

U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Reports No. 50-315/84-10(DPRP); 50-316/84-11(DPRP)

Docket Nos. 50-325; 50-316

Licenses No. DPR-58; DPR-74

Licensee: American Electric Power Service Corporation
Indiana and Michigan Electric Company
1 Riverside Plaza
Columbus, OH 43216

Facility Name: Donald C. Cook Nuclear Power Plant, Units 1 and 2

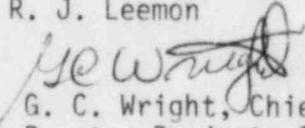
Inspection At: Donald C. Cook Site, Bridgman, MI

Inspection Conducted: May 1, 1984 through June 10, 1984

Inspectors: E. R. Swanson

J. K. Heller

R. J. Leemon

Approved By:  G. C. Wright, Chief
Reactor Projects Section 2A

6/29/84
Date

Inspection Summary

Inspection on May 1, 1984 through June 10, 1984 (Reports No. 50-315/84-10(DPRP); 50-316/84-11(DPRP))

Areas Inspected: Routine unannounced inspection by the resident inspectors of operational safety; maintenance; surveillance; Licensee Event Reports; licensee action on previous inspection findings; I.E. Bulletins; Operator Licensing; and management meeting - Regulatory Performance Improvement Program (RPIP). The inspection involved a total of 279 inspector-hours by three NRC inspectors including 68 inspector-hours off-shift.

Results: No items of noncompliance, deficiencies or deviations were identified.

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DETAILS

1. Persons Contacted

- a. Personnel attending May 3, 1984 Regulatory Performance Improvement Meeting.

American Electric Power Service Corporation (AEP)

M. P. Alexich, Vice President Nuclear Engineering (AEPSC)
W. G. Smith, Jr., Plant Manager
B. A. Svensson, Assistant Plant Manager
E. L. Townley, Assistant Plant Manager
R. F. Kroeger, Quality Assurance Manager (AEPSC)
T. P. Beilman, Quality Assurance Supervisor (AEPSC)
J. F. Stietzel, Quality Control Superintendent
B. H. Bennett, Assistant Manager Nuclear Engineering (AEPSC)
P. A. Barrett, Safety and Licensing (AEPSC)
E. A. Smarrella, Staff Assistant
R. S. DiStefano, Nuclear Operation Support (AEPSC)

NRC Region III Attendees

W. D. Shafer, Chief, Projects Branch No. 2
G. C. Wright, Chief, Projects Section 2A (effective April 30, 1984)
E. R. Swanson, Senior Resident Inspector
J. K. Heller, Resident Inspector
R. J. Leemon, Resident Inspector

- b. Personnel Contacted During Inspection Activities

*W. G. Smith, Jr., Plant Manager
*E. Townley, Assistant Plant Manager
*B. Svensson, Assistant Plant Manager
*T. Kriesel, Technical Superintendent - Physical Science
*A. Blind, Technical Superintendent - Engineering
*K. Baker, Operations Superintendent
*D. Dudding, Maintenance Superintendent
J. Wojcik, Chemical Supervisor
*J. Stietzel, Quality Control Superintendent
D. Palmer, Plant Radiation Protection Supervisor

The inspectors also contacted a number of licensee and contract employees and informally interviewed operation, technical and maintenance personnel during this period.

*Denotes personnel attending exit interview on June 18, 1984.

2. Licensee Action on Previous Inspection Findings

(Closed) Noncompliance Item 316/84-02-04(DPRP): The opening and closing force of the Ice Condenser Lower Inlet Doors was determined with a scale range other than specified in the surveillance procedure. The licensee's letter of May 4, 1984, (AEP:NRC:0883) documented that the errors induced by using the wrong scale did not result in exceeding the acceptance criteria. In addition, the licensee reinforced to plant personnel the importance of following procedures and the use of temporary change sheet.

No items of noncompliance or deviations were identified.

