

Additional Event Description and Probable Consequences

Spent fuel assembly X-13 was being moved to another location in the Unit 4 spent fuel pool for the installation of a burnable poison insert. Lifting the spent fuel assembly out of its designated rack in the pool, the hoisting cable on the fuel handling crane came back into its rack from the fully lifted position, in accordance with T.S. 6.9.2.a.2. There is no indication of cladding damage, as radiation monitor readings and survey results are pre-event values. The public's health and safety were not affected. LERs were 251-73-2 and 251-75-7.

Additional Cause Description and Corrective Action

A malfunction of the two limit switches on the hoisting crane failed to stop the upward movement of the fuel assembly. This caused the cable to be overstressed, subsequently, the hoisting cable broke.

The weekly periodic test of the Fuel Pit Bridge Crane - Periodic Test, had been performed and was used to demonstrate proper function of the equipment prior to the event. At this time, it was verified that the hoisting cable was controlled by the limit switches. Apparently, the limit switch linkage broke at the time this check was performed and the hoisting cable was moved. The linkage in the power circuit limit switch was found unhooked. The geared limit switch was found out of adjustment. The necessary repairs and adjustments were made on the limit switch. The hoisting cable was replaced. Plant procedures 16300, Spent Fuel Pit Bridge Crane Operating Instructions, and 16304.1, Spent Fuel Pit Bridge Crane - Periodic Test, have been changed to provide clarification of instructions concerning the function and operation of the two limit switches.

The dropped fuel assembly will remain in its present position until an inspection is made to verify that no cladding damage is present. Two spent fuel assemblies from the Unit 3 spent fuel pool have been designated to be used as replacements for X-13 and its sister assembly in the Unit 4 core. (At the time of this event, Unit 4 was defueled while awaiting generator replacement. Fuel assembly X-13 was to be used to start the generator load.) When the ultimate disposition of X-13 has been determined, an update will be issued. An investigation is still continuing. A detailed information on the circumstances surrounding this event will be included in the LER update.

20. In § 150.20, the introductory text of paragraph (b) is revised to read as follows:

§ 150.20 Recognition of agreement state licenses.

(b) Notwithstanding any provision to the contrary in any specific license issued by an Agreement State to a person engaging in activities in a non-Agreement State or in offshore waters under the general licenses provided in this section, the general licenses provided in this section are subject to the provisions of §§ 30.7(a) through (e), 30.14(d) and §§ 30.34, 30.41, and 30.51 to 30.63, inclusive, of Part 30 of this chapter; paragraphs 40.7(a) through (e) and §§ 40.41, 40.51, 40.61 to 40.63, inclusive, 40.71 and 40.81 of Part 40 of this chapter; and paragraphs 70.7(a) through (e) and §§ 70.32, 70.42, 70.51 to 70.56, inclusive, 70.61, 70.62, and 70.71 of Part 70 of this chapter; and to the provisions of Parts 19, 20, and 71 and Subpart B of Part 34 of this chapter. In addition, any person engaging in activities in non-Agreement States or in offshore waters under the general licenses provided in this section:

Dated at Washington, D.C. this 9th day of July, 1982.

For the Nuclear Regulatory Commission,
Samuel J. Chalk,
Secretary of the Commission.

[FR Doc. 82-18024 Filed 7-13-82; 8:45 am]
BILLING CODE 7590-01-M

10 CFR Part 50

Codes and Standards for Nuclear Power Plants

AGENCY: Nuclear Regulatory Commission.

ACTION: Final rule.

SUMMARY: The Commission is amending its regulations to incorporate by reference the Summer 1981 Addenda of the ASME Boiler and Pressure Vessel Code. The sections of the ASME Code being incorporated provide rules for the construction of nuclear power plant components. Adoption of these amendments will permit the use of improved methods for construction.

EFFECTIVE DATE: August 13, 1982.

FOR FURTHER INFORMATION CONTACT: Mr. E. Baker, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Telephone (301) 443-6892.

SUPPLEMENTARY INFORMATION: On February 3, 1982 the Nuclear Regulatory Commission published in the Federal

Register [47 FR 5010] proposed amendments to its regulation, 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities." The proposed amendments revised § 50.55a to incorporate by reference the Summer 1981 Addenda to Section III of the ASME Boiler and Pressure Vessel Code.

