50-341

March 3, 1992

NOTE TO: James L. Blaha, Assistant for Operations
Office of the Executive Director
for Operations

FROM: James G. Partlow, Associate Director
for Projects
Office of Nuclear Reactor Regulation

SUBJECT: UPDATE STATUS OF REACTORS FOR MARCH 12, 1992
BEVILL HEARING

Enclosed is the update package for the requested plants per EDO Green Ticket 0007506. Only the following plants had a change from your most current information:

Fermi 2
Fort St. Vrain
Hope Creek Units 1 and 2
Midland
Oyster Creek
Rancho Seco
Salem Units 1 and 2
Sequoyah Units 1 and 2
Shoreham
Vogtle Units 1 and 2
Watts Bar Units 1 and 2
Yankee Rowe

Original Signed by:

James G. Partlow, Associate Director
for Projects
Office of Nuclear Reactor Regulation

Enclosure:
Update Package

cc: J. Sniezek
H. Thompson
T. Murley
F. Miraglia

CONTACT:
Ron Villafranco, NRR
504-1201

DISTRIBUTION:

PRAS R/F	R. Wessman	E. Tana (G.T. 0007506)	Central Files
PMSB R/F	R. Villafranco	T. Murley/F. Miraglia	NRC PDR
J. Partlow	P. Krayer	W. Russell	EDO 7506
F. Gillespie	E. Poteat	D. Crutchfield	J. Sniezek
OGC	OPA	OCA	H. Thompson
			J. Taylor

*See Previous Concurrence

PRAS:PMSB	PRAS:PMSB	C:PMSB	D:PMAS	ADPR:NRR
EPoteat*	RVillafranco*	RWessman*	FGillespie*	JPartlow*
03/02/92	03/02/92	03/02/92	03/02/92	03/03/92

DOC: A: BEVILL.HEA

FERMI-2

Reactor Type: BWR Vendor: GE Size: 1093 MWe

o CURRENT OPERATING STATUS

- Plant currently operating at power.

o CURRENT ISSUES

- Power uprate evaluation ongoing. Fermi-2 is the lead plant.
- Licensing backlog is low, with no items greater than two years in age.
- Licensee is working with Chem-Nuclear of Illinois to resolve a low level waste storage issue.

o RECENT LICENSEE PERFORMANCE

- Plant operations have been good with continuing improvement in all areas. Operator performance in abnormal situations is good and day-to-day activities are improving.
- Historically, inadequate and slow corrective actions to inspection findings.
- During the Main Transformer #2A replacement in the December 1991 outage, a mobile crane contacted one of the 120-kV transmission lines with no plant damage or personnel injuries occurring.
- Licensee continues to have difficulties in implementing changes to their Technical Specifications.
- Last SALP evaluation, May 1991, resulted in:

  "1" in Security and Radiological Controls; and,

  "2" for Emergency Preparedness, Operations, Engineering and Technical Support, Maintenance and Surveillance, and Safety Assessment and Quality Verification.

CONTACT:
T. Colburn
504-1341

February 1992

FORT ST. VRAIN

Reactor Type: High Temperature Gas Vendor: General Atomics Size: 330 MWe

CURRENT OPERATING STATUS

- Permanently shut down on August 18, 1989.
- Over 1/2 of core is defueled.
- A Possession Only License was issued May 21, 1991.
- Licensee is preparing to decommission the facility and is disposing of some unneeded components.
- Licensee plans to repower the facility by installing a gas-fired boiler to drive the existing turbine-generator.

CURRENT ISSUES

- Licensee has contract with DOE to ship spent fuel to DOE-Idaho. However, Governor Andrus of Idaho notified the licensee that Idaho will not allow the shipment. Idaho is in court challenging shipment of fuel to DOE-Idaho as a violation of Federal law. Three spent fuel shipments were made to DOE-Idaho before a Court Order was issued which prevented further shipment.
- The licensee has completed and is now placing spent fuel in an Independent Spent Fuel Storage Facility Installation (ISFSFI) which can store the fuel until a Federal Repository is available. Of 247 total shipments (liner plus spent fuel):
    - 85 shipments now in ISFSI
    - 2 shipments in transit to ISFSI
    - 65 shipments in spent fuel storage wells
    - 3 shipments received at DOE-Idaho
    - 92 shipments remain in core
- NRR and NMSS are reviewing the Decommissioning Plan which proposes immediate dismantlement (DECON).