3. Operational Safety Verification

The inspectors observed control room operations including shift turnover, reviewed applicable logs and conducted discussions with control room operators during the period of May 1, through June 10, 1984. The inspectors verified the operability of selected emergency systems, reviewed tagout records, verified proper return to service of affected components and verified a portion of the containment isolation lineup. Tours of Unit 1 and 2 auxiliary building, turbine building, and screenhouse and Unit 2 upper and lower containment were conducted to observe plant equipment conditions, including potential fire hazards, fluid leaks, and excessive vibrations and to verify that maintenance requests had been initiated for equipment in need of maintenance. The inspectors found that the top access cover for the Unit 1 Condensate Storage Tank was secured in place with three of approximately twenty-one bolts (the remaining bolts were lying next to the cover). This was discussed with the Maintenance Superintendent.

The inspectors by observation and interview verified that the physical security plan was being implemented in accordance with the station security plan. During interviews with contractor security personnel the inspectors identified examples where guards were not cognizant of their post orders. These examples were brought to the attention of plant management.

The inspector performed a walkdown/review of the systems listed below to verify that: each accessible flow path valve was in its correct position; power (visual breakers and fuses) was aligned to actuate on automatic signal; essential instrumentation was operable and; no condition existed that degraded the system.

- a. Unit 1 Containment Spray Heat Exchangers and Eductor's using licensee procedure 1 OHP 4030 STP.007, Data/Signoff Sheet 6.5 and 6.6 and print OP 1-5144.
- b. Unit 1 Safety Injection System using print OP 1-5142 .
- c. Unit 2 4160 VAC Buses, 600 VAC Buses, 480 VAC Buses, 250 VDC Buses and 120 VAL.
- d. Unit 2 Diesel Generators and Support Systems using print OP 1-5150.

During these walkdowns the inspector found a number of heat tracing thermostats with the protective covers missing; several electrical junction boxes with the covers off or the covers secured in place with one of two screws; a number of electrical conduit access boxes with the covers missing; and one electronic control panel with a sign stenciled on the door "Danger-Energized" but a side panel was missing. The inspector provided the licensee with pictures of these items. The licensee is taking action to insure that protective covers are replaced.

The inspector also discovered a fire watch who appeared not to be attentive to his duties. This was later identified to the fire protection coordinator who stated that the fire watch, a contractor employee, had been terminated.

The inspector observed a number of Auxiliary Equipment Operators (AEO) dressed in Anti-contamination clothing (lab coats, gloves and shoe covers) leaving a contamination controlled area by removing the shoe covers and gloves but not removing the lab coats. Through discussion with AEO's the inspector found it was common practice to maintain control of the lab coats during the shift tours and then pass them on to the next shift. The AEO's or a Radiation Protection Supervisor could not identify when the lab coats would be sent to the laundry. In addition, the inspector noted, that for some contaminated controlled areas, the contaminated clothing containers were located outside the contaminated area and in some cases a number of feet away from the step-off boundary. The inspector discussed these items with the Health Physic Supervisor who agreed to review these matters. A memorandum documenting the policy to turn in lab coats at least shiftly was issued on June 15, 1984.

No items of noncompliance or deviations were identified.

4. Monthly Surveillance Observation

The inspector reviewed Technical Specifications required surveillance testing on the systems listed below and verified that testing was performed in accordance with adequate procedures, that test instrumentation was calibrated, that limiting conditions for operation were met, that removal and restoration of the affected components were accomplished, that test results conformed with technical specifications and procedure requirements and were reviewed by personnel other than the individual directing the test, and that any deficiencies identified during the testing were properly reviewed and resolved by appropriate management personnel.

Unit 1

**1 OHP 4030 STP.027

Unit 1 Emergency Diesel Generator Operability Test

**1 OHP 4021.032.001

Starting, Loading, Paralleling, and Shutting down the Emergency Diesel Generator

Unit 2

**2 OHP 4030 STP.027	Unit 2 AB Diesel Generator Operability Test
**2 THP 4030 STP.203	Type B and C Leak Rate Testing
**2 THI 4030 STP.217A	Unit 2 Emergency Diesel Generator Operability Test

No items of noncompliance or deviations were identified.

5. Monthly Maintenance Observation

Station maintenance activities of safety related systems and components listed below were observed and/or reviewed to ascertain that they were conducted in accordance with approved procedures, regulatory guides and industry codes or standards and in conformance with Technical Specifications.

