TO:

James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

FROM:

Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket 50-2
Forked River, New Jersey 08731

SUBJECT:

Abnormal Occurrence Report No. 50-219/75/ 22

The following is a preliminary report being submitted in compliance with the Technical Specifications, paragraph 6.6.2.

Preliminary Approval:

W R Lewis Jr for 8/11/75

J. T. Carroll, Jr. Date

CC: Mr. A. Giambusso

Initial Telephone Report Date: 8/11/75

Date of Occurrence: 8/10/75

Initial Written Report Date: 8/11/75

Time of Occurrence: 1435

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/22

IDENTIFICATION OF OCCURRENCE: Violation of the Technical Specifications, paragraph 3.6.A.3, failure of the stack gas sample system to continuously monitor stack releases while the reactor was in an unisolated condition.

This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15B.

CONDITIONS PRIOR TO OCCURRENCE:

X	Steady State Power	____	Routine Shutdown Operation
____	Hot Standby	____	Load Changes During Routine Power Operation
____	Cold Shutdown	____	Other (Specify)
____	Refueling Shutdown		
____	Routine Startup Operation		

Power: Core, 1675 MWt
Electric, 550 MWe

Flow: Recirculation 13.5×10^4 gpm
Feed, 6.55×10^6 lb/hr.

Reactor Pressure: 1020 psig

Stack Gas: 10,200 μci/sec

DESCRIPTION OF OCCURRENCE: At 1435 on August 10, 1975 a stack gas sample line low flow alarm was received in the Control Room. An operator was dispatched to check the Stack Gas Sample Pump at which time, Stack Gas Sample Pump A was found not running. The operator reset the thermal overload which started Stack Gas Sample Pump A at 1437. The total amount of time the Stack Gas Sample Pump was out of service was approximately two (2) minutes.

PARENT CASE OCCURRENCE:

_____ Design
_____ Manufacture
_____ Installation/ Construction
_____ Operator
_____ Procedure
_____ Unusual Service Condition Inc. Environmental
_____ Component Failure
__X__ Other (Specify) The cause of this occurrence is under investigation.

ALYSIS OF CURRENCE: A review of the stack gas radiation monitor recorder traces showed the levels of both monitor channels to be relatively constant t 400cpsA) with no spiking before and after the pump trip. In a further 250cpsB fort to determine if excessive stack releases might have occurred during the proximate two(2) minute period that the Stack Gas Sample Pump was not erating, recorder traces of radiation monitoring systems associated with two seous streams released through the stack were reviewed. Off Gas radiation nitor recorder traces showed that the levels of both monitor channels were latively constant (at approximately 1.25×10^3 mr/hr) with no spiking for a riod of approximately 60 minutes prior to this event. Sixty(60) minutes is e Off Gas System holdup time prior to releasing to the stack. In addition, review of the Reactor Building Ventilation Exhaust radiation monitor corder traces showed that at the time of this event, the levels of both nitor channels were relatively constant (at approximately 1.5 mr/hr) with spiking. Based on these considerations and the very short period of time at the stack gas sample pump was not operating, the safety significance of is event is considered to be minimal.

RRECTIVE TION: The thermal overload protection was reset. After verifying that Stack Gas Sample Pump A was running with no abnormal conditions esent, Stack Gas Sample Pump B was put into service and Stack Gas Sample mpA was taken out of service.

ILURE DATA: Previous abnormal occurrences involving the Stack Gas Sample Pumps ar

1. Abnormal Occurrence Report No. 74/53
2. " " " " 74/54
3. " " " " 74/57
4. " " " " 74/61
5. " " " " 75/6

pared by: Thomas E Gunter Date: 8/11/75

E. CORRECTIVE ACTION

The load was removed from the plant at 4:15 a.m., on August 11, 1975, and plant cooldown was commenced at 11:10 a.m., in preparation for replacement of the faulty seal.

F. FAILURE DATA

To be addressed in the final report.

INVESTIGATOR:

R. E. Denton

cc: Director of Office of Nuclear Reactor Regulation