

Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111

MEMBER OF THE
General  Public Utilities Corporation
SYSTEM

June 24, 1975



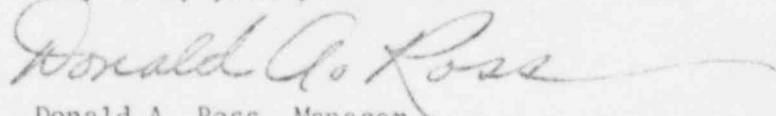
Mr. A. Giambusso
Director, Division of Reactor Licensing
Office of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Giambusso:

Subject: Oyster Creek Station
Docket No. 50-219
Abnormal Occurrence Report No. 50-219/75-16

The purpose of this letter is to forward to you the attached abnormal occurrence report in compliance with paragraph 6.6.2.a of the Technical Specifications.

Very truly yours,


Donald A. Ross, Manager
Generating Stations-Nuclear

pk


Enclosures

cc: Mr. J. P. O'Reilly, Director
Office of Inspection and Enforcement, Region 1

50-219
inquiry

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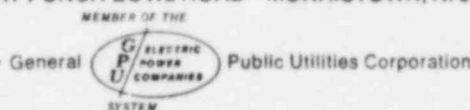
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OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Abnormal Occurrence
Report No. 50-219/75-16

Report Date

June 24, 1975

Occurrence Date

June 14, 1975

Identification of Occurrence

Violation of the Technical Specifications, paragraph 2.3.4, Electromatic Relief Valve Pressure Switches, 1A83B and 1A83E, were found to trip at pressures in excess of the maximum allowable value of 1070 psig. This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15A.

Conditions Prior to Occurrence

The reactor mode switch was in the REFUEL position with reactor coolant temperature less than 212°F.

Description of Occurrence

On Saturday, June 14, 1975, while performing surveillance on the five Electromatic Relief Valve Pressure Switches, it was found that 1A83B and 1A83E tripped at 1089 psig and 1085 psig, respectively. These values are in excess of the maximum allowable trip points of 1084 psig and 1082 psig, respectively, which are derived by adding appropriate head correction factors to the Technical Specification limit of 1070 psig. It is noted here that switches 1A83B and 1A83E are associated with valves NR108B and NR108E, respectively.

The "as found" and "as left" switch settings were:

<u>Switch</u>	<u>Associated Valve</u>	<u>"As Found" Setting</u>	<u>"As Left" Setting</u>
1A83A	NR108A	1077 psig	1079 psig
1A83B	NR108B	1089 psig	1084 psig
1A83C	NR108C	1077 psig	1077 psig
1A83D	NR108D	1082 psig	1082 psig
1A83E	NR108E	1085 psig	1081 psig

Apparent Cause of Occurrence

Instrument set point drift is identified as the cause of this occurrence. This has been discussed in Supplement No. 8 to the Application for a Full Term License (i.e., Amendment No. 68) for the Oyster Creek Nuclear Generating Station.

Analysis of Occurrence

The relief valves are provided to remove sufficient energy from the primary system to prevent the safety valves from lifting during a transient. The limiting pressure transient is that which is produced upon a turbine trip from rated design power with a failure of the bypass system to function. Under these conditions, the five relief valves are required to operate in order to prevent reaching the lowest set point of the primary system safety valves. It should be noted that a 25 psig margin exists between the resulting peak pressure and the lowest safety valve set point as added assurance that the safety valves will not lift during this transient. With valves NR108B and NR108E actuating at 5 psig and 3 psig, respectively, above the maximum allowable trip point of 1070 psig, and assuming the most limiting pressure transient had occurred, the lowest set point safety valve or valves may have been required to actuate in order to limit the pressure transient. Since the safety valve capacity provides sufficient vessel over-pressure protection against failure of all pressure release devices in addition to a failure of the reactor to scram, over-pressurization of the vessel would not have occurred.

Corrective Action

The involved pressure switches, 1A83B and 1A83E, were immediately reset to trip at allowable pressure levels. There are continuing efforts to resolve the incompatibilities between the Technical Specification setpoint limits and the sensor performance limits. It is felt that the conservative design margins associated with the derivation of the plant safety limits will permit

a change in the Technical Specifications to be made which will take into account the expected sensor performance variations. This will eliminate instances of abnormal occurrence reports caused by the normal variation in sensor setpoint within the design margins of the plant safety limits.

Failure Data

Manufacturer data pertinent to these switches are as follows:

Manufacturer	--	Dresser
Type	--	1539VX
Serial Nos.	--	BK3339 (1A83B)
		BN7126 (1A83E)

Previous abnormal occurrence reports:

Abnormal Occurrence Report No. 50-219/74-28

Abnormal Occurrence Report No. 50-219/74-39