NARRATIVE REPORT
UNIT 1

June 2nd	2206	Load decrease from rated thermal power to 636 GMWe for weekly turbine test (HNP-1-1053).
June 3rd	0330	Returned to rated power; turbine testing completed.
June 4th	0104	Load dropped to 730 GMWe for daily turbine testing (HNP-1-1051).
June 4th	0121	Rx scram on turbine stop valve fast closure.
June 9th	1835	Reactor critical.
June 10th	0909	Turbine tied to line.
June 11th	2200	Load at approximately 640 GMWe.
June 12th	1852	Reducing power via recirc flow to approximately 475 GMWe for rod pattern adjustment.
June 14th	2300	Load at approximately 740 GMWe.
June 14th	2352	Load reduction via recirc flow to approximately 665 GMWe due to condensate system limitations.
June 15th	0014	Load reduction stopped at 664 GMWe. Investigating problems with high condensate demin system delta P.
June 15th	0052	Increasing load with recirc flow to 675 GMWe.
June 15th	2230	Load at 751 GMWe. 99% CMWt.
June 16th	2033	Reducing load to take 8th stage B feedwater heater out to be repaired.

IE24

NARRATIVE REPORT
UNIT 1

June 16th	2230	Load at 601 GMWe. 78.9% CMWt.
June 17th	0500	Load at approximately 550 GMWe.
June 17th	0925	Rapid loss of vacuum as bolts are removed from B feedwater heater.
June 17th	0950	Vacuum back to normal. 7th and 8th stage B feedwater heaters taken out of service and isolated to repair tube leak.
June 17th	1000	Load at approximately 310 GWMe.
June 18th	0100	Load maintained at approximately 350 GMWe.
June 19th	1325	7th and 8th stage B feedwater heater back in service. Increasing load via rod pulls and recirc flow.
June 20th	2310	Load reduction started from 748 GMWe to repair feedwater check valve leak.
June 21st	1034	Reactor manual scram per HNP-1-1020. Going to cold shutdown for feedwater check valve repair.
June 23rd	0609	Reactor critical.
June 24th	0155	Turbine tied to line.
June 27th	0800	Load at approximately 750 GMWe. Back to rated power.
June 29th	2007	Reducing load to approximately 625 GMWe for rod pattern adjustment.
June 30th	1430	Load at approximately 749 GMWe. Back to rated power.

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR June 1984

NUMBER	DATE COMPLETED	DESCRIPTION
84-1534	04-04-84	Replaced existing core spray discharge line level switch (Ref. DCR 84-100).
84-2831	05-31-84	Cable spread & computer room rebuild CO_2 discharge valve 1Z43-F008 (Ref. DCR 84-093).
84-2694	05-22-84	Repaired 1B PSW pump cooling water line leak. (Ref. DCR 80-183).
82-7863	01-21-83	Fabricated & installed supports & relocated Vac. system piping & supports. (Ref. DCR 80-348).
84-2659	05-11-84	Replaced RWCU isolation relay located on 1H11-P609. (Ref. DCR 82-171).
84-2825	05-23-84	Diesel building rebuild CO_2 discharge valve 1X43-F010A. (Ref. DCR 84-093).
84-2827	05-23-84	Diesel building rebuild CO_2 discharge valve 1X43-F010B. (Ref. DCR 84-093).
84-2828	05-23-84	Diesel building rebuild CO_2 discharge valve 1X43-F010C. (Ref. DCR 84-093).
84-2829	05-23-84	Diesel building rebuild CO_2 discharge valve 1X43-F010D. (Ref. DCR 84-093).
84-523	02-14-84	Replaced kaowool in cable tray RAAO-01. (Ref. DCR 81-174).

