALVE (SRV) FAILURE TO CLOSE DURING TESTING. UNIT RESPONDED TO SCRAM AS DESIGNED. ONE SRV SOLENOID VALVE WAS REPLACED, SUCCESSFULLY RETESTED AND THE SYSTEM RETURNED TO SERVICE.

SUMMARY

SUSQUEHANNA 1 RETURNED ONLINE FEBRUARY 22ND FROM UNIT 1&2 TIE-IN OUTAGE AND OPERATED WITH ONE OUTAGE DUE TO EQUIPMENT FAILURE ON THE 25TH.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SUSQUEHANNA 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....LUZERNE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...7 MI NE OF
BERWICK, PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 10, 1982

DATE ELEC ENER 1ST GENER...NOVEMBER 16, 1982

DATE COMMERCIAL OPERATE....JUNE 8, 1983

CONDENSER COOLING METHOD...CC,HNDCT

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PENNSYLVANIA POWER & LIGHT

CORPORATE ADDRESS.....2 NORTH NINTH STREET
ALLENTOWN, PENNSYLVANIA 18101

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....R. JACOBS

LICENSING PROJ MANAGER.....R. PERCH
DOCKET NUMBER.....50-387

LICENSE & DATE ISSUANCE....., NOVEMBER 12, 1982

PUBLIC DOCUMENT ROOM.....OSTERHOUT FREE LIBRARY
71 SOUTH FRANKLIN STREET
WILKES-BARRE, PENNSYLVANIA 18701

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* SUSQUEHANNA 1 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

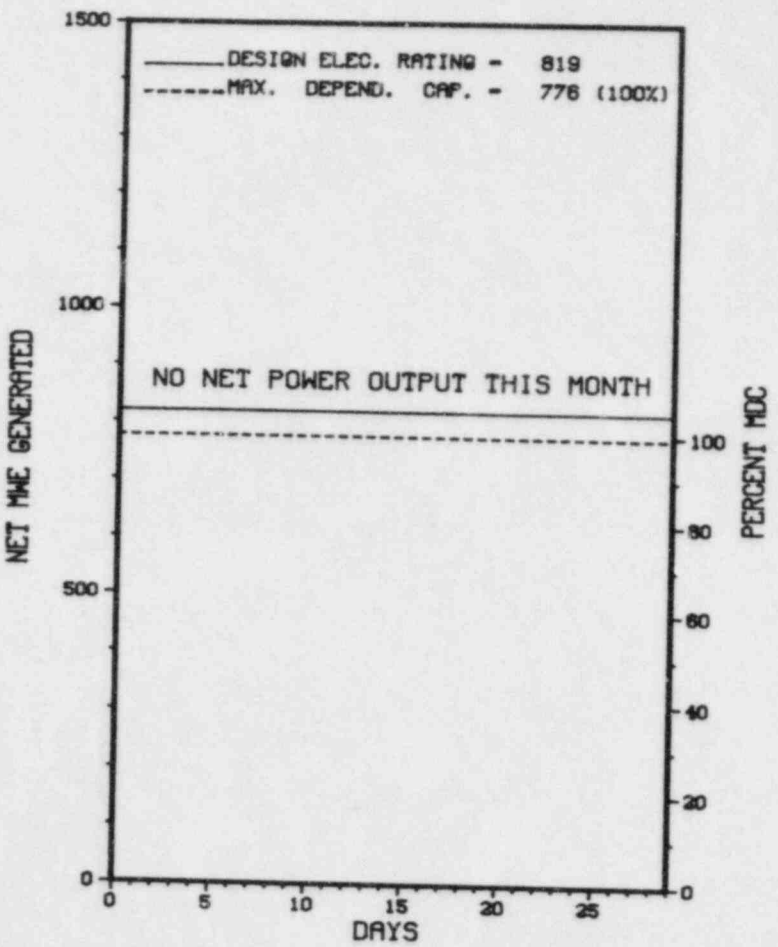
1. Docket: 50-289 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: C. W. SMYTH (717) 948-8551
4. Licensed Thermal Power (MWt): 2535
5. Nameplate Rating (Gross MWe): 968 X 0.9 = 871
6. Design Electrical Rating (Net MWe): 819
7. Maximum Dependable Capacity (Gross MWe): 840
8. Maximum Dependable Capacity (Net MWe): 776
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	83,233.0
13. Hours Reactor Critical	.0	.0	31,731.8
14. Rx Reserve Shtdwn Hrs	.0	.0	839.5
15. Hrs Generator On-Line	.0	.0	31,180.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	76,531,071
18. Gross Elec Ener (MWH)	0	0	25,484,330
19. Net Elec Ener (MWH)	0	0	23,840,053
20. Unit Service Factor	.0	.0	37.5
21. Unit Avail Factor	.0	.0	37.5
22. Unit Cap Factor (MDC Net)	.0	.0	36.6*
23. Unit Cap Factor (DER Net)	.0	.0	35.0
24. Unit Forced Outage Rate	100.0	100.0	58.9
25. Forced Outage Hours	696.0	1,440.0	44,565.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* THREE MILE ISLAND 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

THREE MILE ISLAND 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	696.0	D	4		ZZ	ZZZZZZ	REGULATORY RESTRAINT ORDER CONTINUES.

SUMMARY

THREE MILE ISLAND 1 REMAINS SHUTDOWN FOLLOWING THE ACCIDENT AT UNIT 2.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sche	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

THREE MILE ISLAND 1

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....DAUPHIN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI SE OF
HARRISBURG, PA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 5, 1974

DATE ELEC ENER 1ST GENER...JUNE 19, 1974

DATE COMMERCIAL OPERATE....SEPTEMBER 2, 1974

CONDENSER COOLING METHOD... COOLING TOWERS

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GPU NUCLEAR CORP.

CORPORATE ADDRESS.....P.O. BOX 480
MIDDLETOWN, PENNSYLVANIA 17057

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....R. CONTE

LICENSING PROJ MANAGER.....J. VANVLIET
DOCKET NUMBER.....50-289

LICENSE & DATE ISSUANCE....DPR-50, APRIL 19, 1974

PUBLIC DOCUMENT ROOM.....GOVERNMENT PUBLICATIONS SECTION
STATE LIBRARY OF PENNSYLVANIA
FORUM BUILDING
COMMONWEALTH AND WALNUT STREET
HARRISBURG, PENNSYLVANIA 17105

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

OPERATING STATUS

1. Docket: 50-344
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: L. S. PETERSON (503) 556-3713 X496
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1280 X 0.95 = 1216
6. Design Electrical Rating (Net MWe): 1130
7. Maximum Dependable Capacity (Gross MWe): 1122
8. Maximum Dependable Capacity (Net MWe): 1080
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	65,712.0
13. Hours Reactor Critical	666.3	1,410.3	40,260.6
14. Rx Reserve Shtdwn Hrs	.0	.0	3,875.4
15. Hrs Generator On-Line	661.4	1,405.4	38,959.5
16. Unit Reserve Shtdwn Hrs	.0	.0	3,237.0
17. Gross Therm Ener (MWH)	2,213,221	4,750,422	123,314,275
18. Gross Elec Ener (MWH)	712,121	1,529,899	40,105,390
19. Net Elec Ener (MWH)	682,397	1,467,844	37,881,870
20. Unit Service Factor	95.0	97.6	59.3
21. Unit Avail Factor	95.0	97.6	64.2
22. Unit Cap Factor (MDC Net)	90.8	94.4	53.4
23. Unit Cap Factor (DER Net)	86.8	90.2	51.0
24. Unit Forced Outage Rate	5.0	2.4	17.6
25. Forced Outage Hours	34.6	34.6	8,336.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

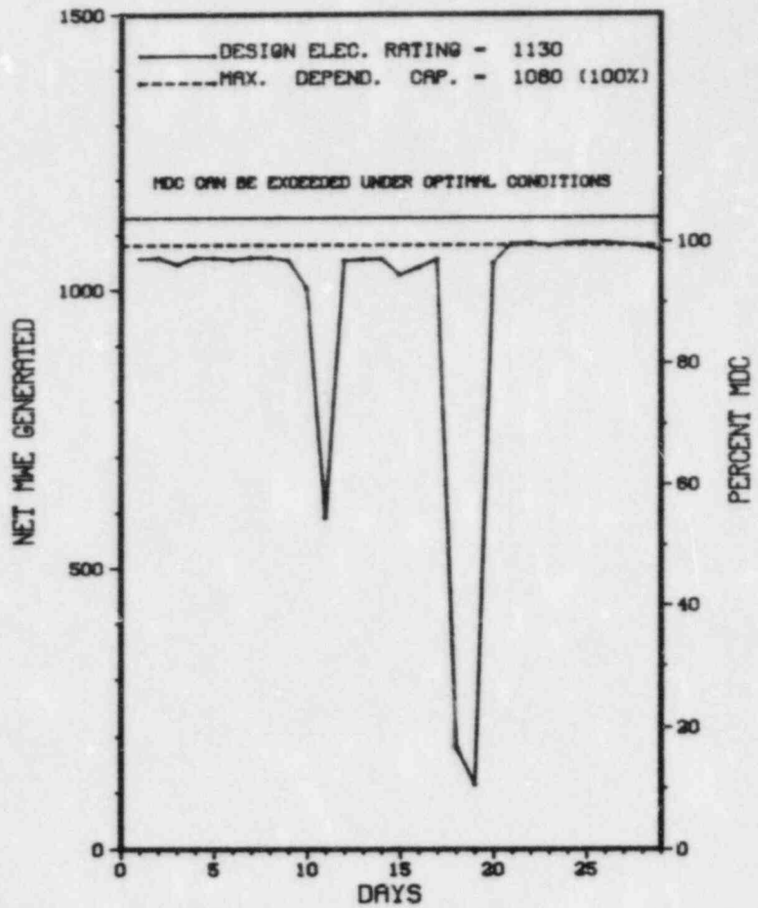
REFUELING - APRIL 27, 1984 (78 DAYS).

27. If Currently Shutdown Estimated Startup Date: N/A

****
* TROJAN *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

TROJAN



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

TROJAN

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-1	02/11/84	S	0.0	B	5				REDUCED POWER TO 50% TO INVESTIGATE FEEDWATER HEATER TUBE LEAKS.
84-2	02/18/84	F	34.6	G	3	84-03	EB	CKTBRK	AN ELECTRICAL SHORT CIRCUIT WAS CREATED BETWEEN PREFERRED INSTRUMENT BUSES Y11 AND Y22 WHILE ATTEMPTING TO REPLACE A FUSE DURING TESTING OF A CONTAINMENT ATMOSPHERE SAMPLING SYSTEM ISOLATION VALVE (SV-5643). THIS RESULTED IN A REACTOR TRIP AND INITIATION OF 'B' TRAIN SAFETY INJECTION. CORRECTIVE ACTION WAS TO REROUTE THE WIRING AND FUSING WITHIN THE PANEL TO PROVIDE MORE TRAIN SEPARATION.

