

For Docket  
Only 1-2

NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION  
AND NOTICING ACTION

Docket No. 50-325 and 50-324 Facility: ~~50-325~~ Units 1 and 2  
Licensee: Carolina Power & Light Co. Date of application: June 3, 1982  
Request for:

(See attached.)

Initial Determination:

- Proposed determination - amendment request involves no significant hazards considerations (NSHC).
- Final determination - amendment request involves significant hazards considerations (SHC).

Basis for Determination

- Licensee's NSHC discussion has been reviewed and is accepted. See attached amendment request.
- Basis for this determination is presented in the attached.
- Other (state): \_\_\_\_\_

(Attach additional sheets as needed.)

Initial Noticing Action: (Attach appropriate notice or input for monthly FRN)

1.  Monthly FRN. Notice of opportunity for hearing (30 days) and request for comments on proposed NSHC determination - monthly FRN input is attached (Attachment 8).
2.  Individual FRN (30 days). Same notice matter as above. Time does not allow waiting for next monthly FRN (Attachments 9a and 9b).

(THIS FORM SHOULD BE TYPED EXCEPT FOR UNUSUAL, URGENT CIRCUMSTANCES.)

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Single Soap  
Jan 1

Request for:

The amendment would change the Technical Specifications to incorporate revised safety and operating limits associated with the operation of Brunswick Steam Electric Plant, Units 1 and 2 with one recirculation loop out of service. The changes proposed by the licensee would provide for reduced Average Power Range Monitor (APRM) flux scram trip and rod block settings, an increase in the safety limit Minimum Critical Power Ratio (MCPR) value and revisions to the allowable Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) values suitable for use with an idle recirculation loop. Presently the Brunswick Technical Specifications would require plant shutdown if an idle recirculation loop cannot be returned to service within 24 hours. The amendment would authorize the plant to operate up to 50% of rated power for extended periods of time. Supporting the amendment request, is a report prepared by General Electric that presents the analysis for core performance.

Basis for:

The Commission has provided guidance for the application of the standards for determining whether a significant hazards consideration exists by providing examples of amendments that are considered not likely to involve significant hazards consideration (48 FR 14870). One such amendment involves a relief granted upon demonstration of acceptable operation from an operating restriction that was imposed because acceptable operation was not yet demonstrated. This assumes that the operating restriction and the criteria to be applied to a request for relief have been established in a prior review and that it is justified in a satisfactory way that the criteria have been met.

The Brunswick Technical Specifications presently require plant shutdown if an idle recirculation loop cannot be returned to service within 24 hours. This restriction was imposed because insufficient information existed to enable the staff to establish criteria for operation with an idle recirculation loop. Although such criteria have since been established, and analyses have indicated that it should be safe to operate Boiling Water Reactors (BWRs) on a single loop in the range of 85% power, operating experience with an idle recirculation loop at Browns Ferry 1 (BF-1) in late 1979 raised concerns about authorizing single loop operation for BWRs. When the Tennessee Valley Authority (TVA) tried to increase power at BF-1 above about 59% of rated power while operating on a single loop, unexpected variations in jet pump flow, neutron flux, and related parameters were noted. Neither the causes nor the potential effects of these variations have been determined or reviewed by the staff for operation with a single recirculation loop. Thus, it has not been justified in a satisfactory way that the criteria for operation with a single loop have been met. The application for amendment involves changes which do not satisfy the criteria of the applicable example of an amendment which would likely be found not to involve significant hazards consideration. Therefore, the staff has made a determination that the application for amendment may involve a significant hazards consideration.