Some of the changes effected in the addenda which are incorporated through the adoption of the amendments are:

1. Article NCA-3000 of Section III was revised to add a requirement that N, NA, and NPT certificate holders be responsible for documentation of the review and approval of materials used by them and the preparation, accumulation, control, and protection of required records while in their custody. Also, the owner must review the materials documentation to verify that the Code Edition, Addenda, and Code Cases used satisfy NCA-1140 and are acceptable to the regulatory and enforcement authorities.

2. Article NCA-8000 of Section III was revised editorially to make it easier to read and understand. Also, two new provisions, NCA-8240(b) and NCA-8430, were added. NCA-8240(b) describes the provisions that must be met if a nameplate is to be removed from an item which has been installed in a nuclear power plant system. NCA-8430 describes alternatives for compiling the Code Data Reports so that they can be traced from the Data Report Form.

3. Article NB-3500 of Section III was revised to remove the nomenclatures "normal duty valve," "severe duty valve," "standard valve," and "expected cycle," but there were no technical changes associated with dropping these nomenclatures.

4. Article NB-6000 was given an extensive editorial rewrite which mainly reorganized the paragraphs into a more comprehensive form. Also added were a subarticle on special test procedures and a subparagraph allowing the hydrostatic testing of pump and valve subassemblies.

Interested persons were invited to submit written comments for consideration in connection with the proposed amendment by May 5, 1982. One information/editorial comment on the supplementary information section of the proposed rule was received but no significant comments were received. The necessary editorial corrections were made. The Commission has adopted the proposed amendment with a minor editorial revision to accommodate the incorporation by reference of the ASME Code.

Paperwork Reduction Act Statement

The recordkeeping requirements contained in this Regulation have been approved by the Office of Management and Budget, OMB approval No. 3150-0011.

Regulatory Flexibility Certification

In accordance with the Regulatory Flexibility Act of 1980, 5 U.S.C. 605(b), the Commission hereby certifies that this rule will not have a significant economic impact on a substantial number of small entities. This rule affects only the licensing and operation of nuclear power plants. The companies that own these plants do not fall within the scope of the definition of "small entities" set forth in the Regulatory Flexibility Act or the Small Business Size Standards set out in regulations issued by the Small Business Administration at 13 CFR Part 121. Since these companies are dominant in their service areas, this rule does not fall within the purview of the Act.

List of Subjects in 10 CFR Part 50

Antitrust, Classified information, Fire prevention, Intergovernmental relations, Nuclear Power plants and reactors, Penalty, Radiation protection, Reactor siting criteria, Reporting requirements.

PART 50—DOMESTIC LICENSING OF PRODUCTION AND UTILIZATION FACILITIES

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974, as amended, and sections 552 and 553 of title 5 of the United States Code, the following amendments to Title 10, Chapter 1, Code of Federal Regulations, Part 50 are published as a document subject to codification.

1. The authority citation for Part 50 continues to read as follows:

Authority: Secs. 103, 104, 161, 182, 183, 88 Stat. 936, 937, 948, 953, 954, 955, 956, as amended (42 U.S.C. 2133, 2134, 2201, 2232, 2233, 2238), secs. 201, 202, 206, 88 Stat. 1243, 1244, 1248 (42 U.S.C. 5841, 5842, 5846), unless otherwise noted.

Section 50.78 also issued under sec. 122, 68 Stat. 939 (42 U.S.C. 2152). Sections 50.80-50.81 also issued under sec. 154, 68 Stat. 954, as amended; (42 U.S.C. 2234). Sections 50.100-50.102 issued under sec. 186, 68 Stat. 955; (42 U.S.C. 2236).

For the purposes of sec. 223, 68 Stat. 958, as amended (42 U.S.C. 2273), §§ 50.10 (a), (b), and (c), 50.44, 50.48, 50.68, 50.84, and 50.80(a) are issued under sec. 151b, 68 Stat. 948, as amended (42 U.S.C. 2201(b)); §§ 50.10 (b) and (c) 50.54 are issued under sec. 1611, 68 Stat. 949, as amended (42 U.S.C. 2201(f)); and §§ 50.57(e), 50.58(b), 50.70, 50.71, 50.72, and



Mr. James P. O'Reilly
Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

REPORTABLE OCCURRENCE 251-83-02

TURKEY POINT UNIT 4

DATE OF OCCURRENCE: APRIL 6, 1983

TECHNICAL SPECIFICATION 6.9.2.a.9

FUEL ASSEMBLY X-13

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9 to provide prompt notification of the subject occurrence.