SUMMARY

TROJAN OPERATED WITH 1 REDUCTION AND 1 OUTAGE DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* TROJAN *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....OREGON

COUNTY.....COLUMBIA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...42 MI N OF
PORTLAND, ORE

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 15, 1975

DATE ELEC ENER 1ST GENER...DECEMBER 23, 1975

DATE COMMERCIAL OPERATE....MAY 20, 1976

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....COLUMBIA RIVER

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PORTLAND GENERAL ELECTRIC

CORPORATE ADDRESS.....121 S.W. SALMON STREET
PORTLAND, OREGON 97204

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....G. JOHNSTON

LICENSING PROJ MANAGER.....C. TRAMMELL
DOCKET NUMBER.....50-344

LICENSE & DATE ISSUANCE....NPF-1, NOVEMBER 21, 1975

PUBLIC DOCUMENT ROOM.....MULTNOMAH COUNTY LIBRARY
SOCIAL SCIENCES & SCIENCE DEPARTMENT
801 SW 10TH AVENUE
PORTLAND, OREGON 97205

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON JANUARY 3 - FEBRUARY 10, 1984 (REPORT NO. 50-344/84-02) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON FEBRUARY 6 - MARCH 2, 1984 (REPORT NO. 50-344/84-03) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

+ THE MEETING ON JANUARY 11, 1984, REVEALED SIGNIFICANT REGULATORY CONCERNS WITH THE ADMINISTRATIVE AND TECHNICAL ADEQUACY AS WELL AS TIMELINESS IN WHICH SAFETY-RELATED INFORMATION WAS ASSESSED, COMMUNICATED AND ACTED UPON BY BOTH THE LICENSEE AND WESTINGHOUSE.

PLANT STATUS:

ROUTINE POWER OPERATION.

LAST IE SITE INSPECTION DATE: 02/06-03/02/84+

INSPECTION REPORT NO: 50-344/84-03+

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
83-11 01X-0	10-22-83	11-30-83	FAILURE OF CABLE SPREADING ROOM FIRE SUPPRESSION SYSTEM (SPECIAL REPORT)
83-22 01L-0	01-03-84	01-27-84	FAILURE OF AFW PUMP STEAM DRIVER TO START WHEN OPERATED REMOTELY
83-23 01L-0	12-29-83	- -	TRAIN B-12.47KV BUS UNDERVOLTAGE RELAY SETPOINT BELOW TECHNICAL SPECIFICATION LIMITS

OPERATING STATUS

1. Docket: 50-250
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: N. W. GRANT (305) 552-3675
4. Licensed Thermal Power (MWt): 2200
5. Nameplate Rating (Gross MWe): 894 X 0.85 = 760
6. Design Electrical Rating (Net MWe): 693
7. Maximum Dependable Capacity (Gross MWe): 700
8. Maximum Dependable Capacity (Net MWe): 666
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	98,505.6
13. Hours Reactor Critical	446.1	1,049.2	69,074.5
14. Rx Reserve Shtdwn Hrs	.0	.0	844.3
15. Hrs Generator On-Line	432.5	976.7	66,898.9
16. Unit Reserve Shtdwn Hrs	.0	.0	121.8
17. Gross Therm Ener (MWH)	943,756	1,992,922	137,481,514
18. Gross Elec Ener (MWH)	306,200	640,265	43,850,830
19. Net Elec Ener (MWH)	287,728	600,816	41,513,833
20. Unit Service Factor	62.1	67.8	67.9
21. Unit Avail Factor	62.1	67.8	68.0
22. Unit Cap Factor (MDC Net)	62.1	62.6	65.1*
23. Unit Cap Factor (DER Net)	59.7	60.2	60.8
24. Unit Forced Outage Rate	37.9	22.9	5.7
25. Forced Outage Hours	263.5	290.2	3,470.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

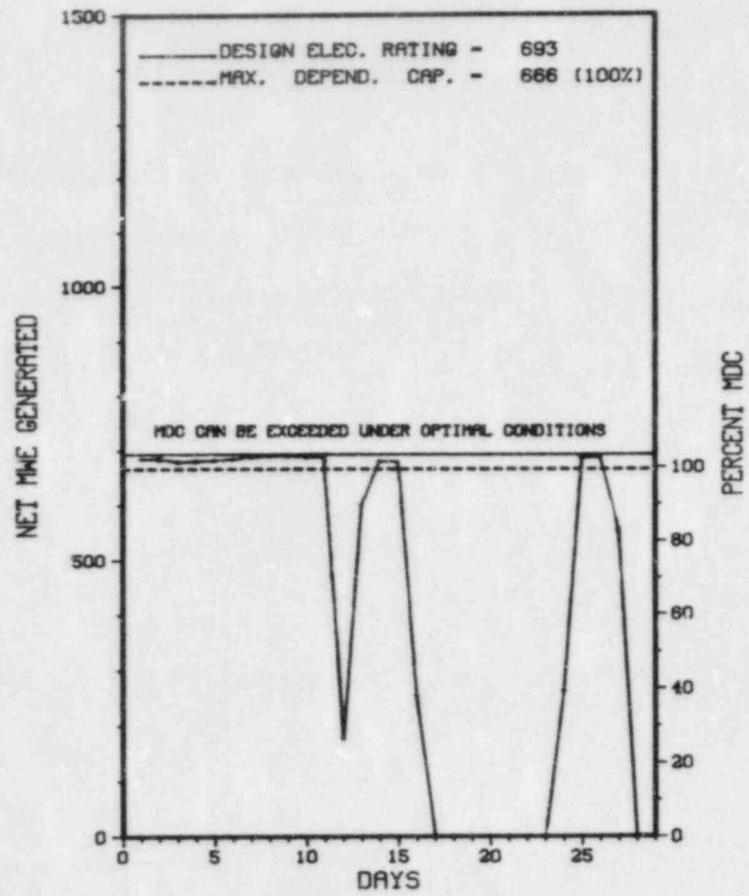
NONE

27. If Currently Shutdown Estimated Startup Date: 03/03/84

****
* TURKEY POINT 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

TURKEY POINT 3



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

TURKEY POINT 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
08	02/12/84	F	18.4	A	3	84-06	EB	RELAYX	ELECTRICAL RELAY MALFUNCTION CAUSED LOSS OF POWER SUPPLY TO MAIN FEED PUMP WHICH RESULTED IN A REACTOR TRIP. UNIT WAS RETURNED TO SERVICE.
09	02/16/84	F	195.2	A	3	84-07	EB	RELAYX	WHILE PREPARING TO MAKE REPAIRS TO A BREAKER, A RELAY WAS JARRED WHICH CAUSED THE LOSS OF THE POWER SUPPLIES TO FEED AND CONSENDATE PUMPS. THIS RESULTED IN A REACTOR TRIP. THE UNIT REMAINED OUT OF SERVICE FOR A COMPLETE INVESTIGATION.
10	02/27/84	F	49.9	A	1		EB	RELAYX	THE UNIT WAS REMOVED FROM SERVICE TO INSPECT, ADJUST AND REPAIR AUXILIARY AIR CURCUIT BREAKERS.

SUMMARY

TURKEY POINT 3 OPERATED WITH 3 OUTAGES AND NO REDUCTIONS DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* TURKEY POINT 3 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....DADE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI S OF
MIAMI, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 20, 1972

DATE ELEC ENER 1ST GENER...NOVEMBER 2, 1972

DATE COMMERCIAL OPERATE....DECEMBER 14, 1972

CONDENSER COOLING METHOD...CLOSED CANAL

CONDENSER COOLING WATER....CLOSED CYCLE CANAL

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....9250 WEST FLAGLER STREET P.O. BOX 013100
MIAMI, FLORIDA 33174

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. VOGT LOWELL

LICENSING PROJ MANAGER.....D. MCDONALD
DOCKET NUMBER.....50-250

LICENSE & DATE ISSUANCE....DPR-31, JULY 19, 1972

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MIAMI, FLORIDA 33199

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION OCTOBER 18-21 (83-37): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR HOURS ON SITE IN THE AREAS OF RADIOLOGICAL CONTROLS ASSOCIATED WITH THE UNIT 3 REFUELING OUTAGE INCLUDING EXTERNAL EXPOSURE CONTROL, INTERNAL EXPOSURE CONTROL, POSTING, LABELING AND CONTROL, SURVEYS, QUALIFICATIONS OF CONTRACT HP TECHNICIANS, ENTRY INTO THE CONTAINMENT SUMP, SPENT FUEL PIT DEMINERALIZER RESIN TRANSFER, FOLLOWUP ON UNPLANNED GASEOUS RADIOACTIVITY RELEASES AND FOLLOWUP ON PREVIOUS INSPECTOR IDENTIFIED ITEMS. OF THE 9 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 7 AREAS; ONE APPARENT VIOLATION WAS FOUND IN EACH OF 2 AREAS (FAILURE TO FOLLOW TECHNICAL SPECIFICATIONS AND FAILURE TO HAVE AN APPROVED PROCEDURE).

INSPECTION DECEMBER 3, 1983 - JANUARY 6, 1984 (83-41): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 64 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, MAINTENANCE, DOCUMENTATION, LICENSEE PROGRAM IMPROVEMENTS AND FIRE PROTECTION. OF THE SIX AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; AND TWO VIOLATIONS WERE IDENTIFIED IN TWO AREAS, FAILURE TO COMPENSATE INTERMEDIATE RANGE NUCLEAR INSTRUMENTATION ADEQUATELY (PARAGRAPH 7), AND FAILURE TO MAINTAIN ADEQUATE PLANT CHANGE DOCUMENTATION (PARAGRAPH 9). FIVE EXAMPLES OF VIOLATIONS WHICH HAD BEEN PREVIOUSLY IDENTIFIED IN REPORT 250, 251/83-38 WERE IDENTIFIED IN ONE AREA.

INSPECTION JANUARY 16-18 (84-01): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 9 INSPECTOR HOURS ON SITE IN THE AREA OF EMERGENCY PREPAREDNESS. OF THE AREAS INSPECTED, TWO VIOLATIONS WERE IDENTIFIED IN 2 AREAS (INADEQUATE PROCEDURES FOR PROTECTIVE ACTION DECISIONMAKING, FAILURE TO PROPERLY TRAIN EMERGENCY PERSONNEL IN PROTECTIVE ACTION RECOMMENDATION REQUIREMENTS AND IN EMERGENCY DOSE CALCULATIONS). THESE ARE DISCUSSED IN PARAGRAPHS 5, 5, AND 6, RESPECTIVELY.