- 3. ( ) Local media notice. Valid exigent circumstances exist (evaluated below). Local media notice requesting public comments on proposed NSHC determination is attached (Attachment 10).
- 4. ( ) No notice. A valid emergency situation exists (evaluated below) and there is no time for public notice on proposed NSHC determination. (No attachment.)
- 5. ( ) Individual FRN (30-days). Licensee's claim of exigent or emergency circumstances is invalid (evaluated below). Notice of opportunity for hearing (30 days) and request for comments on proposed NSHC determination is attached (Attachments 9a and 9b). Letter of explanation to licensee is also attached.
- 6. (X) Individual FRN (30-days). The amendment request involves SHC. Notice of opportunity for prior hearing is attached (Attachment 5). Letter to licensee also attached.
- 7. ( ) Individual Short FRN. Valid emergency circumstances exist (evaluated below). There is no time for the usual 30-day FRN. (Attachment 16).

Evaluation of exigent or emergency circumstances (if applicable):

(attach additional sheets as needed)

Approvals:

- | <u>Approvals:</u>   | <u>Date</u>                          |
|---|--------------------------------------|
| 1. <u>Sam Mac Kay</u><br>(Project Manager)                | <u>September 22, 1983</u>            |
| 2. <u>[Signature]</u><br>(Branch Chief)                   | <u>10/3/83</u>                       |
| 3. <u>M. Korman</u><br>(see attached letter (OELD) (NRK)) | <u>9/27/83</u><br><u>Oct 3, 1983</u> |

Additional approval (for noticing actions types 3, 4, 5, 6 and 7):

- |   |                |
|---|----------------|
| 4. <u>[Signature]</u><br>(Assistant Director) | <u>10/5/83</u> |
|---|----------------|

Additional approval (for noticing action types 4 and 5):

- |   |       |
|---|-------|
| 5. _____<br>(Director, Division of Licensing) | _____ |
|---|-------|

Attachment: as indicated

cc: Original - Docket File (with note "Docket File only")  
Project Manager  
Licensing Assistant  
Branch Files



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FEB 2 1984

VASSALLO

MEMORANDUM FOR: ~~G. C. Laines~~, Assistant Director  
for Operating Reactors, DL

FROM: L. S. Rubenstein, Assistant Director  
for Core and Plant Systems, DSI

SUBJECT: SLO OPERATION SER'S

As discussed in our meeting of January 26, we are withdrawing our SER approvals for all the plants currently requesting permanent SLO. This decision is based on new data which indicates the potential for local thermal hydraulic instabilities which would not be detected by only monitoring APRM flux noise, which we previously recommended.

We are continuing to evaluate this problem and expect to establish criteria for acceptable SLO in the near future.

*Alan L. Rubin*  
L. S. Rubenstein, Assistant Director  
for Core and Plant Systems, DSI

cc: R. Mattson  
D. Eisenhut  
L. Phillips  
R. Lobel

Contact: G. Schwenk, CPB:DSI  
X-29421

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DEC 16 1982

Docket No. 50-373

Mr. Louis O. DelGeorge  
Director of Nuclear Licensing  
Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

Dear Mr. DelGeorge:

Subject: Amendment No. 11 to Facility Operating License No. NPF-11 - La Salle  
County Station, Unit No. 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 11 to Facility Operating License No. NPF-11 for La Salle County Station, Unit 1. This Amendment is in response to your letter dated December 6, 1982, and authorizes operation at La Salle County Station, Unit 1 with one recirculation loop out of service. This single recirculation loop operation is granted for only the first fuel cycle and only up to 50 percent of rated power. Single loop operation at 50 percent of rated power at Browns Ferry, Unit 1 and several other plants, has shown acceptable flow and power characteristics. Although analyses indicate that operation above 50 percent power with one recirculation loop may be safe, the experience at Browns Ferry Unit 1 has caused concern about flow and power oscillations at such power levels. The staff, therefore, may require additional information in this area of stability for approval for single recirculation loop operation beyond the first cycle. It is important to note that the enclosed Amendment adds a license condition requiring a power level reduction and reduced safety system settings to provide an equivalent level of safety for operation with one recirculation loop out of service.