The following items were considered during this review: the limiting conditions for operation were met while components or systems were removed from service; approvals were obtained prior to initiating the work; activities were accomplished using approved procedures and were inspected as applicable; functional testing and/or calibrations were performed prior to returning components or systems to service; quality control records were maintained; activities were accomplished by qualified personnel; parts and materials used were properly certified; radiological controls were implemented; and, fire prevention controls were implemented.

Work requests were reviewed to determine status of outstanding jobs and to assure that priority is assigned to safety related equipment maintenance which may affect system performance.

The following maintenance activities were observed and/or reviewed:

**1 THP 4030 STP.234	Calibration of Acoustic Valve Monitoring System
QCI 5030, Attachment 11	Weekly inspection of Exciter Brushers
**2 THP 6030 IMP 232	Nuclear Instrumentation Audio Count Rate
Job Order 85450	Adjustment of timers on charcoal filters
**12 MHP 5021.001.031	Installation of Fire Barrier Penetration Seals
**12 MHP 5021.001.060	Fire Wrap Installation Procedures

No items of noncompliance or deviations were identified.

6. Licensee Event Report Followup

Through direct observation, discussions with licensee personnel, and review of records, the following event reports were reviewed to determine that reportability requirements were fulfilled, immediate corrective action was accomplished, and corrective action to prevent recurrence had been accomplished or initiated in accordance with Technical Specifications. The following items were closed.

Unit 1

RO 83-072/03X-0 & 1	B&C Leak Rate Test failure
RO 83-074/03L-0	Copper lug on aluminum 4 KV lines
RO 83-091/03L-0	Fire Door inoperable
RO 83-093/03L-0	Seismic monitoring instrument inoperable
RO 83-094/03L-0	Loss of voltage relays
RO 83-095/01T-0	Nonessential service water leak
RO 83-096/03L-0	Fire Door inoperable
RO 83-097/03L-0	Fire Door inoperable
RO 83-099/03L-0	Ice Condenser flow passages partially blocked
RO 83-106/03L-0	Steam Generator Snubber failure

Unit 2

RO 83-056/03L-0	Ice Condenser Inlet Door exceeded opening force
RO 83-074/03L-0	Diesel Generator Synchronizing Scope inoperable
RO 83-075/03L-0	Steam Generator Blowdown sample valve inoperable
RO 83-076/01T-0	NESW leakage from Fire Water Strainer
RO 83-078/03L-0	Diesel Generator inoperable due to trip cylinder air leak
RO 83-082/03L-0	East Containment Spray Heat Exchanger inlet valve body-to-bonnet leak
RO 83-083/03L-0	Ice Condenser Lower Inlet Doors exceeding opening force
RO 83-085/03L-0	Reactor Coolant pressure less than 2200 PSIA following turbine loading
RO 83-086/03L-0	Sump Flow Recorder inoperable
RO 83-087/03L-0	Pressurizer pressure reference leg leak

No items of noncompliance or deviations were identified.

7. Operator Licensing

On May 4, 1984 the inspector was notified by the Plant Manager that the NRC had renewed the license of an individual who had not met the requirements of the licensee's Licensed Operator Requalification Training Program. Listed below is the chronology of events which led to the above situation:

- The individual received a Reactor Operator license as a Shift Technical Advisor. This license was considered, by the licensee, to be a nonoperating license (no hands-on at the plant) and was obtained for technical background.