INSPECTION SUMMARY

INSPECTION JANUARY 21-31 (84-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 43 INSPECTOR HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING AND PLANT TOURS. OF THE THREE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 30 - FEBRUARY 3 (84-03): THE INSPECTION INVOLVED 16 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD AND FOUR INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE AREAS INSPECTED WERE SECURITY ORGANIZATION - PERSONNEL; SECURITY ORGANIZATION - RESPONSE; SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS - PROTECTED AREA; PHYSICAL BARRIERS - VITAL AREAS; ASSESSMENT AIDS; ACCESS CONTROL - PERSONNEL; ACCESS CONTROL - PACKAGES; ACCESS CONTROL - VEHICLES; DETECTION AIDS - PROTECTED AREA; DETECTION AIDS - VITAL AREAS; AND ALARM STATIONS AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 14 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEMS: ACCESS CONTROL - PERSONNEL; AND TESTING AND MAINTENANCE.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.12 REQUIRES THAT A RADIATION WORK PERMIT BE ISSUED FOR ENTRIES INTO A HIGH RADIATION AREA (AN AREA WHERE THE DOSE RATE EXCEEDS 100 MILLIREMS PER HOUR), THAT WORKERS ENTERING INTO HIGH RADIATION AREAS POSSESS A RADIATION MONITORING DEVICE WHICH IS CONTINUOUSLY CAPABLE OF INDICATING THE RADIATION DOSE-RATE IN THE AREAS, AND THAT ENTRIES INTO LOCKED HIGH RADIATION AREAS (AREAS WHERE THE DOSE RATE EXCEEDS 1000 MILLIREMS PER HOUR) BE CONTROLLED BY LOCKS WHICH THEIR KEYS MAINTAINED UNDER ADMINISTRATIVE CONTROL. TECHNICAL SPECIFICATION 6.11 REQUIRES THAT PROCEDURES FOR RADIATION PROTECTION BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10CFR20 AND SHALL BE APPROVED, MAINTAINED, AND ADHERED TO FOR ALL OPERATIONS INVOLVING RADIATION EXPOSURE. PLANT PROCEDURE 11550.2(HP-2) PROHIBITS ENTRY BY PERSONNEL INTO LOCAL RADIATION CONTROL AREAS UNTIL THEY COMPLY WITH THE PRECAUTIONS AND LIMITATIONS POSTED AT THE ENTRY TO THE AREA. CONTRARY TO THE ABOVE, ON OCTOBER 14, 1983, A RADIATION PROTECTION TECHNICIAN AND A SHIFT TECHNICAL ADVISOR ENTERED AN AREA WHERE DOSE RATES WERE IN EXCESS OF 50 REMS PER HOUR AND DID NOT: (1) OBTAIN A RADIATION WORK PERMIT AS REQUIRED BY TECHNICAL SPECIFICATIONS, (2) COMPLY WITH PRECAUTIONS AND INSTRUCTIONS POSTED AT THE ENTRY TO A LOCAL RADIATION CONTROL AREA, AND (3) POSSESS A RADIATION MONITORING DEVICE WHICH CONTINUOUSLY INDICATED THE RADIATION DOSE RATE IN THE AREA. IN ADDITION TO THE ABOVE, THE LICENSEE DID NOT: (4) IMPLEMENT BY PROCEDURE THE REQUIREMENT FOR A RADIATION WORK PERMIT FOR ENTRIES INTO HIGH RADIATION AREAS BY ALL WORKERS AND (5) MAINTAIN THE KEYS TO A LOCKED HIGH RADIATION AREA UNDER ADEQUATE ADMINISTRATIVE CONTROL TO PRECLUDE UNAUTHORIZED ENTRY.
(8337 3)

TECHNICAL SPECIFICATION 6.8 REQUIRES WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX A OF REGULATORY GUIDE 1.33 AND THAT EACH PROCEDURE TO BE REVIEWED BY THE PLANT NUCLEAR SAFETY COMMITTEE AND APPROVED BY THE NUCLEAR PLANT SUPERINTENDENT PRIOR TO IMPLEMENTATION. REGULATORY GUIDE 1.33, APPENDIX A, STATES THAT PLANTS SHOULD HAVE PROCEDURES FOR THE CONTROL OF RADIOACTIVITY, INCLUDING PROCEDURES FOR SPENT RESIN AND FILTER SLUDGE HANDLING AND PROCEDURES FOR DEMINERALIZER RESIN REPLACEMENT. CONTRARY TO THE ABOVE, THE PROCEDURE USED TO TRANSFER RADIOACTIVE RESIN FROM THE SPENT FUEL POOL DEMINERALIZER TO A TEMPORARY COLLECTION FACILITY IN THE CASK WASHDOWN AREA ON OCTOBER 18, 1983 HAD NOT BEEN REVIEWED BY THE PLANT NUCLEAR SAFETY COMMITTEE, NOR APPROVED BY THE NUCLEAR PLANT SUPERINTENDENT.
(8337 4)

VIOLATION OF TECHNICAL SPECIFICATION 3.4.2.6 - TWO CONTAINMENT SPRAY PUMPS NOT OPERABLE DURING NORMAL OPERATION. FAILURE TO FOLLOW PROCEDURE TECHNICAL SPECIFICATION 6.8.1 WITH SEVEN EXAMPLES. FAILURE TO REPORT TO INSTANCES IN ACCORDANCE WITH 10CFR50.72.
(8338 4)

FAILURE TO FOLLOW PROCEDURE TECHNICAL SPECIFICATION 6.8.1 IN THAT PROCEDURAL CHANGES WERE PERFORMED WITHOUT TWO SRO QUALIFIED REVIEWS.
(8338 5)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX "A" OF USNRC REGULATORY GUIDE 1.33. SECTION 5.3.1 OF OFF NORMAL OPERATING PROCEDURE, 0208.1, REQUIRES THAT A POST TRIP REVIEW BE

ENFORCEMENT SUMMARY

PERFORMED IN ACCORDANCE WITH APPENDIX A , IN ORDER TO DETERMINE THE CAUSE OF THE TRIP AND TO INSURE ANY SAFETY CONSIDERATIONS ARE RESOLVED PRIOR TO UNIT STARTUP. CONTRARY TO THE ABOVE, UNIT 4 WAS RESTARTED ON OCTOBER 16, 1983, WITHOUT COMPLETING A POST TRIP REVIEW.
(8340 4)

FAILURE TO PROVIDE POSITIVE ACCESS CONTROL TO A VITAL AREA.
(8343 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ OPERATING.

LAST IE SITE INSPECTION DATE: JANUARY 30 - FEBRUARY 3, 1984 +

INSPECTION REPORT NO: 50-250/84-03 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-251 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: M. W. GRANT (305) 552-3675

4. Licensed Thermal Power (MWt): 2200

5. Nameplate Rating (Gross MWe): 894 X 0.85 = 760

6. Design Electrical Rating (Net MWe): 693

7. Maximum Dependable Capacity (Gross MWe): 700

8. Maximum Dependable Capacity (Net MWe): 666

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	92,233.0
13. Hours Reactor Critical	472.0	1,201.1	65,839.9
14. Rx Reserve Shtdwn Hrs	.0	.0	166.6
15. Hrs Generator On-Line	434.3	1,157.9	63,626.3
16. Unit Reserve Shtdwn Hrs	.0	.0	31.2
17. Gross Therm Ener (MWH)	931,821	2,522,918	134,278,659
18. Gross Elec Ener (MWH)	303,040	821,410	42,742,772
19. Net Elec Ener (MWH)	283,309	777,469	40,484,577
20. Unit Service Factor	62.4	80.4	69.0
21. Unit Avail Factor	62.4	80.4	69.0
22. Unit Cap Factor (MDC Net)	61.1	81.1	67.8*
23. Unit Cap Factor (DER Net)	58.7	77.9	63.3
24. Unit Forced Outage Rate	37.6	19.6	4.8
25. Forced Outage Hours	261.7	282.1	2,823.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

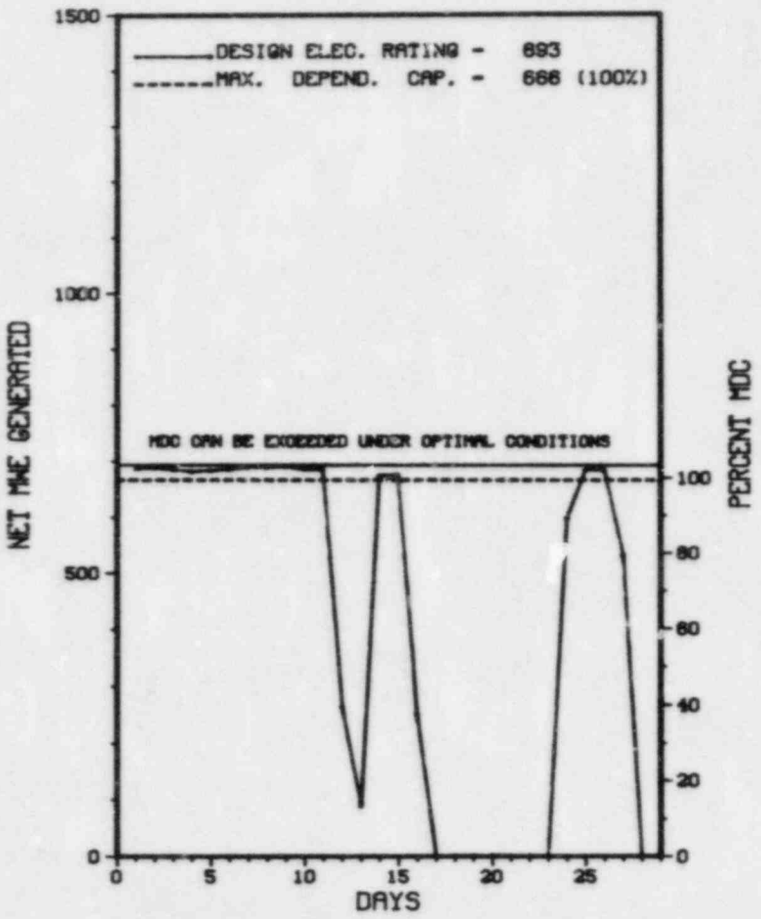
NONE

27. If Currently Shutdown Estimated Startup Date: 04/30/84

****
* TURKEY POINT 4 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

TURKEY POINT 4



FEBRUARY 1984

* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

TURKEY POINT 4

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
02	02/12/84	F	8.4	A	1	84-01	EB	RELAYX	ELECTRICAL RELAY MALFUNCTION CAUSED LOSS OF POWER SUPPLY TO MAIN FEED PUMP WHICH RESULTED IN A REACTOR TRIP. UNIT WAS RETURNED TO SERVICE.
03	02/12/84	F	19.4	A	3	84-02	EB	RELAYX	HIGH STEAM FLOW SIGNAL AND AN ACTUAL STEAM GENERATOR LOW LEVEL RESULTED IN A REACTOR TRIP. THE MALFUNCTIONING TRANSMITTER WAS REPLACED. UNIT WAS RETURNED TO SERVICE.
04	02/16/84	F	182.7	A	3	84-03	EB	RELAYX	WHILE PREPARING TO MAKE REPAIRS TO A BREAKER, A RELAY WAS JARRED WHICH CAUSED THE LOSS OF THE POWER SUPPLIES TO FEED AND CONDENSATE PUMPS. THIS RESULTED IN A REACTOR TRIP. THE UNIT REMAINED OUT OF SERVICE FOR A COMPLETE INVESTIGATION.
05	02/27/84	F	51.2	A	1		EB	RELAYX	THE UNIT WAS REMOVED FROM SERVICE TO INSPECT, ADJUST AND REPAIR AUXILIARY POWER AIR CIRCUIT BREAKERS.

