


# Jersey Central Power & Light Company



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MEMBER OF THE  
General  Public Utilities Corporation

OYSTER CREEK NUCLEAR GENERATING STATION  
Forked River, New Jersey 08731

Abnormal Occurrence  
Report No. 50-219/75-19

## Report Date

July 17, 1975

## Occurrence Date

July 8, 1975

## Identification of Occurrence

Violation of the Technical Specifications, paragraph 2.3.3, failure of the RE 03A, B, and D Reactor High Pressure Sensors to trip at a value corresponding to a reactor pressure of 1060 psig. This event is considered to be an abnormal occurrence as defined in Technical Specifications, paragraph 1.15.A.

## Conditions Prior to Occurrence

The plant was at steady state power with the following major parameters:

Power:	Core, 1625 MWt
	Electric, 530 MWe
Flow:	Recirculation, $50.3 \times 10^6$ lb/hr
	Feedwater, $5.98 \times 10^6$ lb/hr
Stack Gas:	11,300 $\mu$ ci/sec

## Description of Occurrence

While performing a routine monthly surveillance test on the Reactor High Pressure Scram Sensors, RE 03A, B, C, and D, it was observed that RE 03A, B, and D tripped above the normal trip points of 1068, 1068, and 1066 respectively. The trip points of RE 03A and B reflect an 8 psig head correction, while RE 03C and D reflect a 6 psig correction. The surveillance test results were as follows:

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	<u>"As Found"</u> <u>Trip Point</u>	<u>"As Left"</u> <u>Trip Point</u>	<u>"Normal"</u> <u>Trip Point</u>
RE 03A	1077	1068	1069
RE 03B	1076	1068	1068
RE 03C	1065	1065	1066
RE 03D	1071	1066	1066

#### Apparent Cause of Occurrence

The cause of this occurrence is switch repeatability.

#### Analysis of Occurrence

Since three of the four Reactor High Pressure Scram Sensors exceeded the normal trip point, a reactor scram due to high reactor pressure would not have been initiated until reactor pressure reached 1065 psig. (This is equivalent to 1071 psig with the 6 psig head correction; sensors A and C - channel 1, sensors B and D - channel 2.) The purpose of these sensors is to prevent power operation above 1060 psig and to back up other scrams in order to assure never reaching the reactor coolant system pressure safety limit. The turbine trip with failure of bypass valves is the worst case transient with respect to peak pressure. The additional reactivity contributing to this transient by delay of Reactor High Pressure Scram action is considered to be minimal.

#### Corrective Action

The sensors were recalibrated and returned to service. As reported previously, efforts are continuing in an attempt to resolve the incompatibilities which exist between the Technical Specification limits and the sensor performance limits.

#### Failure Data

Manufacturer: Barksdale  
Type: Pressure Actuated Switch  
Switch No.: B2T - A12SS

Previous abnormal occurrence reports:

No. 50-219/75-7 dated January 23, 1974  
No. 50-219/74-36 dated July 10, 1974