RECENT LICENSEE PERFORMANCE

- Headquarters and Regional staff are monitoring the licensee's efforts to plan and carry out decommissioning and the transfer of spent fuel to the ISFSFI.

CONTACT:
Peter Erickson, NRR/PDNP
504-1101

March 2, 1992

HOPE CREEK GENERATING STATION

- Electric, 1067 MWe (Net)

- CURRENT OPERATING STATUS

  Operating in its fourth cycle at or near 100% power. A mini-outage is scheduled for March 7 thru 17, 1992.

- KEY ISSUES

  Lead plant for BWROG initiative to increase allowable MSIV leakage rate and eliminate leakage control system. Periodic meetings are being held with BWROG and PSE&G representatives.

  Installation of hardened wetwell vent. Licensee has agreed to modify currently installed vent to meet the design criteria developed by the BWROG and approved by NRC

  Based on the staff's review of Hope Creek's Station Blackout (SBO) Rule response, it was determined that Hope Creek does not currently conform to the SBO rule. This finding was based on the staff's inability to independently verify the conclusions reached in Hope Creek's SBO submittals. The staff requested that PSE&G reevaluate the areas of non-conformance and submit a revised response to the SBO rule. The staff is currently reviewing PSE&G's latest response.

- RECENT LICENSEE PERFORMANCE

  Operation reflects conservative and safety conscious attitude. Good management support and oversight evident. In the SALP, which ended 7/31/90, six Category 1's and one Category 2 improving. The most recent SALP period ended 12/28/91. The report is being prepared.

  Hope Creek experienced four automatic scrams during this SALP period. However, the plant has operated continuously since May 12, 1991.

CONTACT:
Stephen Dembek
504-1422

MARCH 1992

MIDLAND UNITS 1 AND 2

PLANT STATUS

- By letter dated July 1, 1986, Consumers Power Company informed the NRC that it planned to convert Unit 1 to a gas-fired facility and to abandon Unit 2 in-place.

- On October 15 and 16, 1986, the NRC conducted an inspection of the site and verified that Consumers Power Company had satisfactorily restored the site to an environmentally stable condition and that Units 1 and 2 were not capable of being operated as a nuclear facility.

- NRC issued the Order revoking the Construction Permits on December 18, 1986.

March 1992

OYSTER CREEK

- o BWR - GE - 650 MWe

- o CURRENT OPERATING STATUS

  - Plant is presently operating at 100% power. Next refueling outage scheduled for spring of 1993.

- o CURRENT ISSUES

  - Extension of operating license to 40 years from the date of issuance of the full power license.

  - Isolation Condenser Steam Line Isolation Valve issue. Two Isolation Valves outside containment. Seismic response spectra as it relates to IEB79-02 and 79-14.

  - Drywell Corrosion.

  - IGSCC Inspection Program.

  - Containment Combustible Gas Control Issue.

  - Wind Loading of Tornado Missiles.

  - Simulator certification and control room operator training. The staff is currently evaluating licensee's request for an exemption to extend the simulator certification date to December 31, 1992.

  - Motor operated valve program - GL 89-10, "Safety Related Motor-Operated Valve Testing and Surveillance" including Supplement 3.

- o RECENT LICENSEE PERFORMANCE

  - During the last SALP period which ended April 14, 1991, GPUN received a Category 1 in Emergency Preparedness. In all other functional areas GPUN received a Category 2. There was a significant improvement in Radiological Controls during the last SALP period.

  - Engineering support was good and improving. However, despite this improvement, ineffective resolution of some technical issues has resulted in a slow rate of equipment performance improvements and a continuing high number of day-to-day plant equipment problems.

Contact: Alexander W. Dromerick
504-3473

February 28, 1992

RANCHO SECO

Reactor Type: PWR Vendor: B&W Size: 918 MWe

CURRENT OPERATING STATUS

- Plant is shut down due to negative outcome of referendum vote.
- All fuel removed from vessel and stored in the spent fuel pool.
- Licensee is preparing for decommissioning.