SUMMARY

TURKEY POINT 4 OPERATED WITH 4 OUTAGES AND NO REDUCTIONS DURING FEBRUARY.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
		9-Other	(LER) File (NUREG-0161)

****
* TURKEY POINT 4 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....DADE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI S OF
MIAMI, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 11, 1973

DATE ELEC ENER 1ST GENER...JUNE 21, 1973

DATE COMMERCIAL OPERATE....SEPTEMBER 7, 1973

CONDENSER COOLING METHOD...CLOSED CANAL

CONDENSER COOLING WATER....CLOSED CYCLE CANAL

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....9250 WEST FLAGLER STREET P.O. BOX 013100
MIAMI, FLORIDA 33174

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. VOGT LOWELL

LICENSING PROJ MANAGER.....D. MCDONALD
DOCKET NUMBER.....50-251

LICENSE & DATE ISSUANCE....DPR-41, APRIL 10, 1973

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MIAMI, FLORIDA 33199

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION OCTOBER 18-21 (83-37): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR HOURS ON SITE IN THE AREAS OF RADIOLOGICAL CONTROLS ASSOCIATED WITH THE UNIT 3 REFUELING OUTAGE INCLUDING EXTERNAL EXPOSURE CONTROL, INTERNAL EXPOSURE CONTROL, POSTING, LABELING AND CONTROL, SURVEYS, QUALIFICATIONS OF CONTRACT HP TECHNICIANS, ENTRY INTO THE CONTAINMENT SUMP, SPENT FUEL PIT DEMINERALIZER RESIN TRANSFER, FOLLOWUP ON UNPLANNED GASEOUS RADIOACTIVITY RELEASES AND FOLLOWUP ON PREVIOUS INSPECTOR IDENTIFIED ITEMS. OF THE 9 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 7 AREAS; ONE APPARENT VIOLATION WAS FOUND IN EACH OF 2 AREAS (FAILURE TO FOLLOW TECHNICAL SPECIFICATIONS AND FAILURE TO HAVE AN APPROVED PROCEDURE).

INSPECTION DECEMBER 3, 1983 - JANUARY 6, 1984 (83-40): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 65 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, MAINTENANCE, DOCUMENTATION, LICENSEE PROGRAM IMPROVEMENTS AND FIRE PROTECTION. OF THE SIX AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; AND TWO VIOLATIONS WERE IDENTIFIED IN TWO AREAS, FAILURE TO COMPENSATE INTERMEDIATE RANGE NUCLEAR INSTRUMENTATION ADEQUATELY (PARAGRAPH 7), AND FAILURE TO MAINTAIN ADEQUATE PLANT CHANGE DOCUMENTATION (PARAGRAPH 9). FIVE EXAMPLES OF VIOLATIONS WHICH HAD BEEN PREVIOUSLY IDENTIFIED IN REPORT 250, 251/83-38 WERE IDENTIFIED IN ONE AREA.

INSPECTION JANUARY 16-18 (84-01): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 9 INSPECTOR HOURS ON SITE IN THE AREA OF EMERGENCY PREPAREDNESS. OF THE AREAS INSPECTED, TWO VIOLATIONS WERE IDENTIFIED IN 2 AREAS (INADEQUATE PROCEDURES FOR PROTECTIVE ACTION DECISIONMAKING, FAILURE TO PROPERLY TRAIN EMERGENCY PERSONNEL IN PROTECTIVE ACTION RECOMMENDATION REQUIREMENTS AND IN EMERGENCY DOSE CALCULATIONS). THESE ARE DISCUSSED IN PARAGRAPHS 5, 5, AND 6, RESPECTIVELY.

INSPECTION SUMMARY

INSPECTION JANUARY 21-31 (84-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 43 INSPECTOR HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING AND PLANT TOURS. OF THE THREE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 30 - FEBRUARY 3 (84-03): THE INSPECTION INVOLVED 16 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD AND FOUR INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE AREAS INSPECTED WERE SECURITY ORGANIZATION - PERSONNEL; SECURITY ORGANIZATION - RESPONSE; SECURITY PROGRAM AUDIT; TESTING AND MAINTENANCE; PHYSICAL BARRIERS - PROTECTED AREA; PHYSICAL BARRIERS - VITAL AREAS; ASSESSMENT AIDS; ACCESS CONTROL - PERSONNEL; ACCESS CONTROL - PACKAGES; ACCESS CONTROL - VEHICLES; DETECTION AIDS - PROTECTED AREA; DETECTION AIDS - VITAL AREAS; AND ALARM STATIONS AND COMMUNICATIONS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 14 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEMS: ACCESS CONTROL - PERSONNEL; AND TESTING AND MAINTENANCE.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.8 REQUIRES WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX A OF REGULATORY GUIDE 1.33 AND THAT EACH PROCEDURE TO BE REVIEWED BY THE PLANT NUCLEAR SAFETY COMMITTEE AND APPROVED BY THE NUCLEAR PLANT SUPERINTENDENT PRIOR TO IMPLEMENTATION. REGULATORY GUIDE 1.33, APPENDIX A, STATES THAT PLANTS SHOULD HAVE PROCEDURES FOR THE CONTROL OF RADIOACTIVITY, INCLUDING PROCEDURES FOR SPENT RESIN AND FILTER SLUDGE HANDLING AND PROCEDURES FOR DEMINERALIZER RESIN REPLACEMENT. CONTRARY TO THE ABOVE, THE PROCEDURE USED TO TRANSFER RADIOACTIVE RESIN FROM THE SPENT FUEL POOL DEMINERALIZER TO A TEMPORARY COLLECTION FACILITY IN THE CASK WASHDOWN AREA ON OCTOBER 18, 1983 HAD NOT BEEN REVIEWED BY THE PLANT NUCLEAR SAFETY COMMITTEE, NOR APPROVED BY THE NUCLEAR PLANT SUPERINTENDENT.
(8337 4)

VIOLATION OF TECHNICAL SPECIFICATION 3.4.2.6 - TWO CONTAINMENT SPRAY PUMPS NOT OPERABLE DURING NORMAL OPERATION. FAILURE TO FOLLOW PROCEDURE TECHNICAL SPECIFICATION 6.8.1 WITH SEVEN EXAMPLES. FAILURE TO REPORT TWO INSTANCES IN ACCORDANCE WITH 10CFR50.72.
(8338 4)

FAILURE TO FOLLOW PROCEDURE TECHNICAL SPECIFICATION 6.8.1 IN THAT PROCEDURAL CHANGES WERE PERFORMED WITHOUT TWO SRO QUALIFIED REVIEWS.
(8338 5)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX "A" OF USNRC REGULATORY GUIDE 1.33. SECTION 5.3.1 OF OFF NORMAL OPERATING PROCEDURE, 0208.1, REQUIRES THAT A POST TRIP REVIEW BE PERFORMED IN ACCORDANCE WITH APPENDIX A , IN ORDER TO DETERMINE THE CAUSE OF THE TRIP AND TO INSURE ANY SAFETY CONSIDERATIONS ARE RESOLVED PRIOR TO UNIT STARTUP. CONTRARY TO THE ABOVE, UNIT 4 WAS RESTARTED ON OCTOBER 16, 1983, WITHOUT COMPLETING A POST TRIP REVIEW.
(8339 4)

FAILURE TO PROVIDE POSITIVE ACCESS CONTROL TO A VITAL AREA.
(8342 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

COMPLETED STEAM GENERATOR REPLACEMENT.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JANUARY 30 - FEBRUARY 3, 1984 +

INSPECTION REPORT NO: 50-251/84-03 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-271 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: F. J. BURGER (802) 257-7711 X136

4. Licensed Thermal Power (MWt): 1593

5. Nameplate Rating (Gross MWe): 626 X 0.9 = 563

6. Design Electrical Rating (Net MWe): 514

7. Maximum Dependable Capacity (Gross MWe): 535

8. Maximum Dependable Capacity (Net MWe): 504

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	100,298.8
13. Hours Reactor Critical	696.0	1,332.6	81,031.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	696.0	1,306.7	78,799.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,103,861	2,028,043	114,188,715
18. Gross Elec Ener (MWH)	376,040	688,148	37,981,226
19. Net Elec Ener (MWH)	361,147	660,655	36,025,671
20. Unit Service Factor	100.0	90.7	78.6
21. Unit Avail Factor	100.0	90.7	78.6
22. Unit Cap Factor (MDC Net)	103.0	91.0	71.3
23. Unit Cap Factor (DER Net)	101.0	89.3	69.9
24. Unit Forced Outage Rate	.0	9.3	7.4
25. Forced Outage Hours	.0	133.3	5,024.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

