



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
101 MARIETTA STREET, N.W.  
ATLANTA, GEORGIA 30303

Report Nos.: 50-321/84-17 and 50-366/84-17

Licensee: Georgia Power Company  
P. O. Box 4545  
Atlanta, GA 30302

Docket Nos.: 50-321 and 50-366

License Nos.: DPR-57 and NPF-5

Facility Name: Hatch 1 and 2

Inspection Date: May 29 - June 1, 1984

Inspection at Hatch site near Baxley, Georgia

Inspector:

*S. P. Burris*  
S. P. Burris

*12 June 84*  
Date Signed

Approved by:

*F. Jape*  
F. Jape, Section Chief  
Engineering Branch  
Division of Reactor Safety

*12 June 84*  
Date Signed

SUMMARY

Scope: This routine, unannounced inspection involved 24 inspector-hours on site in the areas of Average Power Range Monitor System, Rod Block Monitor System and Technical Specification (ARTS) program modifications witnessing, review of status of Bulletin 80-25, Target Rock Safety Relief Valve, followup on item 321/82-38-01, and plant tour.

Results: No violations or deviations were identified.

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## REPORT DETAILS

### 1. Persons Contacted

#### Licensee Employees

- \*H. Nix, Plant Manager
- \*C. Belflower, QA Site Manager
- \*S. Baxley, Superintendent of Operations
- \*J. Betsill, Acting Superintendent of Engineering Services
- \*T. Elton, Acting Superintendent of Regulatory Compliance
- \*R. Tracy, Plant Engineer (Regulatory Compliance)
- W. Thigpen, Regulatory Compliance Supervisor
- W. Stovill, Shift Supervisor, Operations
- J. Ponder, Regulatory Compliance Engineer
- C. Goodman, Regulatory Compliance Engineer
- W. Kirkley, H.P. and Chemistry Supervisor
- D. Willyard, Systems Engineer
- D. Townley, Fuel Cycle Engineer

Other licensee employees contact included two technicians, three operators, three security force members, and three office personnel.

#### Other Organization

- E. Thorsen, General Electric C&I Services Engineer
- A. Hekmati, General Electric C&I Services Engineer

#### NRC Resident Inspector

P. Holmes-Ray  
Resident Inspector

\*Attended exit interview

### 2. Exit Interview

The inspection scope and findings were summarized on June 1, 1984, with those persons indicated in paragraph 1 above. The licensee acknowledged the findings without significant comment.

### 3. Licensee Action on Previous Enforcement Matters

The inspector obtained a copy of the Local Leak Rate Test procedure and other documents in reference to violation 82-38-01 for review in the regional office.

### 4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Independent Inspection Effort, Units 1 and 2 (92706)

The inspector toured portions of the Units 1 and 2 reactor building, control building and control complex, turbine building, and switchyard to observe on-going activities for compliance with NRC requirements and licensee commitments.

No violations or deviations were identified in the areas inspected.

6. ARTS Program Modification Witnessing (97705)

The inspector witnessed portions of the wiring and card modifications performed by GE personnel on Rod Block Monitor Cabinets A & B. These changes were performed under the outline listed in General Electric Boiling Water Reactor Licensing Report: Average Power Range Monitor, Rod Block Monitor and Technical Specification Improvement (ARTS) Program for Edwin I. Hatch Nuclear Plant, Units 1 and 2, NEDC-30474-P, Class III. The inspector observed overall personnel performance and verified that an approved change procedure was available and in use, equipment used was properly installed and changes to the circuitry were being correctly removed, replaced, or installed as required.

During modification witnessing, the inspector noted several minor discrepancies, which were identified to plant personnel. The first item was placement of "Dymo" tape with the words "card pulled" printed on it on the front exterior door for rod block monitor Cabinet B. The inspector questioned the shift supervisor as to what card was pulled, when it was pulled, where it was pulled, and how was this pulled card officially identified to plant recordkeeping.

The shift supervisor informed the inspector that he thought the referenced card was in the annunciator circuit. Plant management committed to resolving this problem by the next inspection tour.

Secondly, the consultant personnel involved with the rod block modifications noted that there appeared to be a problem with Average Power Range Monitor "C" meter in that when the selector switch was placed in "average" position, the needle would deflect to the high peg. The inspector questioned this action and was informed by plant personnel that they thought that the power supply for APRM "C" was faulty.

Both of these items will be monitored during future inspection tours.

No violations or deviations were identified in the areas inspected.

7. Review of Status of IE Bulletin 80-25 (Target Rock Safety/Relief Valves) and NUREG-0737, Item II.K.3.16 (92703)

The inspector reviewed the plant's corrective action as outlined in Hatch commitment response to problems identified with the Target Rock Safety/Relief Valves. Plant Hatch has met the commitments identified in

letter #NED-83-280 from Mr. J. T. Beckman Jr., Vice President Nuclear Generation to Region II NRC dated June 24, 1983, in that they have:

- a. Installed a high pressure alarm switch which will alarm in the control room for the pneumatic system.
- b. Installed a low pressure alarm which will annunciate in the control room for the seven ADS valves
- c. Installed a relief valve in drywell pneumatic system to prevent exceeding 125 psig
- d. Installed soft seat check valves to preclude leakage of the SRV accumulators

The modifications were completed under control of Design Change Request (DCR)-81-111 for Unit 1 and DCR-81-112 for Unit 2. The inspector's review indicated that although the modifications were complete as identified above, additional changes would be performed by:

- a. Technical Specification changes
- b. Piping System modifications to increase system reliability

Hatch personnel informed the inspector that DCR-81-112 would be completed during the current Unit 2 outage and DCR-81-111 is scheduled for completion the next scheduled outage for Unit 1. The inspector informed Hatch management that these items would be reviewed during later inspection tours.

No violations or deviations were identified in the areas inspected.