CURRENT ISSUES

- Active intervention in decommissioning plan by Environmental Conservation Organization.
- On February 6, 1992, the Commission authorized the staff to issue the Possession Only License amendment after addressing certain contentions made by the intervenor. The staff plans to issue this amendment in March 1992.
- The staff plans to issue the Defueled Technical Specifications by the end of March 1992.
- Licensing of Independent Spent Fuel Storage Installation (NMSS Lead).
- Decommissioning Plan was submitted May 1991. The staff expects to complete its review by mid June 1992.
- Price-Anderson insurance exemption under staff review.

RECENT LICENSEE PERFORMANCE

- SMUD's performance has been adequate to support plant operations in the defueled condition.

CONTACT:
Stewart Brown, NRR/PDNP
504-3143

March 2, 1992

SALEM NUCLEAR GENERATING STATION
UNITS 1 AND 2

- PWR, Westinghouse, 1106 MWe (Net)

- CURRENT OPERATING STATUS

  Unit 1 is currently operating at 100 percent power. It is scheduled to begin its tenth refueling outage on April 4, 1992.

  Unit 2 is currently in its sixth refueling outage. The outage started early because of the turbine overspeed event that occurred on November 9, 1991. Restart has been scheduled for April, 1992.

- CURRENT ISSUES

  Station Blackout: The staff has completed their review and has found that Salem does not comply with the rule. PSE&G had proposed to use a 4-hour coping period and the staff's analysis found that an 8-hour coping period is required. PSE&G has been requested to respond within 60-days.

  The material condition of the plants have shown improvement during the last year, but additional effort is required and being actively pursued by PSE&G.

  The Radiation Monitoring System (RMS) has caused numerous ESF actuation and reportable events. Antiquated equipment and poor spare parts availability is the cause of these events. Short term corrective action include electronics upgrade and new power supplies. The ultimate solution is a complete system upgrade schedule for completion in 1994.

  An application to rerack the spent fuel pools at Salem 1 and 2 is anticipated in 1992. Without reracking, the existing spent fuel pools will reach their capacity in 1998 and 2002 for Salem 1 and 2, respectively. The reracking will provide storage capability until 2007 and 2011 for Salem 1 and 2, respectively.

- RECENT LICENSEE PERFORMANCE

  On November 9, 1991, during testing of the main turbine trips at Salem 2, an overspeed event occurred, damaging the three low pressure turbines, the main generator and the exciter. The reactor tripped as designed and was subsequently placed in cold shutdown. Region dispatched an AIT to investigate the event. The AIT found equipment failures and personnel errors were main contributors to the event.

  Salem 1 experienced a reactor trip on June 16, 1991, when lightning struck the phase B main power transformer.

  The SALP period ended on December 28, 1991. Evaluations are in process. For the SALP period ending 7/31/90, the ratings were mostly 2's, with 1's in Emergency Preparedness and Security/Safeguards.

CONTACT:
J. Stone
504-1419

MARCH 1992

SEQUOYAH

Plant Type: PWR
Vendor: Westinghouse
Size: 1148 MWe

- In August 1985, the units were voluntarily shut down to address environmental qualification issues before the regulatory deadline of November 30, 1985. Unit 2 remained down for 33 months and Unit 1 for 39 months.
- On September 17, 1985, following low SALP ratings, the staff formally requested TVA to address Browns Ferry and Sequoyah programmatic and hardware issues and TVA management weaknesses.
- On November 1, 1985, TVA submitted its Corporate Nuclear Performance Plan to address the programmatic, hardware, and management issues.
- The staff issued its Safety Evaluations for the restart of Sequoyah, on July 18, 1987 (Corporate), May 18, 1988 (Unit 2) and February 3, 1989 (Unit 1).
- Both units had TVA record runs for power operation following the restarts from the extended outage.
- SALP ratings for 1988 and present performance indicate that the units are average performers among Region II facilities.
- Unit 1 completed Cycle 5 refueling outage with plant startup on November 19, 1991.
- Unit 2 Cycle 5 outage scheduled to start March 13, 1992.
- Licensee has discovered that many discrepancies exist related to Fire Protection and has developed a comprehensive action plan.
- Eagle-21 Reactor Protection System installed during Cycle 4 outage (microprocessor-based system) is performing extremely well.
- Organizational and management changes made two years ago seem to have resulted in marked improvements, and a trend toward further improvement.
- A new annunciator system has been installed in the control room for Unit 1 during the Cycle 5 outage. Same system will be installed for Unit 2 during the outage.
- Plans are underway to rerack the spent fuel pool starting in 1994 to increase storage capacity.