A copy of the related safety evaluation supporting Amendment No. 11 to Facility Operating License NPF-11 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 11 to NPF-11
2. Safety Evaluation Report
3. Federal Register notice

cc w/enclosures:  
See next page

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La Salle

Mr. Louis O. DelGeorge  
Director of Nuclear Licensing  
Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

cc: Philip P. Steptoe, Esquire  
Suite 4200  
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Assistant Attorney General  
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Chairman  
La Salle County Board of Supervisors  
La Salle County Courthouse  
Ottawa, Illinois 61350

Attorney General  
500 South 2nd Street  
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Department of Public Health  
Attn: Chief, Division of Nuclear Safety  
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Springfield, Illinois 62761

The Honorable Tom Corcoran  
United States House of Representatives  
Washington, D. C. 20515

Chairman  
Illinois Commerce Commission  
Leland Building  
527 East Capitol Avenue  
Springfield, Illinois 62706



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-11  
Amendment No. 11

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
  - A. The application for amendment filed by the Commonwealth Edison Company, dated December 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this Amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; set forth in 10 CFR Chapter I;
  - D. The issuance of this Amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this Amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by the addition of Paragraph 2.C.(34) to the Facility Operating License No. NPF-11 to read as follows:

(34) Through the First Fuel Cycle of Plant Operation, Technical Specification 3.4.1.1 is Modified for One Recirculation Loop out of Service with Provisions

- (a) The steady-state thermal power level will not exceed 50 percent of rated power.
- (b) The minimum critical power ratio (MCPR) safety limit will be increased by 0.01 to 1.07.

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- (c) The minimum critical power ratio limiting condition for operation (LCO) will be increased by 0.01.
- (d) The maximum average planar linear heat generation (MAPLHGR) limit will be reduced by 0.85.

(e) Technical Specifications Setpoints shall read as follows:

T. S. 2.2.1       $S \leq 0.66W + 45.7$  (Trip Setpoint)  
                     $S \leq 0.66W + 48.7$  (Allowable)

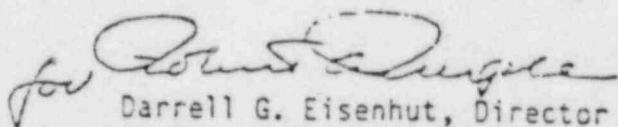
T. S. 3.2.2       $S \leq (0.66W + 45.7) T^*$   
                     $S \leq (0.66W + 36.7) T^*$   
                    RB  
                     $T^*$  as defined in T. S. 3.2.2

T. S. 3.3.6      APRM Upscale  $\leq 0.66W + 36.7$  (Trip Setpoint)  
                    APRM Upscale  $\leq 0.66W + 39.7$  (Allowable)  
                    RBM Upscale  $\leq 0.66W + 34.7$  (Trip Setpoint)  
                    RBM Upscale  $\leq 0.66W + 37.7$  (Allowable)

- (f) The average power range monitor (APRM) flux noise will be measured once per shift; and the recirculation loop flow will be reduced if the flux noise averaged over 1/2 hour exceeds 5 percent peak to peak, as measured by the APRM chart recorder.
- (g) The core plate delta P noise will be measured once per shift, and the recirculation loop flow will be reduced if the noise exceeds one (1) psi peak-to-peak.

3. This Amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Date of Issuance: DEC 16 1982

# SAFETY EVALUATION

AMENDMENT NO. 41 TO LICENSE NPF-11

LA SALLE COUNTY STATION, UNIT NO. 1

POCKET NO. 50-373

## 1.0. INTRODUCTION

The current La Salle County Station, Unit 1 Technical Specifications do not allow plant operation beyond 12 hours if an idle recirculation loop cannot be returned to service. The ability to operate at reduced power with a single loop is highly desirable from an availability outage planning standpoint in the event that maintenance or component unavailability rendered one loop inoperable.