- The individual was promoted to a supervisory position which did not allow adequate time for attending requalification classes. Numerous quizzes and the annual requalification examination were not taken. On September 1, 1983 the Plant Manager signed a memo to the Training Coordinator instructing that no licensed duties were to be performed by the individual until deficiencies in the Requalification Training Program had been corrected. The individual was verbally notified of the restriction and did not perform any licensed activities.
- A Personal Qualification Statement (NRC Form 398) was prepared and in doing so it was noted that the individual did not meet the requalification requirements. The form included an entry noting that he had participated in the requalification program "from 1/81 to present".
- Based on the known training deficiencies the individual was instructed not to sign his license application.
- The license renewal forms were then sent to the Training Coordinator for his signature. He noted that the individual's unsigned form was in the stack and gave instructions to set it aside after he had signed it. The Training Coordinator was aware of the training status of the individual when he signed the application on December 8, 1983.
- The applications were forwarded to the Plant Manager when, although he questioned the applicant having not signed, he also signed the application on December 10, 1983.
- The applications were forwarded to the Corporate Office and the Vice President Nuclear Engineering signed the application on December 16, 1983. The applications were sent to the NRC on December 20, 1983 and received on December 23, 1983.
- During the processing it was noted that the applicant had not signed his application and it was therefore returned to him with a letter of explanation requesting his signature on line 17a of NRC Form 398. This letter of explanation had cc: for Plant Manager and Training Supervisor.
- When the applicant received the above request from the NRC and his license application with signatures from training, management and corporate levels in place, his opinion was the requirements had been met with the existing deficiencies. He signed the application on January 5, 1984 and sent it back to the NRC.
- Upon receiving the signed application, the NRC issued a license renewal based on the information represented in the application. A copy of the renewal notification effective February 11, 1984, was received by the plant January 13, 1984.
- Due to job assignment the individual continued not performing any licensed activities.

- On February 24, 1984 IE Information Notice 84-11: Training Program Deficiencies was issued and included discussion of facilities who failed to fully implement licensed operator training and qualification programs. This Notice was interpreted by the licensee to be concerned only with initial qualification, not requalification, as evidenced by internal correspondence.
- On May 3, 1984 licensee management was notified of discovery of the license renewal with existing requalification training program deficiencies. This discovery was made during routine review of STA training status.
- The licensee is investigating to ensure that no further requalification program discrepancies exist for licensed individuals.

The inspector provided Region III with the above information for review and determination of a noncompliance. This is an unresolved item until Region III review is completed (315/84-10-01; 316/84-11-01).

No items of noncompliance or deviations were identified.

8. IE Bulletins

For the IE Bulletins listed below the inspector verified that the Bulletin was received by licensee management and reviewed for its applicability to the facility. If the Bulletin was applicable the inspector verified that the written response was within the time period stated in the Bulletin, that the written response included the information required to be reported, that the written response included adequate corrective action commitments based on information presented in the Bulletin and the licensee's response, that licensee management forwarded copies of the written response to the appropriate onsite management representatives, that information discussed in the licensee's written response was accurate, and that corrective action taken by the licensee was as described in the written response.

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| (Closed) IE Bulletin No. 83-08 | Electrical Circuit Breakers With An Undervoltage Trip Feature Used In Safety Related Applications Other Than The Reactor Trip System |
| (Closed) IE Bulletin No. 83-05 | ASME Nuclear Code Pumps and Spare Parts Manufactured by the Hayward Tyler Pump Company (Reference Inspection Report 315/84-02; 316/84-02(DPRP) Paragraph 7). The licensee's response of May 7, 1984, concluded there are no Hayward Tyler Pumps at the Donald C. Cook Plant nor is Hayward Tyler Pump Company listed on the Donald C. Cook qualified suppliers' lists. |

No items of noncompliance or deviations were identified.

9. Management Meeting - Regulatory Performance Improvement Program (RPIP)

A management meeting was held on May 3, 1984, at the Donald C. Cook plant site to update the status of the RPIP dated February 23, 1984 (AEP:NRC:0625F). The licensee provided a revised schedule for item C-1 "Drawing and Labeling Accuracy". The schedule was revised to maximize walkdowns during the Unit 2 refueling outage. In addition, Item C-6 "Action Item Commitment List" was revised to reflect a new date to issue a draft revision to General Procedure 2.2 "Operation, Control, Maintenance of the AEPSC Commitment List"; the final revision date is unchanged. The licensee stated that the remaining nine items contained in Appendix C are on schedule as of April 15, 1984.

No items of noncompliance or deviation were identified.

10. Unresolved Items

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable items, items of noncompliance, or deviations. Unresolved items are discussed in Paragraph 7.

11. Exit Interview

The inspector met with the licensee representatives (denoted in Paragraph 1) throughout the inspection and at the conclusion of the inspection on June 18, 1984 and summarized the scope and findings of the inspection.