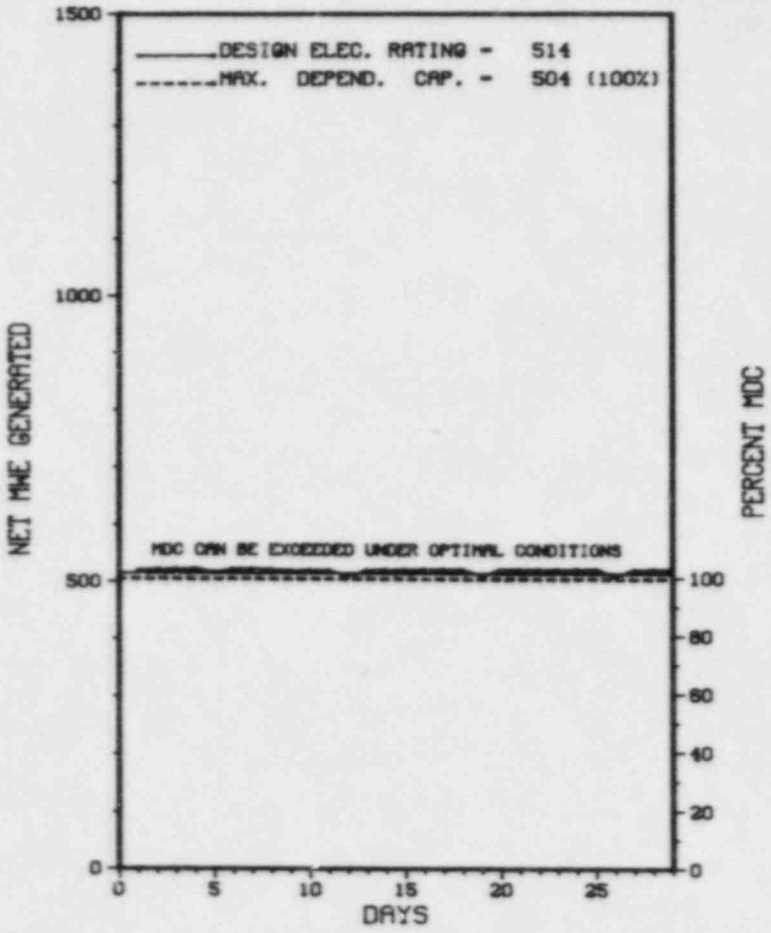
REFUELING MAINTENANCE OUTAGE - 06/16/84 - 8 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* VERMONT YANKEE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

VERMONT YANKEE 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

VERMONT YANKEE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

VERMONT YANKEE OPERATED AT FULL POWER WITH NO OUTAGES OR REDUCTIONS IN FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* VERMONT YANKEE 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....VERMONT

COUNTY.....WINDHAM

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
BRATTLEBORO, VT

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MARCH 24, 1972

DATE ELEC ENER 1ST GENER...SEPTEMBER 20, 1972

DATE COMMERCIAL OPERATE....NOVEMBER 30, 1972

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CONNECTICUT RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VERMONT YANKEE NUCLEAR POWER

CORPORATE ADDRESS.....1671 WORCESTER ROAD
FRAMINGHAM, MASSACHUSETTS 01701

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....W. RAYMOND

LICENSING PROJ MANAGER.....V. ROONEY
DOCKET NUMBER.....50-271

LICENSE & DATE ISSUANCE....DPR-28, FEBRUARY 28, 1973

PUBLIC DOCUMENT ROOM.....BROOKS MEMORIAL LIBRARY
224 MAIN STREET
BRATTLEBORO, VERMONT 05301

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-029 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: S. WHIPPLE (617) 872-8100

4. Licensed Thermal Power (MWt): 600

5. Nameplate Rating (Gross MWe): 185 X 1.0 = 185

6. Design Electrical Rating (Net MWe): 175

7. Maximum Dependable Capacity (Gross MWe): 180

8. Maximum Dependable Capacity (Net MWe): 169

9. If Changes Occur Above Since Last Report, Give Reasons:

ITEM 7 CHANGED TO REFLECT WINTER PERIOD

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	204,141.0
13. Hours Reactor Critical	679.5	1,248.8	162,773.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	674.7	1,244.0	158,156.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	393,961	731,669	85,615,263
18. Gross Elec Ener (MWH)	121,318	225,330	25,948,196
19. Net Elec Ener (MWH)	113,819	211,453	24,279,842
20. Unit Service Factor	96.9	86.4	77.5
21. Unit Avail Factor	96.9	86.4	77.5
22. Unit Cap Factor (MDC Net)	96.8	87.4	73.3*
23. Unit Cap Factor (DER Net)	93.4	83.9	69.8*
24. Unit Forced Outage Rate	3.1	13.6	5.3
25. Forced Outage Hours	21.3	196.0	7,682.4

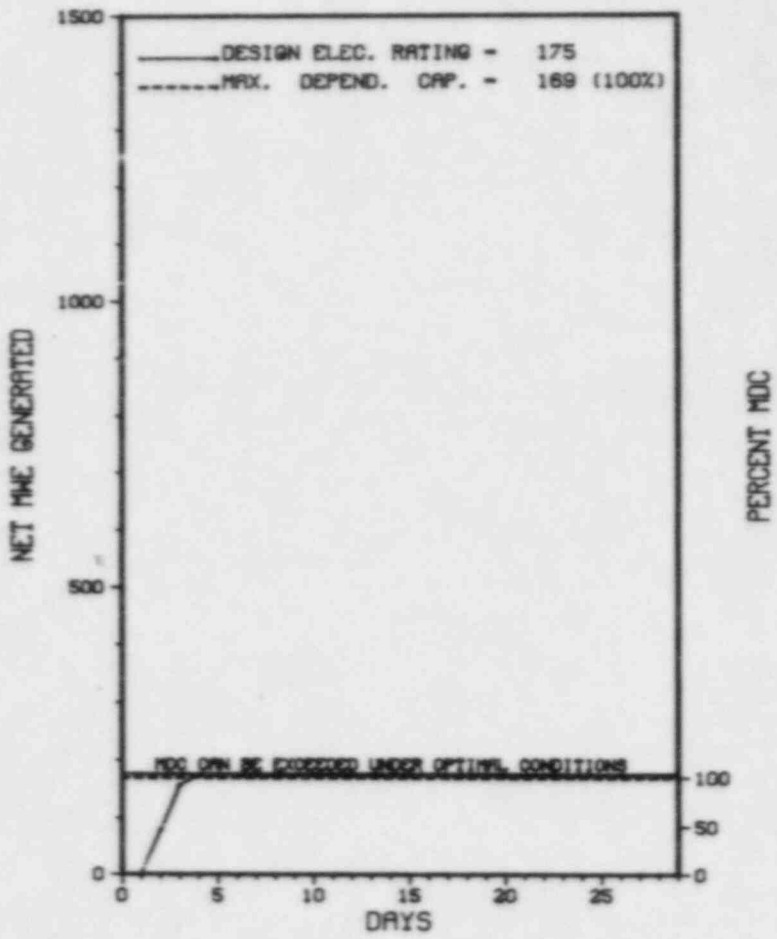
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

REFUELING - 04/01/84 - 8 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* YANKEE-ROWE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT



* Item calculated with a Weighted Average

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

YANKEE-ROWE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
84-1	01/24/84	F	21.3	A	4	84-1			PIN HOLE THRU-WALL LEAK REPAIR CONCLUDED.

SUMMARY

YANKEE-ROWE 1 RETURNED ONLINE FEBRUARY 1ST FROM A REPAIR OUTAGE AND OPERATED ROUTINELY THE REMAINDER OF FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* YANKEE-ROWE 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....MASSACHUSETTS

COUNTY.....FRANKLIN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI NE OF
PITTSFIELD, MASS

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 19, 1960

DATE ELEC ENER 1ST GENER...NOVEMBER 18, 1960

DATE COMMERCIAL OPERATE....JULY 1, 1961

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DEERFIELD RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....YANKEE ATOMIC ELECTRIC

CORPORATE ADDRESS.....1671 WORCESTER RD.
FRAMINGHAM, MASSACHUSETTS 01701

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....H. EICHENHOLZ

LICENSING PROJ MANAGER.....P. ERICKSON
DOCKET NUMBER.....50-029

LICENSE & DATE ISSUANCE....DPR-3, DECEMBER 24, 1963

PUBLIC DOCUMENT ROOM.....GREENFIELD COMMUNITY COLLEGE
1 COLLEGE DRIVE
GREENFIELD, MASSACHUSETTS 01301

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* YANKEE-ROWE 1 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-295 OPERATING STATUS

2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0

3. Utility Contact: GERRI AUSTIN (312) 746-2084

4. Licensed Thermal Power (MWt): 3250

5. Nameplate Rating (Gross MWe): 1220 X 0.9 = 1098

6. Design Electrical Rating (Net MWe): 1040

7. Maximum Dependable Capacity (Gross MWe): 1085

8. Maximum Dependable Capacity (Net MWe): 1040

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	89,112.0
13. Hours Reactor Critical	528.7	528.7	62,596.8
14. Rx Reserve Shtdwn Hrs	.0	.0	2,621.8
15. Hrs Generator On-Line	430.2	430.2	60,895.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,088,987	1,088,987	171,018,478
18. Gross Elec Ener (MWH)	350,548	350,548	55,070,427
19. Net Elec Ener (MWH)	330,243	330,243	52,233,548
20. Unit Service Factor	61.8	29.9	68.3
21. Unit Avail Factor	61.8	29.9	68.3
22. Unit Cap Factor (MDC Net)	45.6	22.1	56.4
23. Unit Cap Factor (DER Net)	45.6	22.1	56.4
24. Unit Forced Outage Rate	38.2	55.5	13.8
25. Forced Outage Hours	265.8	537.3	9,149.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

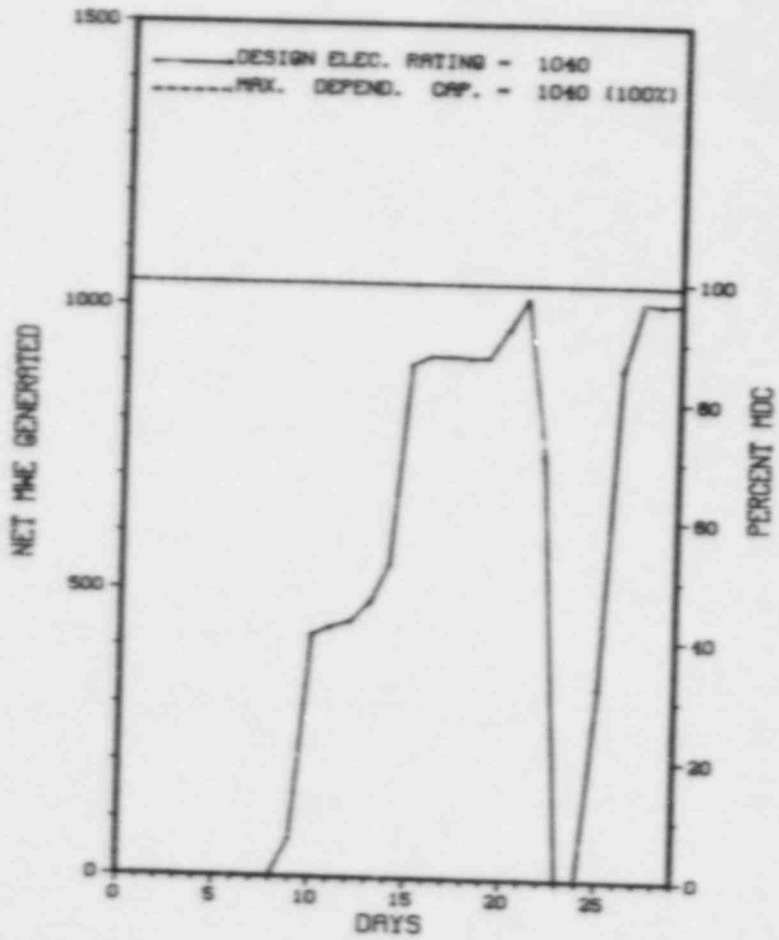
330243

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ZION 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ZION 1



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* ZION 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	01/28/84	F	208.1	A	4				CONTINUED OUTAGE FOR REACTOR COOLANT LEAK AT THE SEAL TABLE.
2	02/22/84	F	19.5	A	3				MAIN TURBINE GOVERNOR VALVE WENT SHUT AND CAUSED STEAM GENERATOR LOW-LEVEL REACTOR TRIP WITH STEAM FLOW MISMATCH.
3	02/23/84	F	8.4	A	2				MANUAL TRIP DUE TO ROD URGENT ALARM, RODS WOULD NOT MOVE WHILE ATTEMPTING TO GO CRITICAL.
4	02/23/84	F	7.5	A	3				MALFUNCTION OF FEED REGULATOR VALVE CAUSED REACTOR TRIP HIGH LEVEL ON B STEAM GENERATOR.
5	02/24/84	F	22.1	A	3				1C FEEDWATER PUMP FAILED TO MINIMUM SPEED CAUSING A REACTOR TRIP ON 1D STEAM GENERATOR WITH LOW LEVEL AND STEAM FLOW FEED FLOW MISMATCH.

