

JUL 19 1984

DMB o/c

Docket No. 50-346

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Mr. Richard P. Crouse
Vice President, Nuclear
Toledo Edison Company
Edison Plaza - Stop 712
300 Madison Avenue
Toledo, Ohio 43652

Dear Mr. Crouse:

SUBJECT: POST ACCIDENT SAMPLING SYSTEM, NUREG-0737, ITEM II.B.3
REQUEST FOR ADDITIONAL INFORMATION

Toledo Edison Company responded by letter dated April 25, 1983 (No. 931) to our request to document how the criteria of NUREG-0737, Item II.B.3 have been satisfied. On May 5, 1984, May 18, 1984 and June 26, 1984, we held telephone discussions with your staff to obtain clarification with respect to certain of your responses. These discussions revealed the need for additional documentation. The information we require is shown in the enclosure to this letter.

Please provide the information requested no later than July 30, 1984. The information requested in this letter affects fewer than ten respondents; therefore, OMB clearance is not required under P.L. 95-511.

Sincerely,

"ORIGINAL SIGNED BY:"

George W. Rivenbark, Acting Chief
Operating Reactors Branch #4
Division of Licensing

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page

ORB#4:DL *ma*
Ade Agazio;cf
7/19/84

ORB#4:DL *GR*
GRivenbark
7/19/84

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PDR

Toledo Edison Company

cc w/enclosure(s):

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Columbus, Ohio 43216

REQUEST FOR ADDITIONAL INFORMATION

NUREG-0737, ITEM II.B.3

1. Identify the method used to determine dissolved hydrogen in the reactor coolant sample. State the accuracy and sensitivity of the method or instrument used and whether the sample is pressurized or unpressurized and if sampling is continuous and diluted. Also identify how long the instrument takes to obtain a steady reading.
2. Verify that samples for offsite laboratory analysis will be transported using licensed carriers. Identify the offsite laboratory or laboratories expected to be used.
3. Describe the sampling point for the containment air sample and verify that a sample taken at that location will be representative of the containment atmosphere.
4. Are the valves used in the PASS environmentally qualified for the post-accident sampling conditions, e.g., temperature, pressure, activity, etc., in which they must operate?
5. Describe the portable oxygen monitor used to determine dissolved oxygen in the reactor coolant sample. State the accuracy and sensitivity of the monitor.
6. State the method used for chloride analysis at the offsite laboratory and the associated accuracy and sensitivity.
7. Provide the radiation exposure results from your time and motion studies of post-accident sampling and analysis. Confirm that the results include the effects of all radiation sources to which an operator would be exposed, that the exposures would be as-low-as-reasonably-achievable, and that the requirements of GDC 19 are met.
8. Describe the offsite laboratory method used to determine dissolved hydrogen in the reactor coolant. Provide details of the type sample and indicate if it is transported pressurized.
9. Confirm that there is a backup counting facility for post-accident samples at Davis-Besse site and indicate its location.
10. Describe provisions for backup power to the PASS in event of loss-of-offsite power.
11. Identify the method for boron analysis at the offsite laboratory and state the applicable accuracy and sensitivity.

12. Confirm that the post-accident sample analytical methods and instrumentation have been tested using the standard test matrix for the types of contamination anticipated in the post-accident samples. Confirm that the analytical methods and instrumentation will yield reliable results in the presence of anticipated radioactivity.
13. Provide your procedure for estimating the extent of core damage.