Very truly yours,

J. M. Williford, Jr.
Vice President
Nuclear Energy

4/11/83

4/11/83

4/11/83, Office of the
Director, U.S. Nuclear
File 251.1 TP

TRANSMITTAL MANIFEST

NORTHERN STATES POWER COMPANY

NUCLEAR SUPPORT SERVICES DEPARTMENT

MONTICELLO NUCLEAR GENERATING PLANT

RCIC ECM Resistor Failure

Date: February 8, 1983

N-RO-83-02

USNRC		General Electric	4
Regional Admin-III	1	San Jose-Cust. Serv	
BCD	1	San Jose-Licensing	
Resident Inspector	1	San Jose-Fuel Proj. Mgr	
A V Diekhart	1	Mpls-I & SE Service Supv	
D E Gilberts	1	J W Sorensen (NUS)	1
L R Eliason	1	NSS File	1
G T Goering/C H Neils	1		
C E Larson	1	Safety Audit Committee	8
Monticello Plant		D M Musolf	
Oper Comm & File	8	M R Clarity	
Surveillance Coord	1	H S Isbin	
Q A Commitment File	1	D D Lanning	
Monti Training Section	1	T M Parker	
M M DeLong	1	W A Shanla	
External Coms Dept	1	J A This	
MPCA	1	F F Tierney	
Attn: J W Ferman		SAC File	Manifest Only
P W Kamman	1	USAR File	yes ___ no <u>X</u>
INPO	1	NRC Commitment	yes ___ no <u>X</u>



Northwestern Electric Power Company

614 North 4th
Minneapolis, Minnesota 55401
Telephone (612) 330-1000

2/15
600-576-009

February 8, 1983

Regional Administrator
Region III
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

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LIC 6
EVENT
BPO
NSAC

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

RCIC ECM Resistor Failure

The License Event Report for this occurrence is attached.

This event is reported in compliance with Technical Specification 4.7.2.2.b since it represents operation in a degraded mode permitted by a limiting condition for operation.

David Musolf for
David Musolf
Manager - Nuclear Support Services

DMM/KNC/dab

cc: NRC Document Control Desk (1)
NRC Resident Inspector
MPCA
Attn: J W Ferman

Attachment

0 5 0 0 0 2 6 3 0 2 0 7 1 8 3 0 3 1 0 1 8 3

During normal operation #12 RHR Hx dP not maintainable. B RHR SW loop inoperable to repair Hx control valve (CV-1729). T.S.4.5.C.1. SW loop to be inoperable for 7 days. Reported per T.S.4.7.3.1. repair completed A RHR SW loop made inoperable to repair Hx control valve. Repairs to both loops completed in 6 days. Two previous similar events (RO-81-02, RO-82-06). Redundant loop was operable and tested daily per T.S.4.5.C.1. No effect on public health or safety.

COMPLETE CODES: V A L L V I E X I
GOIN SUCCESS: X I
VALVE SUCCESS: G I
OCC. FRINGS CODE: 0 1 3 1
REPORT TYPE: L
REVISED: []
VALVE TESTED: N
LEAKAGE FORMULA: Y
DRIVE COIL SUCCESS: A
REPAIRS: P I A I S I P I S I

Root cause - piston ring wear. Fisher, 8", 657ED air actuated control valve. Piston ring wear due to normal flow induced vibration. Original graphite rings replaced with spring loaded teflon rings. RHR SW operability tests performed after repairs.

OTHER STATUS: NA
METHOD OF DISCOVERY: LA
AMOUNT OF ACTIVITY: NA
PERSONNEL EXPOSURES: NA
PERSONNEL INJURIES: NA
LOSS OF OR DAMAGE TO FACILITY: NA
PUBLISHED: NA

NAME OF PERSONNEL: G. L. Carpenter PHONE: (612) 330-1111

NOTICE

REGISTRATION

Date: March 9, 1963 4-20-83-03

USNR:	General Electric	4
Regional Admin-III	1	San Jose-Cust. Serv
DD	1	San Jose-Licensing
Resident Inspector	1	San Jose-Fuel Prod. Mgr
A V Dienhart	1	San Jose-IA Officer/Supv
D F Gilbreath	1	San Jose-IA Officer
L R Eliason	1	San Jose-IA Officer
G T Bertlinghoff	1	San Jose-IA Officer
C T Jones	1	San Jose-IA Officer
Monte L. ...		
Chas ...		
Supervisor ...		
A ...		
Monte ...		
M ...		
E ...		
M ...		
P ...		
NBS		
C ...		



