

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 5 9										PAGE (3) 1 OF 0 2					
TITLE (4) High Drywell Leakage and Subsequent Manual Scram																									
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)										
6	4	0	8	4	8	4	0	2	6	0	0	0	7	1	7	8	4	0 5 0 0 0 0							
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																							
POWER LEVEL (10) 0 5 9		20.402(b)						20.405(c)						60.73(a)(2)(iv)						73.71(b)					
		20.405(a)(1)(i)						60.36(a)(1)						60.73(a)(2)(v)						73.71(e)					
		20.405(a)(1)(ii)						60.36(a)(2)						60.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 306A)					
		20.405(a)(1)(iii)						60.73(a)(2)(i)						60.73(a)(2)(vii)(A)											
		20.405(a)(1)(iv)						60.73(a)(2)(ii)						60.73(a)(2)(vii)(B)											
		20.405(a)(1)(v)						60.73(a)(2)(iii)						60.73(a)(2)(v)											
LICENSEE CONTACT FOR THIS LER (12)																									
NAME William A. Roberts, Jr.												TELEPHONE NUMBER AREA CODE 2 0 5 7 2 9 - 0 7 8 8													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS															
X	A	D	S	E	A	L	B	5	8	0	Y														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)						MONTH DAY YEAR							
X YES (If yes, complete EXPECTED SUBMISSION DATE: ) NO																		0 8 2 0 8 4							

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Unidentified drywell leakage exceeded Technical Specification 3.6.C.1.a., 5-gallons per minute reactor coolant leakage rate. An orderly shutdown was initiated as required by Technical Specification 3.6.C.3 and the unit was manually scrambled at 59 percent power.

The drywell was entered and "B" recirculation pump upper seal was discovered to have failed. The seal was replaced with a spare seal assembly and the unit returned to service.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Browns Ferry - Unit 1	05000259	84	026	00	02	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On June 20, 1984, unit 1 was at 59 percent power; unit 2 was operating normally at 61 percent power; and unit 3 was in a refueling outage. Only unit 1 was affected by this event.

On June 19, 1984, at 2227, the unit 1 drywell floor drain sump pump (P) rate for 64 minutes equaled 5.125-gallons per minute. This unidentified leakage exceeded the Technical Specification 5-gallons per minute rate requiring a reactor shutdown within 24-hours. An orderly load reduction was initiated and on June 20, 1984, (at 0050, with the unit at 59 percent power), the unit was manually scrambled.

A drywell entry was subsequently made and the reactor coolant (AD) leakage source was identified to be from a failed upper seal (SEAL) on the "B" recirculation pump. The seal was replaced with a spare seal assembly and the unit returned to service.

The recirculation pump shaft seal is a compound seal. The breakdown bushing will only allow 20-gallons per minute leakage to the drywell sump in the event both seals fail (complete seal failure). This would be negligible for orderly shutdown.

The failed seal assembly is being disassembled to determine cause of failure. A followup report will be submitted when failure cause is determined.

Responsible Section - N/A

Previous Similar Events - None

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

Browns Ferry Nuclear Plant  
P. O. Box 2000  
Decatur, Alabama 35602

July 17, 1984

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

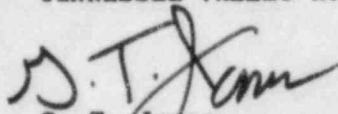
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE  
REPORT BFRO-50-259/84026

The enclosed report provides details concerning high drywell leakage and  
subsequent manual scram. This report is submitted in accordance with  
10 CFR 50.73 (a)(2)(i) and (a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



G. T. Jones  
Power Plant Superintendent  
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, GA 30303

INPO Records Center  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, GA 30339

NRC Resident Inspector, BFN

LEW  
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