****
* SUMMARY *
****

ZION 1 RETURNED ONLINE FEBRUARY 9TH FROM A REPAIR OUTAGE AND OPERATED ROUTINELY THE REMAINDER OF FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report (LER) File (NUREG-0161)
			9-Other	

****
* ZION 1 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....LAKE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI N OF
CHICAGO, ILL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 19, 1973

DATE ELEC ENER 1ST GENER...JUNE 28, 1973

DATE COMMERCIAL OPERATE....DECEMBER 31, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. WATERS

LICENSING PROJ MANAGER.....J. NORRIS
DOCKET NUMBER.....50-295

LICENSE & DATE ISSUANCE....DPR-39, OCTOBER 19, 1973

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ZION, ILLINOIS 60099

INSPECTION STATUS

INSPECTION SUMMARY

SPECIAL INSPECTION ON FEBRUARY 3 - 14,(84-01): INSPECTION FOLLOWUP ON COMMONWEALTH EDISON COMPANY REMARKS TO SPECIAL INSPECTION REPORT CONCERNING ALLEGATIONS OF IMPROPER OPERATION AT DRESDEN, QUAD CITIES, AND ZION NUCLEAR POWER PLANTS. THE INSPECTION INVOLVED 14 INSPECTION-HOURS BY THREE NRC INSPECTORS. MEASURES TO CORRECT IDENTIFIED WEAKNESSES WERE TAKEN AS DESCRIBED IN THE REPONSE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT RESUMED OPERATION ON 2/06/84, AND IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: JANUARY 18-31, 1984

INSPECTION REPORT NO: 84-01

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-01	01/06/84	02/02/84	REACTOR TRIP.
84-02	01/07/84	02/02/84	REACTOR TRIP.
84-03	01/16/84	02/14/84	REACTOR TRIP.
84-04	01/20/84	02/17/84	PLANT CONDITIONS NOT BOUNDED BY SAFETY ANALYSES.
84-05	01/20/84	02/17/84	INCORE INSTRUMENTATION. SEAL TABLE HI PRESSURE SEAL FAILURE.

1. Docket: 50-304 OPERATING STATUS
2. Reporting Period: 02/01/84 Outage + On-line Hrs: 696.0
3. Utility Contact: GERRI AUSTIN (312) 746-2084
4. Licensed Thermal Power (MWt): 3250
5. Nameplate Rating (Gross MWe): 1220 X 0.9 = 1098
6. Design Electrical Rating (Net MWe): 1040
7. Maximum Dependable Capacity (Gross MWe): 1085
8. Maximum Dependable Capacity (Net MWe): 1040
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

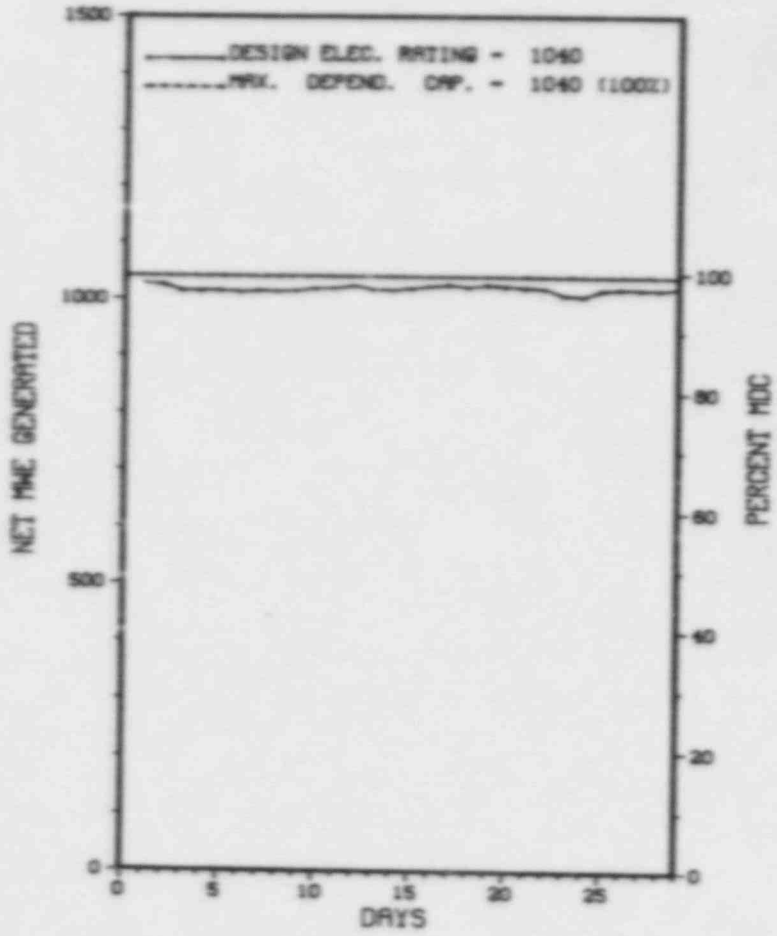
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	696.0	1,440.0	82,825.0
13. Hours Reactor Critical	696.0	1,485.2	68,638.2
14. Rx Reserve Shtdwn Hrs	.0	.0	226.1
15. Hrs Generator On-Line	696.0	1,391.8	58,918.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,245,646	4,424,879	169,348,162
18. Gross Elec Ener (MWH)	737,598	1,448,912	54,152,949
19. Net Elec Ener (MWH)	708,524	1,390,713	51,467,658
20. Unit Service Factor	100.0	96.7	71.1
21. Unit Avail Factor	100.0	96.7	71.1
22. Unit Cap Factor (MDC Net)	97.9	92.9	59.8
23. Unit Cap Factor (DER Net)	97.9	92.9	59.8
24. Unit Forced Outage Rate	.0	3.3	17.3
25. Forced Outage Hours	.0	48.2	12,424.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
MAINTENANCE - 03/27/84
27. If Currently Shutdown Estimated Startup Date: N/A

****
* ZION 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ZION 2



FEBRUARY 1984

Report Period FEB 1984

UNIT SHUTDOWNS / REDUCTIONS

****
* ZION 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

****
* SUMMARY *
****

ZION 2 OPERATED AT NEAR FULL POWER WITH NO OUTAGES OR REDUCTIONS DURING FEBRUARY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ZION 2 *
****

FACILITY DATA

Report Period FEB 1984

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....LAKE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI N OF
CHICAGO, ILL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 24, 1973

DATE ELEC ENER 1ST GENER...DECEMBER 26, 1973

DATE COMMERCIAL OPERATE....SEPTEMBER 17, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....NONE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. WATERS

LICENSING PROJ MANAGER.....J. NORRIS
DOCKET NUMBER.....50-304

LICENSE & DATE ISSUANCE....DPR-48, NOVEMBER 14, 1973

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ZION, ILLINOIS 60099

INSPECTION STATUS

INSPECTION SUMMARY

SPECIAL INSPECTION ON FEBRUARY 3 - 14,(84-01): INSPECTION FOLLOWUP ON COMMONWEALTH EDISON COMPANY REMARKS TO SPECIAL INSPECTION REPORT CONCERNING ALLEGATIONS OF IMPROPER OPERATION AT DRESDEN, QUAD CITIES, AND ZION NUCLEAR POWER PLANTS THE INSPECTION INVOLVED 14 INSPECTION-HOURS BY THREE NRC INSPECTORS. MEASURES TO CORRECT IDENTIFIED WEAKNESSES WERE TAKEN AS DESCRIBED IN THE REPONSE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT IS OPERATING ROUTINELY.

LAST IE SITE INSPECTION DATE: JANUARY 18-31, 1984

INSPECTION REPORT NO: 84-01

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

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SECTION 3

APPENDIX

* PRESSURIZED WATER REACTORS *

STATUS OF SPENT FUEL STORAGE CAPABILITY

FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
ARKANSAS 1	177	968	316	652		N/S	1998
ARKANSAS 2	177	988	168	820		N/S	2003
BEAVER VALLEY 1	157	833	52	781		N/S	1995
CALVERT CLIFFS 1	217	1830(c)	796(c)	1034(c)(m)	1170	03-85	1991
CALVERT CLIFFS 2	217					04-84	1991
COOK 1	193	2050(c)	494(c)	1556(c)		N/S	1994
COOK 2	193					03-84	
CRYSTAL RIVER 3	177	1163	171	992		N/S	1997
DAVIS-BESSE 1	177	735	140	595		N/S	1993
DIABLO CANYON 1							
FARLEY 1	157	675	62	613	1345	02-84	1991
FARLEY 2	157	675	62	613	1345	N/S	1994
FORT CALHOUN 1	133	483	265	218	463	03-84	1985
GINNA	121	595	300	295		03-84	1992
HADDAM NECK	157	1168	493	675		06-84	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	268	214	980	05-84	1984
INDIAN POINT 3	193	837	140	697		N/S	1993
KEWAUNEE	121	998	228	762(m)		03-84	1991
MAINE YANKEE	217	953	577	376	1678	N/S	1987
MCGUIRE 1	193	500	31	469(m)		02-84	1990
MCGUIRE 2							
MILLSTONE 2	217	667	376	291		N/S	1987
NORTH ANNA 1	157	966(c)	116(c)	850		05-84	1991
NORTH ANNA 2	157					03-84	1990
OCONEE 1	177	1312(1)	1123	189(1)(n)		N/S	1991
OCONEE 2	177					N/S	
OCONEE 3	177	825	0	825		03-84	
PALISADES	204	784	480	304		N/S	1988
POINT BEACH 1	121	1058(c)	484(c)	1078(c)		N/S	1995
POINT BEACH 2	121					N/S	
PRAIRIE ISLAND 1	121	1017(c)	561(c)	456(c)(m)	720	N/S	1988
PRAIRIE ISLAND 2	121					08-84	
RANCHO SECO 1	177	579	280	299		10-84	1987
ROBINSON 2	157	276	152	124(e)	431	N/S	1985(g)
SALEM 1	193	1170	212	958		05-84	1996
SALEM 2	193	1170	72	1098		N/S	2000
SAN ONOFRE 1	157	216	94	122		N/S	1985
SAN ONOFRE 2	217	800	0	800		N/S	
SAN ONOFRE 3	217	800	0	800		N/S	
SEQUOYAH 1	193	800	0	800		02-84	1993
SEQUOYAH 2(d)	193	800	65	735		N/S	1994
ST LUCIE 1	217	728	352	376		N/S	1990
ST LUCIE 2							
SUMMER 1	157	682	0	682	1276	N/S	
SURRY 1	157	1044(c)	556(c)	484(c)		N/S	1987
SURRY 2	157					N/S	
THREE MILE ISLAND 1	177	752	208	544		N/S	1986
THREE MILE ISLAND 2	177	442	0	442		N/S	1986