3/15

Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

March 9, 1987

Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Tilton, IL 60137

0000600007

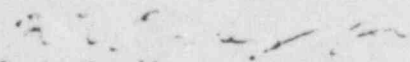
LER
LER # 21-000001
EVENT DATE 3/8/87
IMPO. RCVD. DATE 3/8/87
NSAC RCVD. DATE

MONTECELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

RHR SW Loops Inoperable

The license event report for this occurrence is attached.

This event is reported in compliance with Technical Specification
4.7.2.1.2 since it represents operation in a degraded mode permitted
for a limited number of days for operation.


David Musolf
Manager - Nuclear Support Services

DMM:KRM/nd

cc: NRC Document Control Desk (1)
NRC Resident Inspector
MPCA

Attn: J. W. Ferman

Attachment

NORTHERN STATES POWER COMPANY
 NUCLEAR SUPPORT SERVICES DEPARTMENT
 MONTICELLO NUCLEAR GENERATING PLANT

Low Flow on SGT "B" Train

Date: March 11, 1983

N-RO-83-04

USNRC		General Electric	4
Regional Admin-III	1	San Jose-Cust. Serv	
BCD	1	San Jose-Licensing	
Resident Inspector	1	San Jose-Fuel Proj. Mgr	
A V Dienhart	1	Mpls-1 & SE Service Supv	
D E Gilberts	1	J N Sorensen (NUS)	1
L R Eliason	1	NSS File	1
G T Goering/G H Neils	1		
C E Larson	1	Safety Audit Committee	8
Monticello Plant:		D M Musolf	
Oper Comm & File	8	M H Clarity	
Surveillance Coord	1	H S Isbin	
F L Johnson	1	D D Lanning	
Monti Training Section	1	T M Parker	
M M DeLong	1	W A Shamia	
External Comm Dept	1	J A Thie	
MPCA	1	F E Tierney	
Attn: J W Ferman		SA File	Manifest Onl
P H Kamman		USAR File	yes <input type="checkbox"/> no <input checked="" type="checkbox"/>
INPO	1	NRC Commitment	yes <input checked="" type="checkbox"/> no <input type="checkbox"/>

NSP

Northern States Power Company

614 Nicollet Street
Minneapolis, Minnesota 55401
Telephone (612) 338-3800

20000810

*J30
Ball*

LER	
LER#	<i>20000810</i>
EVENT DATE	
IMPO RCVD DATE	
NSAC RCVD DATE	

March 11, 1983

Regional Administrator
Region III
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Low Flow on SBC "B" Train

The License Event Report for this occurrence is attached.

This event is reported in compliance with Technical Specification 6.7.B.2.d since it represents an abnormal degradation at the Standby Gas Treatment System.

David Musolf for
David Musolf
Manager - Nuclear Support Services

DPM/KNC/js

cc: NRC Document Control Desk (1)
NRC Resident Inspector
MPCA
Attn: J W Ferman

Attachment

TRANSMITTAL MANIFEST

NORTHERN STATES POWER COMPANY

NUCLEAR SUPPORT SERVICES DEPARTMENT

MONTICELLO NUCLEAR GENERATING PLANT

Essential MCC 133A Feeder Breaker Trip

Date: May 11, 1983

N-RO-83-07

USNRC		General Electric	4
Regional Admin-III	1	San Jose-Cust. Serv	
DCO	1	San Jose-Licensing	
Resident Inspector	1	San Jose-Fuel Proj. Mgr	
A V Diebhart	1	Mpis-I & SE Service Supv	
D F Gilberts	1	J N Sorensen (NUS)	1
I R Ellason	1	NSS File	1
C T Goring/C H Neils	1		
C E Larson	1	Safety Audit Committee A	
Monticello Plant		D M Mus-JF	
Oper Comm & File	A	M H Clarity	
Surveillance Coord	1	B S Liskis	
R L Johnson	1	D B Goring	
Monti Training Section	1	T N Parker	
M M DeLong	1	V A Sorensen	
External Comm Dept	1		
NPCA	1		
Attn: J W Fernan			
P H Korman	1		
EXPO	1		



2
5/19

Northern States Power Company

514 Fourth Street
Northbrook, Minnesota 55057
Telephone (612) 430-4000

May 11, 1983

Regional Administrator
Region III
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. NUP-22

Essential MCC 133 Feeder Breaker Trip

The License Event Report for this occurrence is attached.

This event is reported in compliance with Technical Specification 4.7.3.2.b since it represents operation in a degraded mode permitted by a limiting condition for operation.

