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Robert L. Mittl General Manager Nuclear Assurance and Regulation

July 9, 1984

Director (Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission 7920 Norfolk Avenue Bethesda, MD 20814

Attention: Mr. Albert Schwencer, Chief

Licensing Branch 2 Division of Licensing

Gentlemen:

HOPE CREEK GENERATING STATION

DOCKET NO. 50-354

CONTROL ROOM HABITABILITY - DOSE ASSESSMENT INFORMATION

On April 10, 1984, a meeting was held in the Bethesda, Maryland offices of the NRC to discuss control room habitability at the Hope Creek Generating Station. In DSER, Section 6.4, information was requested which would enable the staff to perform a control room X/O analysis using the Murphy-Campe methodology.

In response to the staff's request at the April 10, 1984, meeting, advance copies of new FSAR Table 6.4-5 and updated FSAR Figure 6.4-2 are attached for information in support of the staff's analysis. This information will be incorporated in an upcoming amendment to the HCGS FSAR.

We understand that, upon receipt of this information, the staff will perform a Murphy-Campe X/Q analysis to assess compliance with General Design Criteria 19.

Should you have any questions in this regard, please contact us.

Very truly yours,

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Attachment The Energy People A003

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C D. H. Wagner USNRC Licensing Project Manager

Mr. W. H. Bateman USNRC Senior Resident Inspector

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HCGS FSAR

TABLE 6.4-5

RADIOACTIVE RELEASE LOCATIONS RELATIVE TO CONTROL ROOM INTAKE

DISTANCE FROM CONTROL ROOM INTAKE TO RELEASE (ft) ELEVATION RECEPTOR/ RELEASE POINT OF (ft) (NORTH/SOUTH) (EAST/WEST) 0 ' 156'-9" 0 . Control Room Intake 1. FRVS Exhaust Vent(a) 301'-4" 111'-2"(S) 129'-0"(E) 2. South Plant Vent (b) 217'-0" 111'-2"(S) 303'-3"(E) 3. 217'-0" North Plant Vent 266'-1"(N) 306'-3"(E) 4. 111'-2"(S) 143'-9" 45'-0"(E) 5. Reactor Building Blowout Panel MS Tunnel North(c) 161'-11" 94'-2"(S) 226'-2"(E) 6. Blowout Panel (Emergency) 162'-0" 137'-5"(S) 224'-11"(E) MS Tunnel South 7. Blowout Panel (Emergency) 151'-2"(S) 274'-6"(E) 8. MS Tunnel Blowout 143'-6" Panel

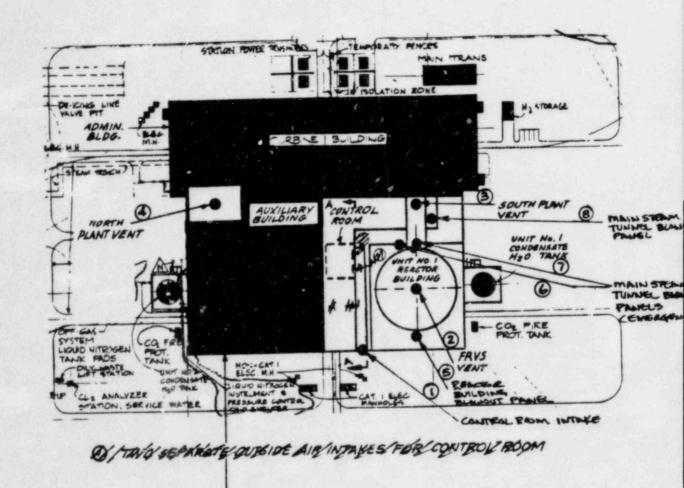
Design Basis Accident(s) associated with release locations (per Chapter 15.0 analyses):

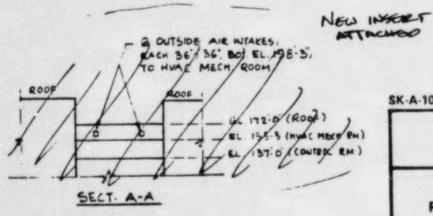
- (a) LOCA, Fuel Handling Accident
- (b) Waste Gas System Failure, Control Rod Drop, Instrument Line Break
- (c) Steam Line Break

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SK-A-1006, REV N :

MOPE CREEK GENERATING STATION FINAL SAFETY ANALYSIS REPORT

PLANT LAYOUT WITH RESPECT TO CONTROL ROOM INTAKE

FIGURE 6.4-2