SHOREHAM

Reactor Type: BWR Vendor: GE Size: 819 MWe

CURRENT OPERATING STATUS

- Plant is shut down due to a settlement agreement between NY State and Long Island Lighting Company (LILCO).
- All fuel removed from vessel and stored in the spent fuel pool.
- License was transferred to the Long Island Power Authority (LIPA) on February 29, 1992. LIPA will decommission the facility.

CURRENT ISSUES

- Active intervention by Shoreham Wading River School District (SWRSP) and Scientists and Engineers for Secure Energy (SE2).
- Possession Only License issued by NRC on June 14, 1991.
- Transfer of license from LILCO to LIPA allowed LILCO to begin a 10-year phased reduction in local property taxes.
- Decommissioning Plan submitted December 1990. Technical review conducted by NMSS was completed on January 22, 1992. Both intervenors (SWRSD and SE2) requested a hearing on the Decommissioning Order. Hearing request is before the Commission.
- Price-Anderson insurance exemption under staff review.

RECENT LICENSEE PERFORMANCE

- Before transfer of license to LIPA, NRC concluded that LIPA, with substantial support from LILCO, was qualified to safely maintain the facility in the defueled condition until decommissioning is completed.

CONTACT:
Stewart Brown, NRR/PDNP
504-3143

March 2, 1992

VOGTLE UNITS 1 AND 2

* PLANT DATA

  - Westinghouse, PWR, 4 Loop, 1157 MWe
  - Docket No. 50-424, 50-425

* CURRENT OPERATING STATUS

  - Unit 1 in currently operating at 100% power and has been at that level since completing its last refueling outage 94 days ago.

  - Unit 2 is currently at 88% power due to fuel depletion and is coasting down for its next refueling outage which begins March 13, 1992. Unit 2 has been on line 296 days, which exceeds Vogtle's prior record of 270 days by Unit 1.

* CURRENT ISSUES

  - During a special team inspection in August 1990, the inspectors concluded that licensee officials provided inaccurate, unsworn, oral statements concerning topics well within their responsibilities. During its exit meeting and in its inspection report January 11, 1991, and November 1, 1991, the team cited four examples regarding (1) containment isolation valves, (2) snubber reductions, (3) Shift Superintendent accountability, and (4) Technical Specification 3.0.3 actions. After the exit meeting, the licensee initiated its own review of these examples. An enforcement conference was held December 16, 1991, and summarized by report of January 8, 1992. The licensee discussed its own review of the examples which reached different conclusions from those in the inspection report. The NRC decided not to issue an enforcement action for the violations of 10 CFR 50.9.

  - An NRC investigation associated with allegations concluded that, in October 1988, chemical injection valves required by Technical Specifications to be closed and locked during shutdown were intentionally opened to facilitate operations. The NRC issued Demands for Information to Georgia Power Company and three operators on June 3, 1991. Written responses were received August 28, 1991. After an enforcement conference held on September 19, 1991, NRC issued a Level III violation and a proposed civil penalty of

CONTACT:
Darl Hood
504-3049

February 28, 1992

$100,000 on December 31, 1991. The licensee denied the violation in their response dated January 30, 1992. The NRC is reviewing the Georgia Power letter and is drafting a response. The NRC sent letters to the three individuals stating that no enforcement action would be taken and indicating that their performance in this matter had not met expectations. Mr. R. P. McDonald, Executive V.P., wrote a letter to Chairman Selin on February 3, 1992, stating that they believed the enforcement action is an example of inappropriate regulatory action. The Office of Enforcement has provided the Chairman with a draft response back to Mr. McDonald.

- On October 26, 1991, while Vogtle Unit 1 was shutdown for refueling, shutdown cooling capability was lost about 20 minutes during draindown to replace the vessel head. The loss was due to a common mode failure of refueling cavity level instruments. All level instruments provided inaccurate indications because they were vented to the pressurizer head and a negative pressure was drawn on the pressurizer when Health Physics connected a fan and HEPA filter to collect potential releases in conjunction with an activity which caused a decrease in water level. The false readings allowed cavity levels at or near mid-loop levels as evidenced by cavitation of the RHR pump. Upon discovery of the problem, appropriate water level was re-established using the other RHR pump. The NRR dispatched an augmented inspection team (AIT) to review the incident. Following issuance of the AIT's report, an enforcement panel was held to consider procedure and reporting deficiencies and inadequate system modification controls. These were determined to be Severity Level IV and no escalated enforcement was imposed.