By letter dated December 6, 1982, Commonwealth Edison Company (CECo) (the licensee) requested changes to the Technical Specification for Single Loop Operation of La Salle, Unit 1. The requested changes would permit Unit 1 to operate at up to 50 percent of rated power with one recirculation loop out of service for unlimited time. While analyses indicate that it may be safe to operate boiling water reactors on a single loop in the range higher than 50 percent of rated power, the experience (reference letter from L. M. Mills, TVA dated March 17, 1980 to H. Denton, NRC) at Browns Ferry, Unit 1 has caused concern about flow and power oscillations. However, because single loop operation at 50 percent of rated power at several plants, including Browns Ferry, Unit 1, has shown acceptable flow and power characteristics, we will permit La Salle, Unit 1 to operate at power levels up to 50 percent of rated power with one loop out of service during its initial fuel cycle. We will request from the licensee any additional information required for permanent approval for single loop operation. If requested, we will also consider operation at a higher power level for La Salle with one recirculation loop out of service after the staff concerns stemming from Browns Ferry, Unit 1 single loop operation, which showed unexpected variation in jet pump flow and neutron flux at power level of 59 percent but these variations stopped when the power level was reduced, are satisfied.

## 2.0 EVALUATION

### 2.1 Accidents (Other than Loss of Coolant Accident (LOCA)) and Transients Affected by One Recirculation Loop out of Service

#### 2.1.1 One Pump Seizure Accidents

The licensee states that the one-pump seizure accident is a relatively mild event during two recirculation pump operations. Analyses were performed to determine the impact this accident would have on one recirculation pump operation. These analyses were performed using NRC staff approved models. The analyses were conservatively conducted from a steady-state operating condition of 78 percent of rated thermal power and 63 percent core flow. Pump seizure was simulated by setting the single operating pump speed to zero instantaneously.

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Results of the analyses indicate that neither pressure nor cladding thermal limits are exceeded during the event. Peak vessel pressure is calculated to be 1031 psig (ASME code limit is 1375 psig) and minimum critical power ratio (MCPR) is 1.17 (safety limit is 1.06).

### 2.1.2 Abnormal Operational Transients

The licensee discussed the effects of single loop operation on the course of abnormal operational transients. Pressurization and cold water increase events, as well as rod withdrawal error, were addressed. Flow decrease is covered by the pump seizure accident already described. The results of calculations for the limiting event for each category were also presented. Initial operating conditions were conservatively assumed to be 78 percent of rated thermal power and 63 percent core flow.

#### 2.1.2.1 a) Pressurization Events

The limiting pressurization event is the generator load rejection without bypass transient. For single loop operation, the licensee has calculated that the maximum vessel pressure is 1128 psig and the MCPR is 1.29. Each of the values satisfies its respective safety limit.

#### b) Cold Water Increase

The limiting cold water increase event is the feedwater controller failure to maximum demand transient. The reactor is assumed to be in single loop operation at conservative initial operating conditions of 78 percent of rated power and 63 percent core flow when failure of the feedwater control system instantaneously increases the feedwater flow to the runout capacity of 160 percent of rated flow. The peak pressure is calculated to be 1126 psig and the MCPR is 1.26, each satisfying its respective safety limit.

#### c) Rod Withdrawal Error

The rod withdrawal error at rated power is given in the Final Safety Analysis Report for the initial core and in cycle dependent reload supplemental submittals. These analyses are performed to demonstrate that, even if the operator ignores all instrument indications and the alarms which could occur during the course of the transient, the rod block system will stop rod withdrawal at a minimum critical power ratio which is higher than the fuel cladding integrity safety limit. Correction of the rod block equation and lower initial power for single-loop operation in the Technical Specifications (see Section 5 of this safety evaluation) assures that the MCPR safety limit is not violated.