Report Period FEB 1984

STATUS OF SPENT FUEL STORAGE CAPABILITY

PRESSURIZED WATER REACTORS FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
TROJAN	193	651	248	403		04-84	1990
TURKEY POINT 3	157	621	445	175(m)		N/S	1987
TURKEY POINT 4	157	621	378	243		03-84	1988
YANKEE-ROWE 1	76	391	225	166	496	04-84	1988
ZION 1	193	2112(c)	795(c)	1317(c)		N/S	1992
ZION 2	193					04-84	1992

BOILING WATER REACTORS — STATUS OF SPENT FUEL STORAGE CAPABILITY

FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
BIG ROCK POINT 1	84	195	152	41	289	08-84	1986
BROWNS FERRY 1	764	3471	1068	2403		07-84	1985
BROWNS FERRY 2	764	3471	889	861(m)	2582	08-84	1985
BROWNS FERRY 3	764	3471	1520	398(m)	2650	N/S	1985
BRUNSWICK 1	560	(f)	160PWR+656BWR	2116		N/S	1986
BRUNSWICK 2	560		144PWR+564BWR	2288		N/S	1986
COOPER STATION	548	2366	848	1518		N/S	1996
DRESDEN 1	484	672	221	451		N/S	1990
DRESDEN 2	724	2659(c)	2894 (c)	996(c)	6129(c)	N/S	1985
DRESDEN 3	724					N/S	
DUANE ARNOLD	368	2050	576	1474		N/S	1998
FITZPATRICK	560	2244	816	1428		N/S	1991
HATCH 1	560	3021	0	3021		N/S	1999
HATCH 2	560	2750	1284	1466		N/S	1999
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	440	207	233		N/S	1990
LASALLE 1							
MILLSTONE 1	580	2184	1136	1048		04-84	1991
MONTICELLO	484	2237	1016	1221		02-84	1991
NINE MILE POINT 1	532	1984	1044	940	1965	03-84	1990
OYSTER CREEK 1	560	1800	1375	425	1225	N/S	1987
PEACH BOTTOM 2	764	2816	1170	1646		02-84	1990
PEACH BOTTOM 3	764	2816	1212	1604		N/S	1991
PILGRIM 1	580	2320	1708	612(m)		N/S	1990
QUAD CITIES 1	724	3657	1730	1927		N/S	2003
QUAD CITIES 2	724	3897	412	3485		N/S	2003

INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS(h)

MORRIS OPERATIONS	750 MTU(j)	315	385 MTU(j)	1490 MTU(j)	
NFS(i)	250 MTU	170 MTU	80 MTU		

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2772 BWR and 1232 PWRassemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium: 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved.
(l) This is the station total.
(m) Installed capacity is less than that authorized.
(n) McGuire 1 authorized to accept Oconee fuel assemblies.

N/S = Not Scheduled

STATUS OF SPENT FUEL STORAGE CAPABILITY

BOILING WATER REACTORS

FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
SUSQUEHANNA 1	764	2840	0	2840		N/S	1997
VERMONT YANKEE 1	368	2088	1082	918		06-84	1992

INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS(h)

MORRIS OPERATIONS	750 MTU(j)	315	385 MTU(j)	1490 MTU(j)	
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N/S = Not Scheduled

(INCLUDES BOTH LICENSED AND NON-LICENSED UNITS)

REACTOR YEARS OF EXPERIENCE

	YEARS	1ST ELEC GENERATE	UNIT
**********	-----	--------	----
* LICENSED *	9.58	08/21/74	ARKANSAS 1
* OPERATING *	21.23	12/08/62	BIG ROCK POINT 1
* ELECTRICAL *	7.47	09/12/76	BROWNS FERRY 3
* PRODUCING *	9.16	01/03/75	CALVERT CLIFFS 1
* UNITS *	5.94	03/22/78	COOK 2
**********	6.51	08/28/77	DAVIS-BESSE 1
	9.79	05/19/74	DUANE ARNOLD
	9.88	02/01/75	FITZPATRICK
	24.25	12/02/69	GINNA
	5.44	09/22/78	HATCH 2
	9.90	04/08/74	KEWAUNEE
	11.31	11/08/72	MAINE YANKEE
	13.25	11/29/70	MILLSTONE 1
	14.31	11/05/69	NINE MILE POINT 1
	10.82	05/06/73	OCONEE 1
	14.44	09/23/69	OYSTER CREEK 1
	9.50	09/01/74	PEACH BOTTOM 3
	11.58	08/02/72	POINT BEACH 2
	11.89	04/12/72	QUAD CITIES 1
	13.43	09/26/70	ROBINSON 2
	16.63	07/16/67	SAN ONOFRE 1
	3.61	07/22/80	SEQUOYAH 1
	.72	06/13/83	ST LUCIE 2
	10.98	03/10/73	SURRY 2
	8.19	12/23/75	TROJAN
	11.44	09/28/72	VERMONT YANKEE 1
	10.18	12/26/73	ZION 2

YEARS	1ST ELEC GENERATE	UNIT
-----	--------	----
5.18	12/26/78	ARKANSAS 2
10.38	10/15/73	BROWNS FERRY 1
7.24	12/04/76	BRUNSWICK 1
7.23	12/07/76	CALVERT CLIFFS 2
9.81	05/10/74	COOPER STATION
13.88	04/15/70	DRESDEN 2
6.54	08/10/77	FARLEY 1
10.52	08/25/73	FORT CALHOUN 1
16.57	08/07/67	HADDAM NECK
10.68	06/26/73	INDIAN POINT 2
15.85	04/26/68	LA CROSSE
2.67	06/30/81	MCGUIRE 1
8.31	11/09/75	MILLSTONE 2
5.87	04/17/78	NORTH ANNA 1
10.24	12/05/73	OCONEE 2
12.17	12/31/71	PALISADES
11.62	07/19/72	PILGRIM 1
10.24	12/04/73	PRAIRIE ISLAND 1
11.77	05/23/72	QUAD CITIES 2
7.18	12/25/76	SALEM 1
1.45	09/20/82	SAN ONOFRE 2
2.19	12/23/81	SEQUOYAH 2
1.29	11/16/82	SUMMER 1
1.29	11/16/82	SUSQUEHANNA 1
11.33	11/02/72	TURKEY POINT 3
23.38	11/10/60	YANKEE-ROWE 1

YEARS	1ST ELEC GENERATE	UNIT
-----	--------	----
7.71	06/14/76	BEAVER VALLEY 1
9.51	08/28/74	BROWNS FERRY 2
8.84	04/29/75	BRUNSWICK 2
9.05	02/10/75	COOK 1
7.08	01/30/77	CRYSTAL RIVER 3
12.61	07/22/71	DRESDEN 3
2.77	05/25/81	FARLEY 2
7.22	12/11/76	FORT ST VRAIN
9.30	11/11/74	HATCH 1
7.84	04/27/76	INDIAN POINT 3
1.49	09/04/82	LASALLE 1
.77	05/23/83	MCGUIRE 2
12.99	03/05/71	MONTICELLO
3.52	08/25/80	NORTH ANNA 2
9.50	09/01/74	OCONEE 3
10.83	02/18/74	PEACH BOTTOM 2
13.32	11/06/70	POINT BEACH 1
9.19	12/21/74	PRAIRIE ISLAND 2
9.38	10/13/74	RANCHO SECO 1
2.74	06/03/81	SALEM 2
.43	09/25/83	SAN ONOFRE 3
7.82	05/07/76	ST LUCIE 1
11.66	07/04/72	SURRY 1
9.70	06/19/74	THREE MILE ISLAND 1
10.69	06/21/73	TURKEY POINT 4
10.67	06/28/73	ZION 1

TOTAL 721.22 YRS

	YEARS	1ST ELEC GENERATE	SHUTDOWN DATE	UNIT
**********	-----	--------	--------	----
* PERMANENTLY *	3.80	08/14/64	06/01/68	BONUS
* OR *	18.54	04/15/60	10/31/78	DRESDEN 1
* INDEFINITELY*	6.32	08/05/66	11/29/72	FERMI 1
* SHUTDOWN *	13.21	04/18/63	07/02/76	HUMBOLDT BAY
* UNITS *	1.19	07/25/66	10/01/67	PATHFINDER
**********	2.16	11/04/63	01/01/66	PIQUA

TOTAL 74.77 YRS

YEARS	1ST ELEC GENERATE	SHUTDOWN DATE	UNIT
-----	--------	--------	----
3.04	12/18/63	01/01/67	CVTR
4.44	08/24/63	02/01/68	ELK RIVER
1.26	05/29/63	09/01/64	HALLAM
12.12	09/16/62	10/31/74	INDIAN POINT 1
7.76	01/27/67	11/01/74	PEACH BOTTOM 1
.93	04/21/78	03/28/79	THREE MILE ISLAND 2

The total reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year. If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

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* RESEARCH *
* REACTORS *
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NON-POWER REACTORS IN THE U.S.