- On July 3, 1991, the licensee discovered that its calibration procedure did not include a hydrostatic head correction factor for the static column of water in the instrument tubing used to sense pressurizer pressure. The effect of this oversight was that the small break LOCA, as analyzed in the FSAR, exceeded 10 CFR 50.46 criteria. The NRC staff identified three violations in Inspection Report 50-424,425/91-15. At an enforcement conference August 19, 1991, the licensee discussed Westinghouse's subsequent reevaluation of the small break LOCA analysis using more realistic but conservative assumptions regarding the timing of adverse conditions in the containment. The new

analysis demonstrated compliance with 10 CFR 50.46. Subsequently, a Severity Level IV violation was cited.

* RECENT LICENSEE PERFORMANCE

- For the most recent SALP period ending September 28, 1991, Vogtle was rated 1 in Radiological Controls and 2 in all other areas. Improving trends were noted in Engineering/Technical Support and in Security/Safeguards.

WATTS BAR

Plant Type: PWR
Vendor: Westinghouse
Size: 1177 MWe

- On February 20, 1985, TVA submitted its certification that the design, construction, testing, and preparation for operation of Watts Bar Unit 1 were essentially completed.

- In April 1985, the NRC became aware of numerous TVA employee concerns which called into question the readiness of Watts Bar for licensing, and the general quality of its design and construction as well as the ability of TVA to manage the nuclear power program.

- On May 30, 1985, the NRC requested TVA's basis for its determination that Watts Bar Unit 1 met all licensing commitments.

- In the fall of 1985, TVA established an employee concerns program to interview all nuclear employees at Watts Bar to identify all concerns.

- On September 17, 1985, NRC issued a formal request for information on actions TVA was taking to resolve NRC's concerns with its nuclear power program.

- On April 11, 1986, TVA rescinded its certification that Watts Bar, Unit 1 was ready for fuel loading, thereby putting the plant back to construction status.

- In November 1987, TVA established the Watts Bar Program Team (WBPT) to perform an integrated systematic evaluation of plant design and construction and identify necessary corrective actions. This effort included a Vertical Slice Review by Sargent and Lundy.

- In May 1989, TVA submitted the Watts Bar Nuclear Performance Plan (WBNPP), which contained the site-specific response to the NRC September 17, 1985 letter. Many discovery and corrective actions had already taken place. These were evaluated and incorporated into the WBNPP.

- Staff review of TVA's plans to resolve identified problems contained in the Corrective Action Programs (CAPs) in the site-specific nuclear performance plan is mostly complete. The staff is monitoring the implementation of these programs. Examples of issues being addressed by these programs are cable installation, seismic design, design baseline and verification and QA/QC records.

- Unit 2 is in layup with no current licensing schedule.

- In late December 1990 TVA issued a stop work order for all construction work on site due to deficiencies in work plans. TVA subsequently laid off the entire construction workforce. TVA has hired Ebasco Services to provide the labor, project engineering, and supervision required to complete Watts Bar. Construction work restarted on November 22, 1991, with NRC approval.

- There is no current estimated startup date for Watts Bar, Unit 1 although the staff estimates that licensing will be no earlier than early 1994.

YANKEE ROWE

Westinghouse PWR - 175 MWe

1. CURRENT OPERATING STATUS

The licensee voluntarily shut the plant down on October 1, 1991, after issuance of our September 30, 1991, Memorandum to the Commission recommending such a shutdown. By letter dated October 25, 1991 the licensee further committed not to continue to seek a restart of the plant prior to the scheduled April 1992 outage. On February 26, 1992 the licensee announced that the plant is to be permanently shutdown. The licensee cited economics and regulatory uncertainty as the key factors for its decision. The 570 employees of Yankee Atomic were assured that their employment would continue at least until the end of 1992. The NRC was informed that the licensee will seek a Possession Only License, in the near future, as the first step towards decommissioning.

Contact: M. Fairtile 504-1442

February 27, 1992