OPERATING DATA REPORT

DOCKET NO. 50-321
DATE 07-10-84
COMPLETED BY: P.J. North
TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 1
2. Reporting Period: 06-84
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 809.3
5. Design Electrical Rating (Net MWe): 777.3
6. Maximum Dependable Capacity (Gross MWe): 801.2
7. Maximum Dependable Capacity (Net MWe): 752.2
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe)
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	720	4367	74494
12. Number of Hours Reactor was Critical	539.2	3517.8	53042.9
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	504.9	3387.3	49780.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1056456	7674004	104892032
17. Gross Electrical Energy Generated (MWH)	315410	2424130	33882300
18. Net Electrical Energy Generated (MWH)	296661	2306478	32165841
19. Unit Service Factor	70.1	77.6	66.8
20. Unit Availability Factor	70.1	77.6	66.8
21. Unit Capacity Factor (Using MDC Net)	54.8	70.2	57.4
22. Unit Capacity Factor (Using DER Net)	53.0	67.9	55.6
23. Unit Forced Outage Rate	29.9	20.9	18.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    Refueling 8-1-84 10 weeks

25. If Shutdown at End of Report Period, Estimated Date of Startup:
26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321
DATE: 07-10-84
COMPLETED BY: P.J. North
TELEPHONE (912) 367-7851

MONTH 06-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	730	17	366
2	722	18	339
3	726	19	431
4	23	20	678
5	-15	21	173
6	-15	22	-16
7	-17	23	-17
8	-16	24	307
9	-14	25	556
10	115	26	687
11	569	27	708
12	603	28	715
13	580	29	695
14	670	30	690
15	683	31	
16	706		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June

DOCKET NO. 50-321
UNIT NAME Hatch 1
DATE 7-10-84
COMPLETED BY P. J. North
TELEPHONE 912-367-7851

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
84-40	840601	F	720.0	A	5	NA	HA	TURBIN	13th stage buckets on low pressure turbine were damaged & out for the duration of June.
84-41	840602	S	5.4	B	5	NA	HA	TURBIN	Load reduction for turbine testing.
84-42	840604	S	.3	B	5	NA	HA	TURBIN	Load reduction for turbine testing.
84-43	840604	F	151.8	A	3	NA	HB	VALVEX	Rx scram on turbine stop valve fast closure during turbine testing.
84-44	840612	S	5.9	H	5	NA	RB	CONROD	Load reduction for rod pattern adjustment.
84-45	840614	F	1.0	A	5	NA	CH	PUMPXX	Low condensate booster pump suction & discharge pressure.
84-46	840616	F	64.9	A	5	NA	CH	HTEXCH	Load reduction to take 8th stage B feedwater heater out of service for repair of tube leak.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June

DOCKET NO. 50-321
UNIT NAME Hatch 1
DATE 7-10-84
COMPLETED BY P. J. North
TELEPHONE 912-367-7851

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
84-47	840620	F	74.1	A	2	NA	CH	VALVEX	Outage to repair feedwater check valve leak.
84-48	840629	S	1.7	H	5	NA	RB	CONROD	Load reduction for rod pattern adjustment.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

NARRATIVE REPORT
UNIT 2

June 1st	0000	Recirc Piping Outage still in progress.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR June 1984

NUMBER	DATE COMPLETED	DESCRIPTION
83-2440	07-08-83	Reinstalled SRV (Body, topworks, & solenoid). 2B21-F013C. (Ref. DCR 82-53 & 83-25).
83-2311	05-28-83	Main steam relief valve modified existing hanger 2B21-MSRV-R121. (Ref. DCR 82-76).
83-1397	05-27-83	RHR system modified existing hanger 2E11-RHR-R230. (Ref. DCR 82-76).
83-1131	05-27-83	RHR system installed pipe support 2E11-RHR-H719. (Ref. DCR 82-76).
84-1208	04-17-84	Repaired welds on kates type C2F16-12 3/4"-900# flow regulators per NCR 84-52. (Ref. DCR 79-96).
84-1074	06-15-84	Repaired containmen spray isolation valve. (Ref. DCR 84-170).
83-6042	02-08-84	Installed D.P. indicator, valves, tubing, & supports on standby diesel service water flow element. (Ref. DCR 81-145).
84-1342	05-14-84	Replaced solenoid valve on waste sludge pump "B" flush valve 2G11-F256B. (Ref. DCR 84-96).
84-1631	05-11-84	Repaired D/G 2C air start system. (Ref. DCR 80-98).
84-1682	05-11-84	Installed keepwarm modification in D/G 2C lube oil system. (Ref. DCR 82-259).
83-772	03-26-84	Installed pipe fittings & isolation valves 2P41-F826 & 2P41-F829 on HPCI room air conditioning.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR June 1984