One-pump operation results in backflow through 10 of the 20 jet pumps while flow is being supplied to the lower plenum from the active jet pumps. Because of this backflow through the inactive jet pumps, present rod-block equation and APRM settings must be modified. The licensee has modified the two-pump rod block equation and average power range monitor (APRM) settings that exist in the Technical Specifications for one-pump operation and the staff has found them acceptable.

The staff finds that one loop transients and accidents other than LOCA, which is discussed below, are bounded by the two loop operation analyses and are therefore acceptable.

### 2.1.3 Minimum Critical Power Ratio (MCPR) Uncertainties

For single-loop operation, the rated condition steady-state MCPR limit is increased by 0.01 to account for increased uncertainties in the core total flow and traversing in-core probe (TIP) readings. The MCPR will vary depending on flow conditions. This leads to the possibility of a large inadvertent flow increase which would cause the MCPR to decrease below the safety limit for a low initial MCPR at reduced flow conditions. Therefore, the required MCPR must be increased at reduced core flow by a flow factor  $K_f$ . The  $K_f$  factors for two loop operation are derived assuming both recirculation loop controllers fail. This condition of both recirculation loop controllers failing maximizes the power increase and hence bounds the maximum delta MCPR for single loop operation transients. When operating on one loop, the flow and power increase will be less than that associated with two loops at full flow. The  $K_f$  factors derived from the two-loop assumptions are therefore conservative for single loop operation.

### 2.2 Loss of Coolant Accident (LOCA)

The licensee has performed analyses of a spectrum of recirculation suction line breaks under single loop operation conditions. The licensee states that evaluation of these calculations which are performed according to the procedure outlined in NEDO-20556-2, Rev. 1, "General Electric Company Analytical Model for Loss-of-Coolant Analysis in Accordance with 10 CFR 50 Appendix K - Amendment No. 2 One Recirculation Loop out of Service," indicates that a multiplier of 0.87 should be applied to the maximum average planar linear heat generation rate (MAPLHGR) limits for single loop operation of La Salle, Unit 1. Other plants using 3 x 8R fuel have been required to use a reduction factor of 0.85. We require that La Salle, Unit 1 likewise reduce its MAPLHGR by 0.85.



### 3.0 THERMAL HYDRAULICS

The licensee has confirmed that analysis uncertainties are independent of whether flow is provided by two loops or a single loop. The only exceptions to this are core total flow and TIP reading. The effect of these uncertainties is an increase in the MCPR by .01, which is more than offset by the  $K_f$  factor required at low flows. The steady state operating MCPR with single-loop operation will be conservatively established by multiplying the rated flow MCPR limit by the  $K_f$  factor.

### 4.0 STABILITY ANALYSIS

As indicated in the applicant's submittal, operating along the minimum forced recirculation line with one pump running at minimum speed is more stable than operating with both pumps operating at minimum speed.

The licensee will be required to operate in master manual to reduce the effects of instabilities due to controller feedback. The staff has accepted previous stability analyses results as evidence that the core can be operated safely while our generic evaluation of boiling water reactor stability characteristics and analysis methods continues. The previous stability analysis results include natural circulation conditions and thus bound the single loop operation. In addition, the decay ratio (0.50) predicted for initial cycle for Unit 1 shows margin relative to Browns Ferry, Unit 1 (.83) which had the flow noise oscillations during single recirculation loop operation. We conclude that with appropriate limitations to recognize and avoid operating instabilities, that the reactor can be operated safely in the single loop mode. Our evaluation of the flow/power oscillations evidenced in Browns Ferry will continue and any pertinent conclusions resulting from this study will be applied to La Salle, Unit 1.

### 5.0 SUMMARY ON SINGLE LOOP OPERATION

#### 5.1 Steady State Thermal Power Level will not exceed 50 percent of Rated Power

Operating at 50 percent of rated power with appropriate Technical Specification (TS) changes has been approved on a cycle basis for several operating plants. It is concluded for La Salle, Unit 1 that for operation at 50 percent of rated power, consistent with the provisions of item h below, that transient and accident bounds will not be exceeded.