STATE	CITY	LICENSEE	REACTOR TYPE	DOCKET	LICENSE NUMBER	DATE OL ISSUED	AUTHORIZED POWER LEVEL (KW)
ALABAMA	TUSKEGEE	TUSKEGEE INSTITUTE	AGN-201 #102	50-406	R-122	08-30-74	0.0001
ARIZONA	TUCSON	UNIVERSITY OF ARIZONA	TRIGA MARK I	50-113	R-52	12-05-58	100.0
CALIFORNIA	BERKELEY	UNIVERSITY OF CALIFORNIA, BERKELEY COLLEGE	TRIGA MK. III	50-224	R-101	08-10-66	1000.0
	CANOGA PARK	ROCKWELL INTERNATIONAL CORP.	L-85	50-375	R-188	01-05-72	0.003
	HAWTHORNE	NORTHROP CORP. LABORATORIES	TRIGA MARK F	50-187	R-90	03-04-63	1000.0
	IRVINE	UNIVERSITY OF CALIFORNIA, IRVINE	TRIGA MARK I	50-326	R-116	11-24-69	250.0
	LOS ANGELES	UNIVERSITY OF CALIFORNIA, L.A.	ARGONAUT	50-142	R-71	10-03-60	100.0
	SAN DIEGO	GENERAL ATOMIC COMPANY	TRIGA MARK F	50-163	R-67	07-01-60	1500.0
	SAN DIEGO	GENERAL ATOMIC COMPANY	TRIGA MARK I	50-089	R-38	05-03-58	250.0
	SAN JOSE	GENERAL ELECTRIC COMPANY	NTR	50-073	R-33	10-31-57	100.0
	SAN LUIS OBISPO	CALIFORNIA STATE POLYTECHNIC COLLEGE	AGN-201 #100	50-394	R-121	05-16-73	0.0001
	SAN RAMON	AEROTEST OPERATIONS, INC.	TRIGA (INDUS)	50-228	R-98	07-02-65	250.0
	SANTA BARBARA	UNIVERSITY OF CALIFORNIA, SANTA BARBARA	L-77	50-433	R-124	12-03-74	0.01
COLORADO	DENVER	U.S. GEOLOGICAL SURVEY DEPARTMENT	TRIGA MARK I	50-274	R-113	02-24-69	1000.0
DELAWARE	NEWARK	UNIVERSITY OF DELAWARE	AGN-201 #113	50-098	R-43	07-03-58	0.0001
DIST OF COLUMBIA	WASHINGTON	THE CATHOLIC UNIVERSITY OF AMERICA	AGN-201 #101	50-077	R-31	11-15-67	0.0001
FLORIDA	GAINESVILLE	UNIVERSITY OF FLORIDA	ARGONAUT	50-083	R-56	05-21-59	100.0
GEORGIA	ATLANTA	GEORGIA INSTITUTE OF TECHNOLOGY	AGN-201 #104	50-276	R-111	04-19-68	0.0001
	ATLANTA	GEORGIA INSTITUTE OF TECHNOLOGY	HEAVY WATER	50-160	R-97	12-29-64	5000.0
IDAHO	POCATELLO	IDAHO STATE UNIVERSITY	AGN-201 #103	50-284	R-110	10-11-67	0.0001
ILLINOIS	URBANA	UNIVERSITY OF ILLINOIS	LOPRA	50-356	R-117	12-27-71	10.0
	URBANA	UNIVERSITY OF ILLINOIS	TRIGA	50-151	R-115	07-22-69	1500.0
	ZION	WESTINGHOUSE ELECTRIC CORP.	NTR	50-087	R-119	01-28-72	10.0
INDIANA	LAFAYETTE	PURDUE UNIVERSITY	LOCKHEED	50-182	R-87	08-16-62	10.0
IOWA	AMES	IOWA STATE UNIVERSITY	UTR-10	50-116	R-59	10-16-59	10.0
KANSAS	LAWRENCE	UNIVERSITY OF KANSAS	LOCKHEED	50-148	R-78	06-23-61	250.0
	MANHATTAN	KANSAS STATE UNIVERSITY	TRIGA	50-188	R-88	10-16-62	250.0
MARYLAND	BETHESDA	ARMED FORCES RADIOBIOLOGY RESEARCH INSTITUTE	TRIGA	50-170	R-84	06-26-62	1000.0
	COLLEGE PARK	UNIVERSITY OF MARYLAND	TRIGA	50-166	R-70	10-14-60	250.0

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* RESEARCH *
* REACTORS *
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NON-POWER REACTORS IN THE U.S.

STATE	CITY	LICENSEE	REACTOR TYPE	DOCKET	LICENSE NUMBER	DATE OL ISSUED	AUTHORIZED POWER LEVEL (KW)
MASSACHUSETTS	CAMBRIDGE	MASSACHUSETTS INSTITUTE OF TECHNOLOGY	HWR REFLECTED	50-020	R-37	06-09-58	5000.0
	LOWELL	UNIVERSITY OF LOWELL	GE	50-223	R-125	12-24-74	1000.0
	WORCESTER	WORCESTER POLYTECHNIC INSTITUTE	GE	50-134	R-61	12-16-59	10.0
MICHIGAN	ANN ARBOR	UNIVERSITY OF MICHIGAN	POOL	50-002	R-28	09-13-57	2000.0
	EAST LANSING	MICHIGAN STATE UNIVERSITY	TRIGA MARK I	50-294	R-114	03-21-69	250.0
	MIDLAND	DOW CHEMICAL COMPANY	TRIGA	50-264	R-108	07-03-67	100.0
MISSOURI	COLUMBIA	UNIVERSITY OF MISSOURI, COLUMBIA	TANK	50-186	R-103	10-11-66	10000.0
	ROLLA	UNIVERSITY OF MISSOURI	POOL	50-123	R-79	11-21-61	200.0
NEBRASKA	OMAHA	THE VETERANS ADMINISTRATION HOSPITAL	TRIGA	50-131	R-57	06-26-59	18.0
NEW MEXICO	ALBUQUERQUE	UNIVERSITY OF NEW MEXICO	AGN-201M #112	50-252	R-102	09-17-66	0.005
NEW YORK	BRONX	MANHATTAN COLLEGE - PYHSICS DEPT.	TANK	50-199	R-94	03-24-64	0.0001
	BUFFALO	STATE UNIVERSITY OF NEW YORK	PULSTAR	50-057	R-77	03-24-61	2000.0
	ITHACA	CORNELL UNIVERSITY	TRIGA MARK II	50-157	R-80	01-11-62	500.0
	ITHACA	CORNELL UNIVERSITY	ZPR	50-097	R-89	12-11-62	0.1
	NEW YORK	COLUMBIA UNIVERSITY IN THE CITY OF NEW YORK	TRIGA MARK II	50-208	R-128	04-14-77	250.0
	TUXEDO	UNION CARBIDE CORP	POOL	50-054	R-81	09-07-61	5000.0
NORTH CAROLINA	RALEIGH	NORTH CAROLINA STATE UNIVERSITY AT RALEIGH	PULSTAR	50-297	R-120	08-25-72	1000.0
OHIO	COLUMBUS	OHIO STATE UNIVERSITY	POOL	50-150	R-75	02-24-61	10.0
OKLAHOMA	NORMAN	THE UNIVERSITY OF OKLAHOMA	AGN-211 #102	50-112	R-53	12-29-58	0.100
OREGON	CORVALLIS	OREGON STATE UNIVERSITY	TRIGA MARK II	50-243	R-106	03-07-67	1000.0
	PORTLAND	REED COLLEGE	TRIGA MARK I	50-288	R-112	07-02-68	250.0
PENNSYLVANIA	UNIVERSITY PARK	PENNSYLVANIA STATE UNIVERSITY	TRIGA MK. III	50-005	R-2	07-08-55	1000.0
RHODE ISLAND	NARRAGANSETT	RHODE ISLAND NUCLEAR SCIENCE CENTER	GE POOL	50-193	R-95	07-21-64	2000.0
TENNESSEE	MEMPHIS	MEMPHIS STATE UNIVERSITY	AGN-201 #108	50-538	R-127	12-10-76	0.0001
TEXAS	AUSTIN	UNIVERSITY OF TEXAS	TRIGA MARK I	50-192	R-92	08-02-63	250.0
	COLLEGE STATION	TEXAS A&M UNIVERSITY	AGN-201M #106	50-059	R-23	08-26-57	0.005
	COLLEGE STATION	TEXAS A&M UNIVERSITY	TRIGA	50-128	R-83	12-07-61	1000.0
UTAH	PROVO	BRIGHAM YOUNG UNIVERSITY	L-77	50-262	R-109	09-07-67	0.01

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UTAH	SALT LAKE CITY	THE UNIVERSITY OF UTAH	TRIGA MARK I	50-407	R-126	09-30-75	100.0
	SALT LAKE CITY	UNIVERSITY OF UTAH	AGN-201M #107	50-072	R-25	09-12-57	0.005
VIRGINIA	BLACKSBURG	VIRGINIA POLYTECHNIC INSTITUTE	UTR-10	50-124	R-62	12-18-59	100.0
	CHARLOTTESVILLE	UNIVERSITY OF VIRGINIA	CAVALIER	50-396	R-123	09-24-74	0.1
	CHARLOTTESVILLE	UNIVERSITY OF VIRGINIA	POOL	50-062	R-66	06-27-60	2000.0
	LYNCHBURG	BABCOCK & WILCOX COMPANY	LPR	50-099	R-47	09-05-58	1000.0
WASHINGTON	PULLMAN	WASHINGTON STATE UNIVERSITY	TRIGA	50-027	R-76	03-06-61	1000.0
	SEATTLE	UNIVERSITY OF WASHINGTON	ARGONAUT	50-139	R-73	03-31-61	100.0
WISCONSIN	MADISON	UNIVERSITY OF WISCONSIN	TRIGA	50-156	R-74	11-23-60	1000.0
**********EXPERIMENTAL AND TEST REACTORS**********							
CALIFORNIA	SAN JOSE	GENERAL ELECTRIC COMPANY	GETR	50-070	TR-1	01-07-59	50,000.0
DIST OF COLUMBIA	WASHINGTON	NATIONAL BUREAU OF STANDARDS	TEST	50-184	TR-5	06-30-70	10,000.0
**********CRITICAL EXPERIMENT FACILITIES**********							
NEW YORK	TROY	RENSSELAER POLYTECHNIC INSTITUTE		50-225	CX-22	07-03-64	0.0
VIRGINIA	LYNCHBURG	BABCOCK & WILCOX COMPANY		50-013	CX-10	10-22-58	0.0
WASHINGTON	RICHLAND	BATTELLE MEMORIAL INSTITUTE		50-360	CX-26	11-29-71	0.0

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14 ABSTRACT (200 words or less)

The OPERATING UNITS STATUS REPORT - LICENSED OPERATING REACTORS provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Resource Management from the Headquarters staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. The three sections of the report are: monthly highlights and statistics for commercial operating units, and errata from previously reported data; a compilation of detailed information on each unit, provided by NRC's Regional Offices, IE Headquarters and the utilities; and an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the U.S. It is hoped the report is helpful to all agencies and individuals interested in maintaining an awareness of the U.S. energy situation as a whole.

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