Commonwealth Edison
Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

GCT-92-010

March 2, 1992

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of February 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

G. C. Tietz
Technical Superintendent

GCT/CALS/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

FEBRUARY 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Cynthia Losek-Short and Debra Kelley, telephone number 309-654-2241, extensions 2938 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Unit One began the month of February operating at full load. The unit operated close to or at full power for the entire month except during February 7 thru February 20th. The reactor scrammed on February 7th at 0201 hours due to high steam flow indication. The unit remained off-line for repairs until the reactor went critical on February 18 at 1045 hours. All other load drops that occurred through the month were issued by Chicago Load Dispatch.

B. Unit Two

Unit Two continued scheduled refuel outage.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Admendment No. 134 was issued on February 14, 1992 to Facility Operating License No. DPR-29 and Amendment No. 126 to Facility Operating License No. DPR-30 for Quad Cities Nuclear Power Station, Units 1 and 2 respectively. These amendments revise the Technical Specifications to reflect the addition of two new sections: tne Radiation Protection Program and High Radiaiton Area.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q85102	6600	Repair shaft driven oil pump leaks.	As Found: Gaskets were found to be a little aged. As Left: Cleaned sealing surfaces and installed new gaskets.
Q73408	6600	Investigate and repair problems with 1/2 D.G. Relay found during QMMS 6600-1 and S1 general surveillance.	As Left: Removed old relay on 1/2 D.G. and installed new relay.
Q97905	6700	Replace the chipped insulation plate on phase "C" of NLN #10 with new one.	As Found: Insulation plate on breaker had a broken corner. As Left: Removed stationary arching contact and then removed the broken insulating plate. Installed new insulating plate, assembled and installed stationary arching contact.
Q97910	6700	Replace the SBM switch on the 4KV breaker number 69.	As Found: The SBM switch was not working properly due to high contact resistance. As Left: Removed old SBM switch and replaced with new SBM switch. Tested new SBM with digital fluke.
Q97271	7300	Remove LITI RMS-9 programmer from 480V breaker and install a LSTI-RMS-9 programmer.	Programmer was wrong type for the application and was removed. The LITI RMS-9 was rejected to vendor and a new LSTI-RMS-9 was installed in breaker number N029.
Q95965	0202	Repair reactor recirc pump 1A suction valve indication on 18-19-5 which burns dim.	As Found: Found neutral wire from indicating light at MCC was pulled loose. As Left: Proper light indication was achieved after log was landed.
Q91046	0261	Investigate and repair alarm that came up at 138 indication/other at 142 indication/both on D/P indication switch main steam line flow not repeating very well.	As Found: Mercoid switch didn't trip all the way when actuated. As Left: Installed new switch, calibrated it, adjusted trip point to 142.6 psi and rising. Performed functional test as per QIS 21-2 and switch worked fine.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q97935	0263	Calibrate and test reactor vessel level control indication switches 1-263-59A and B during level recovery following Reactor Scram.	As Found: Performed loop calibration check on 1-263-59A instrument loop switch 3 on 1-263-59A was barely in tolerance and switch 4 was out of tolerance. Calibration check on 1-263-59B discovered that the indicator for 1-263-59B was over range on both ends and switch 3 was out of tolerance. As Left: Performed calibration for 1-263-59A and 1-263-59B loops. After all calibrations were completed performed QIS 5610-26-6 and verified instruments worked fine.
Q98221	0901	Replace reset switch 1B RBFDS which is bad.	As Found: Old switch appears to be overheated. As Left: Removed old switch and replaced with new switch. Verified proper operation of switch.
Q93255	1100	Investigate and repair standby liquid control tank temperature switch.	As Found: Temperature switch TS-1-1149 was out of its hole. As Left: Calibrated and restored temperature switch TS-1-1149 and 1-1155 to their proper position. No adjustment necessary.
Q93255	1100	Investigate and repair standby liquid control tank temperature switch.	As Found: Temperature switch TS-1-1149 was out of its hole. As Left: Calibrated and restored temperature switch TS-1-1149 and 1-1155 to their proper position. No adjustment necessary.
Q93255	1100	Investigate and repair standby liquid control tank temperature switch.	As Found: Temperature switch TS-1-1149 was out of its hole. As Left: Calibrated and restored temperature switch TS-1-1149 and 1-1155 to their proper position. No adjustment necessary.
Q96937	1401	Investigate and repair 1B core spray pump seal leak off line which leaks at the weld to the discharge piping.	As Found: 1/2" nipple was leaking. As Left: Removed old nipple and installed new 5-14"/(1/2") carbon steel nipple.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q97908	2300	Repair 1-2317 HPCI stop valve which has become uncoupled on the actuator side and is not closing.	As Left: Disassembled the valve and its internals. Blue check on main poppet seat failed, all other necessary parts were good except for the main valve seat which had cracking indications. The crack indications on the main poppet were approved exceptable per a G.E. Letter. Assembled valve internals after the main poppet seat was lapped and blue checked okay. Checked stem alignment which looks good with straight edge. Checked and verified full valve stroke which was 4 1/16" and coupled valve to operator.
Q97732	6700	Replace the defective secondary disconnect and charging motor on 4KV breaker.	As Found: 1) Secondary disconnect was damaged - faulty crimp, resulting in loose termination on MG5 Pin. 2) Broken brush holder cap on charging motor. As Left: 1) Removed damaged secondary disconnect after measuring location. Performed wire lift and landed (recorded and verified) crimp leads and installed new disconnect. 2) Performed wire lift for charging motor, removed damaged motor, installed new motor, landed leads (recorded and verified). Performed passive inspection and electrical test on breaker.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q97252	0902	Repair cracked insulation and bad lug on wire contact 14 for 902-32 panel.	As Found: Wiring was showing through insulation due to bad crimp. As Left: We replaced lug with SI #376840, after verifying OOS and installing jumper.
Q98139	6600	Investigate the cause for Unit 2 Diesel Generator field failure to flash.	Tested and recorded tachmeter voltages versus rpms on Diesel Generator, which improved after adjusting toothed wheel gap in linkage. Replaced contacts CA1 and CA2 and cleaned up wiring on speeds sensing panel. Started diesel for test run, ran speed up to 800 rpm and adjusted RH2 to pickup vs relay. The apparent cause of failure due to setpoint drift on speed sensing panel components.
Q83701	6700	Investigate and repair motor breaker Bus 23-1 Cub. 2 auxiliary switch linkage is not making proper contact with auxiliary switch while performing a core spray logic test.	Tested 2A Core Spray pump breaker on Bus 23-1 Cub. 2 and it performed all its functions. Apparent cause of breaker malfunction was due to dirty contacts and linkage not being adjusted.
Q97807	7800	Repair wire on 1/2A Standby Gas Treatment MCC 29-4 Cub. D2 causing short.	As Found: Black wire from cable #2099 was found behind stabs of Cub. D-3 causing a short. As Left: Cut and relugged black wire and relanded it on terminal #7. Also dressed cables.
Q97804	7800	Repair damage to MCC 29-4 cubicle D-3 breaker for 2-1001-26B which has blown internal parts.	As Found: Afternoon shift found a conductor under T1, T2 and T3 terminal block. When 2-1001-26b cubicle was stroked the lead shorted T1 and T3 together causing both overload relays, overload heaters and open contractor to be damaged. As Left: Replaced open contact on left side, both overload relays, both overload heaters, terminal blocks T1, T2, and T3, and the lug for motor field cable on terminal T1. Meggered terminal blocks T1, T2, T3, 1, 2, 3, 3A, 4 and 7 to ground and all read above 500 mohms or greater.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

Licensee Event Report Number	Date	Title of Occurrence
92-002	02-06-92	HPCI Turbine Stop Valve failed open.
92-003	02-06-92	HPCI Suction Pipe Hanger outside design basis.
92-004	02-27-92	Unit 1 Rx Scram from MSL High Flow.
92-005	02-01-92	RCIC 1301-49 Valve could not open.
92-006	02-25-92	Loss of Main Control Room Annunciators for Unit One following loss of line 0405.

UNIT 2

92-006	01-29-92	Missed On-Site Review of a Temporary Procedure # 7370.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE March 3, 1992
COMPLETED BY Cynthia Losek-Short
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 020192 / 2400 022992 GROSS HOURS IN REPORTING PERIOD: 696

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	423.3	1187.3	137678.4
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3421.9
7. HOURS GENERATOR ON LINE	404.0	1148.0	133379.1
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)	963084.0	2808993.8	286059023.6
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	314110.0	917585.0	92991921.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	306700.0	901576.0	87610944.0
12. REACTOR SERVICE FACTOR	60.82	81.06	78.93
13. REACTOR AVAILABILITY FACTOR	60.82	81.06	80.95
14. UNIT SERVICE FACTOR	58.05	79.72	76.52
15. UNIT AVAILABILITY FACTOR	58.05	79.72	77.04
16. UNIT CAPACITY FACTOR (Using MDC)	57.3	81.42	65.38
17. UNIT CAPACITY FACTOR (Using Design MWe)	55.65	79.35	63.70
18. UNIT FORCED OUTAGE RATE	41.9	20.28	5.87

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE March 3, 1992
COMPLETED BY Cynthia Lopek-Short
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 020192 2400 022992 GROSS HOURS IN REPORTING PERIOD: 696

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.25	133482.9
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	2385.8
7. HOURS GENERATOR ON LINE	0.0	0.25	130020.15
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	702.9
9. GROSS THERMAL ENERGY GENERATED (MWH)	0.0	175.2	280[illegible]4062.0
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.0	50.0	89930240.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	-4875.0	-9837.0	85724343.0
12. REACTOR SERVICE FACTOR	0.0	0.02	77.27
13. REACTOR AVAILABILITY FACTOR	0.0	0.02	79.0
14. UNIT SERVICE FACTOR	0.0	0.02	75.27
15. UNIT AVAILABILITY FACTOR	0.0	0.02	75.87
16. UNIT CAPACITY FACTOR (Using MDC)	-0.87	-0.89	64.89
17. UNIT CAPACITY FACTOR (Using Design MWe)	-0.85	-0.87	62.45
18. UNIT FORCED OUTAGE RATE	0.0	0.0	8.09

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: April 10, 1992
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE March 3, 1992
COMPLETED BY Cynthia Losek-Short
TELEPHONE (309) 654-2241

MONTH February 1992

DAY	AVEREAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	804	17.	-6
2.	803	18.	-6
3.	805	19.	129
4.	809	20.	704
5.	809	21.	809
6.	811	22.	772
7.	53	23.	763
8.	-6	24.	775
9.	-5	25.	807
10.	-6	26.	803
11.	-6	27.	790
12.	-6	28.	791
13.	-6	29.	790
14.	-6	30.	
15.	-6	31.	
16.	-6		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE March 3, 1992
COMPLETED BY Cynthia Losek-Short
TELEPHONE (309) 654-2241

MONTH February 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-7	17.	-6
2.	-7	18.	-6
3.	-7	19.	-7
4.	-7	20.	-7
5.	-7	21.	-7
6.	-7	22.	-7
7.	-6	23.	-7
8.	-6	24.	-7
9.	-5	25.	-7
10.	-6	26.	-7
11.	-6	27.	-8
12.	-6	28.	-8
13.	-6	29.	-8
14.	-6	30.	
15.	-6	31.	
16.	-6		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME Unit One

COMPLETED BY Cynthia Losek-Short

DATE March 3, 1992

REPORT MONTH February, 1992

TELEPHONE 309-654-2241

NO.	DATE	T.PE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-02	2-7-92	F	272.7	A	3	- - -	- -	- - - - -	Scram Due to High Steam Flow Indication. Corrective Maintenance on ERV.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Unit Two

DATE March 3, 1992

REPORT MONTH February, 1992

COMPLETED BY Cynthia Losek-Short

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
	2-1-92	S	696	C	4	- - - -	- -	- - - -	Continuation of Unit 2 Refuel Outage.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: One

Date: February 7, 1992

Valves Actuated	No. & Type of Actuation
1-203-3B	2 Per QCOP 203-1

Plant Conditions: Reactor Pressure - 1041 psig & 1018 psig

Description of Events: Reactor pressure control during scram event of February 7, 1992.

Unit: One

Date: February 19, 1992

Valves Actuated	No. & Type of Actuation
1-203-3A	Post Maintenance
1-203-3B	Post Maintenance
1-203-3C	Post Maintenance
1-203-3D	Post Maintenance
1-203-3E	Post Maintenance

Plant Conditions: Reactor Pressure - 927 psig

Description of Events: Post Maintenance surveillance following Unit 1 scram and failure of 3C ERV to actuate when manually operated.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
PERFORMED ON UNIT 1 & 2 CONTROL
ROD DRIVES, FROM 1-1-92 TO 12-31-92

		AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
DATE	NUMBER OF RODS	5 0.375	20 0.900	50 2.00	90 3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
2-19-92	2	0.28	0.67	1.43	2.48	J-2 2.55	Drive Replacement (J-2), Scram Valve N-7
2-20-92	1	0.32	0.69	1.45	2.45	N-5 2.45	Scram Valve Work N-5

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-532
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 11 Cycle: 12

2. Scheduled date for next refueling shutdown: 9-5-92

3. Scheduled date for restart following refueling: 12-5-92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   NOT AS YET DETERMINED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1405

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

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OCT 30 1989
O.C.O.S.R.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: '2 Cycle: 11

2. Scheduled date for next refueling shutdown: 01/01/92

3. Scheduled date for restart following refueling: 04/10/92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment: Yes, as listed below:
   1. Remove Table 3.7-2
   2. Modification to turbine control valve fast acting solenoid valve.
   3. Modification to HPCI turbine exhaust steam line.
   4. HPCI/RCIC 24-hour shutdown action provision.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:
   1. 01/15/92
   2. 04/18/91
   3. 06/28/91
   4. 12/31/91

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 0

   b. Number of assemblies in spent fuel pool: 3163

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

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OCT 30 1989
O.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - Inservice Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRS
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Fraction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center