NUMBER	DATE COMPLETED	DESCRIPTION
84-1631	04-23-84	Machine recirc pump shaft (Ref. DCR 84-136).
84-1648	04-18-84	Machine spare recirc pump shaft. (Ref. DCR 84-136).
84-1649	04-28-84	Machine recirc pump shaft. (Ref. DCR 84-136).
84-293	05-21-84	Install hose for temporary fire protection to dust collector #3. To be removed at end of outage. (Ref. DCR 83-173).
83-6122	01-23-84	Install electrical components in cabinets 2H21-P050 & 2H21-P051. (Ref. DCR 81-174).
83-3273	06-22-83	Installed conduit supports CS-05-173 & CS-05-174. (Ref. DCR 82-76).
83-2641	06-04-83	Revised reactor building nitrogen inerting system isolation valve actuator orientation. (Ref. DCR 83-51).
83-2579	06-08-83	Overlayed manifold to endcap weld 2B31-IRC-22AM-4. (Ref. DCR 83-63).
83-2447	07-08-83	Reinstalled SRV (body, topworks & solenoid). 2B21-F013G (Ref. DCR 82-53 & 83-25).
83-2446	07-08-83	Reinstalled SRV (Body, topworks, & solenoid). 2B21-F013F. (Ref. DCR 82-53 & 83-25).
83-2445	07-08-83	Reinstalled SRV (Body, topworks, & solenoid). 2B21-F013E. (Ref. DCR 82-53 & 83-25).

OPERATING DATA REPORT

DOCKET NO. 50-366
DATE 07-10-84
COMPLETED BY: P. J. North
TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 06-84
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 817.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 803.9
7. Maximum Dependable Capacity (Net MWe): 747.9
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	720	4387	42120
12. Number of Hours Reactor was Critical	0.0	308.2	27379.4
13. Reactor Reserve Shutdown Hours	0.0	0	0
14. Hours Generator On-Line	0.0	308.2	26096.1
15. Unit Reserve Shutdown Hours	0.0	0	0
16. Gross Thermal Energy Generated (MWH)	0	726912	55943167
17. Gross Electrical Energy Generated (MWH)	0	242640	18414420
18. Net Electrical Energy Generated (MWH)	-2495	219274	17508381
19. Unit Service Factor	0.0	7.1	62.0
20. Unit Availability Factor	0.0	7.1	62.0
21. Unit Capacity Factor (Using MDC Net)	-0.5	6.7	55.6
22. Unit Capacity Factor (Using DER Net)	-0.4	6.4	53.0
23. Unit Forced Outage Rate	0.0	0.0	13.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shutdown at End of Report Period, Estimated Date of Startup:
26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-366
DATE: 07-10-84
COMPLETED BY: P. J. North
TELEPHONE (912) 367-7851

MONTH 06-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-1	17	-3
2	-1	18	-4
3	-2	19	-4
4	-3	20	-4
5	-2	21	-4
6	-2	22	-4
7	-3	23	-3
8	-3	24	-4
9	-3	25	-4
10	-4	26	-4
11	-4	27	-4
12	-4	28	-5
13	-4	29	-5
14	-3	30	-6
15	-4	31	
16	-4		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-366
UNIT NAME Hatch 2
DATE 7-10-84
COMPLETED BY P.J. North
TELEPHONE 912-367-7851

REPORT MONTH June

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
84-5	840601	S	720.0	H	2	NA	CB	PIPEXX	Recirc pipe replacement outage.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444

Georgia Power

Edwin I. Hatch Nuclear Plant

July 9, 1984
GM-84-593

PLANT E. I. HATCH
NRC Monthly Operating Report

Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC Monthly Operating Report for Hatch Unit 1, Docket #50-321, and for Hatch Unit 2, Docket #50-366.

H. C. Nix
General Manager

JAB/sw

IE24
1/1