David Masoli for
David Masoli
Manager - Nuclear Support Services

0000672109

DSM/DMC/ld

cc: NRC Document Control Room (3)
NRC Resident Inspector
MPCA
Attn: J. M. [unclear]

LES
[unclear]

TRANSMITTAL MANIFEST

NORTHERN STATES POWER COMPANY

NUCLEAR SUPPORT SERVICES DEPARTMENT

MONTICELLO NUCLEAR GENERATING PLANT

Leak in 15A Feedwater Heater
Extraction Steam Trap Outlet Pipe

Date: May 13, 1983

N-RO-83-08

NSNRC		General Electric	4
Regional Admin-III	1	San Jose-Cust. Serv	
DOD	1	San Jose-Licensing	
Resident Inspector	1	San Jose-Fuel Prof. Mgr	
A. W. Dienhart	1	Mpls-I & SE Service Supv	
D. E. Gilberts	1	J. M. Sorensen (NUS)	1
L. R. Eliason	1	NSS File	1
G. T. Goering/G. H. Nells	1		
C. E. Larson	1	Safety Audit Committee 8	
Monticello Plant		D. M. Musolf	
Oper Comm & File	8	M. H. Clarity	
Surveillance Coord	1	H. S. Isbin	
R. L. Johnson	1	D. D. Lanning	
Monti Training Section	1	I. M. Parker	
M. M. DeLong	1	W. A. Shamba	
External Comm Dept	1	J. A. Thie	
NPCA	1	V. P. Tierney	
ATTN: J. W. Farmer	1	BAC File	Monticello
P. H. Kannan	1	OSAR File	
INPR	1	NRC Commitment	



Handwritten: N
5/21

Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

May 16, 1983

Regional Administrator
Region III
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Leak in 15A Feedwater Heater Extraction Steam Trap Outlet Pipe

The License Event Report for this occurrence is attached.

This event is reported in compliance with Technical Specification
6.7.B.2.d since it represents an abnormal degradation of the Reactor
Feedwater System.

Handwritten: David Musolf for
David Musolf
Manager - Nuclear Support Services

DHM/KNC/js

cc: NRC Document Control Desk (1)
NRC Resident Inspector
NPCA
Attn: J.W. Ferman

Handwritten: KI
KJB

Handwritten: 8000424 A06

LER
LER # 263-83008
INSTR # 100
DATE 5/21/83
BY [Signature]
NSP

Attachment

TRANSMITTAL MANIFEST
 NORTHERN STATES POWER COMPANY
 NUCLEAR SUPPORT SERVICES DEPARTMENT
MONTICELLO NUCLEAR GENERATING PLANT

HPCI REF Ground Fault

Date: May 18, 1983

M-20-83-09

USNRC		General Electric	4
Regional Admin-III	1	San Jose-Cust. Serv	
DCD	1	San Jose-Licensing	
Resident Inspector	1	San Jose-Fuel Proj. Mgr	
A V Dienhart	1	Mpls-I & SE Service Supv	
L E Gilberts	1	J W Sorensen (NUS)	1
L R Eliason	1	NSS File	1
G T Goering/G N Neils	1		
C E Larson	1	Safety Audit Committee	8
Monticello Plant		D M Musolf	
Oper Comm & File	8	M H Clarity	
Surveillance Coord	1	H S Iobin	
R L Johnson	1	D D Lanning	
Monti Training Section	1	T H Parker	
M H DeLong	1	W A Shanks	
External Comm Dept	1	J & Th's	
MPCA	1	F P Tierney	
Attn: J W Ferman		SEC File	Manifest
P H Lawson	1	SEC File	yes
IRPO	1	SEC Commitment	yes
A V Reichowski	1	K N Curley	1

NRP

N

37

Northwest States Power Company
11111
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11111

May 18, 1983

Regional Administrator
Region III
U S Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-422

RPCI EGF Ground Fault

The License Event Report for this occurrence is attached.

This event is reported in compliance with Technical Specification
6.7.E.1.e since it represents failure of a component which could
prevent the fulfillment of the functional requirements of a system
us * to cope with accidents analyzed in the EAR.

David Musolf for
David Musolf
Manager - Nuclear Support Services

DPM/KNC/jw

cc: NRC Document Control Desk (1)
NRC Resident Inspector
MPCA
Attn: J W Burman

Attachment

*KE
KSA*

SEARCHED
INDEXED
SERIALIZED
FILED
MAY 20 1983
FBI - MEMPHIS

Mr. J. M. Allan, Acting Administrator
Office of Inspection & Enforcement, Region
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19106

Dear Mr. Allan:

This L.E.R. deals with the failure of a smoke detector in the drywell
to annunciate. Applicable Tech. Spec. 3.14.C.1.

8305120354