#### 5.2 Minimum Critical Power Ratio (MCPR) Safety Limit Will Be Increased by 0.01 to 1.07

The MCPR Safety Limit will be increased by 0.01 to account for increased (TIP) readings. The licensee has determined that the change conservatively bounds the uncertainties introduced by single loop operation.



5.3 Minimum Critical Power Ratio (MCPR) Limiting Condition for Operation (LCO) will be increased by 0.01

The staff requires that the operating limit MCPR be increased by 0.01 and multiplied by the appropriate two loop  $K_f$  factors that are in the La Salle, Unit 1 TS. This will preclude an inadvertent flow increase from causing the MCPR to drop below the safety limit MCPR.

5.4 The Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) Limits will be Reduced by Appropriate Multipliers

The licensee proposed reducing the TS MAPLHGR by 0.87 for Single Loop Operation. These reductions were based on an analysis method proposed by General Electric in NEDE-20566-2. We require a reduction factor of 0.85 consistent with previous single recirculation loop operation approvals for plants with 8 x 8R fuel. This change has been discussed with and agreed to by licensee.

5.5 The APRM Scram and Rod Block Setpoints will be Reduced

The licensee proposed to modify the two loop APRM Scram, Rod Block and Rod Block Monitor (RBM) setpoints to account for back flow through half the jet pumps. These setpoint equations will be changed in the La Salle, Unit 1 TS. The above changes are similar to other plant TS changes and are acceptable to the staff.

5.6 The Recirculation Control will be in Manual Control

The staff requires that the licensee operate the recirculation system in the manual mode to eliminate the need for control system analyses and to reduce the effects of potential flow instabilities.

5.7 Surveillance Requirements

The staff requires that the licensee perform daily surveillance on the jet pumps to ensure that the pressure drop for one jet pump in a loop does not vary from the mean of all jet pumps in that loop by more than 5 percent.

5.8 Provisions to Allow Operation with One Recirculation Loop Out of Service

- a. The steady-state thermal power level will not exceed 50 percent of rated power.
- b. The Minimum Critical Power Ratio (MCPR) Safety Limit will be increased by .01 to 1.07.



8.0 CONCLUSION

We have concluded, based on the considerations discussed above, that, (1) because the Amendment does not involve a significant increase in the probability or consequences or accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, the Amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this Amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: DEC 16 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-373COMMONWEALTH EDISON COMPANYNOTICE OF ISSUANCE OF AMENDMENT OF FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 11 to Facility Operating License No. NPF-11, issued to Commonwealth Edison Company, which adds License Condition 2.C.(34) for operation of the La Salle County Station, Unit No. 1 (the facility) located in Brookfield Township, La Salle County, Illinois.

The Amendment authorizes the operation of La Salle County Station, Unit 1 with one recirculation loop out of service for the first fuel cycle only. Also, the Amendment adds a license condition limiting a power level to 50 percent of full power or less during the operation with one recirculation loop out of service.

The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this Amendment was not required since the Amendment does not involve a significant hazards consideration.

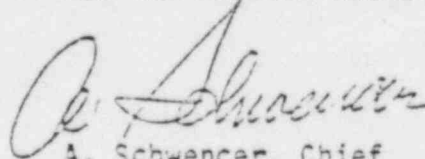
The Commission has determined that the issuance of this Amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this Amendment.

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For further details with respect to this action, see (1) the application for amendment dated December 6, 1982; (2) Amendment No. 11 to License No. NPF-11 dated December 16, 1982; and (3) the Commission's related safety evaluation. All of these items are available for public inspection at the Commission's Public Document Room located at 1717 H Street, NW, Washington, DC 20555, and the Public Library of Illinois Valley Community College, Rural Route No. 1, Ogelsby, Illinois 61348. A copy of items (1), (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 16th day of December 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing