



February 18, 1983  
Attachment

NORTHERN STATES POWER COMPANY  
PRAIRIE ISLAND NUCLEAR GENERATING PLANT

LER 83-001/017-0

### Event Description

On February 4, 1983, an error in the codes used in the ECCS analysis of Prairie Island units was discovered. The peak clad temperature (PCT) of 13°F greater than 2190°F, which means less than 2200°F limit.

With the 13°F error in PCT included, the portion of fuel exposures sufficient PCT was present F limit of 2.21. The exception of standard fuel at greater than 41,940 TOPROD fuel at beginning of life. The F<sub>0</sub> limit will secure that the 2200°F will not core. (Calculations indicate a 1% reduction concludes that operation of the current cycle with the within acceptable ECCS evaluation criteria.

A review of power distribution measurements for all cycles containing Huxor fuel, has been conducted if and when a 1% penalized F<sub>0</sub> limit was not exceeded throughout cycle 5. Unit 1 the 1% penalized limit was not exceeded. However, on the current cycle 5 this map number 108-10 by 0.3%. Measurement taken on Unit 2 and cycle 8. Map 108-10 the occurred in a fresh Huxor the time of the discovery.

### Corrective Action

1. The error in the codes used in the ECCS analysis of Prairie Island units was discovered. The peak clad temperature (PCT) of 13°F greater than 2190°F, which means less than 2200°F limit.
2. The error in the codes used in the ECCS analysis of Prairie Island units was discovered. The peak clad temperature (PCT) of 13°F greater than 2190°F, which means less than 2200°F limit.

**NSP**

February 18, 1983

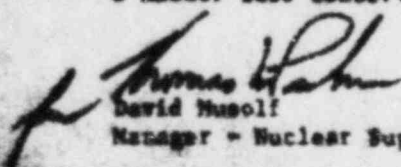
Regional Administrator  
Region III  
U S Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket No. 50-282 License No. NP-42

Error Found in Exxon BCCS Analysis

The License Event Report for this occurrence is attached.

This event is reported in compliance with Technical Specification 6.7.B.1.h since errors were discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

  
David Musolf  
Manager - Nuclear Support Services

DMM/TDP/jc

cc: NRC Document Control Desk (1)  
NRC Resident Inspector  
NPCA  
Attn: J V Forman

Enclosure