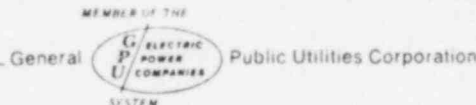


Jersey Central Power & Light Company



MADISON AVENUE AT PUNCH BOWL ROAD • MORRISTOWN, N. J. 07960 • 201-539-6111



OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Abnormal Occurrence
Report No. 50-219/75-24

Report Date

September 8, 1975

Occurrence Date

August 29, 1975

Identification of Occurrence

Violation of the Technical Specifications, paragraph 2.3.4, Electromatic Relief Valve Pressure Switches, 1A83C and 1A83D, were found to trip at pressures in excess of the maximum allowable value of 1070 psig. This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15.A.

Conditions Prior to Occurrence

The plant was in a routine shutdown operation with the reactor mode switch in the refuel position and reactor coolant temperature less than 212°F.

Description of Occurrence

On Friday, August 29, 1975, while performing surveillance on the five Electromatic Relief Valve Pressure Switches, it was found that 1A83C and 1A83D tripped at 1080 and 1084 psig, respectively. These values are in excess of the maximum allowable trip points of 1077 and 1082, which are derived by adding the appropriate head correction factors to the Technical Specification limit of 1070 psig. It is noted here that switches 1A83C and 1A83D are associated with valves NR108C and NR108D, respectively.

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The "As Found" and "As Left" switch settings were:

| <u>Switch</u> | <u>Associated Valve</u> | <u>"As Found" Setting</u> | <u>"As Left" Setting</u> |
|---------------|-------------------------|---------------------------|--------------------------|
| 1A83A | NR108A | 1073 psig | 1079 psig |
| 1A83B | NR108B | 1071 psig | 1084 psig |
| 1A83C | NR108C | 1080 psig | 1077 psig |
| 1A83D | NR108D | 1084 psig | 1082 psig |
| 1A83E | NR108E | 1082 psig | 1082 psig |

Apparent Cause of Occurrence

Instrument set point repeatability is identified as the cause of this occurrence.

Analysis of Occurrence

The relief valves are provided to remove sufficient energy from the primary system to prevent the safety valves from lifting during a transient. The limiting pressure transient is that which is produced upon a turbine trip from rated design power with a failure of the bypass system to function. Under these conditions, the five relief valves are required to operate in order to prevent reaching the lowest set point of the primary system safety valves. It should be noted that a 25 psig margin exists between the resulting peak pressure and the lowest safety valve set point as added assurance that the safety valves will not lift during this transient. With valves NR108C and NR108D actuating at 3 psig and 2 psig, respectively, above the maximum allowable trip point of 1070 psig, and assuming the most limiting pressure transient had occurred, the lowest set point safety valve or valves may have been required to actuate in order to limit the pressure transient. Since the safety valve capacity provides sufficient vessel overpressure protection against failure of all pressure relief devices in addition to a failure of the reactor to scram, overpressurization of the vessel would not have occurred.

Corrective Action

The involved pressure switches, 1A83C and 1A83D, were immediately reset to trip at allowable pressure levels. There are continuing efforts to resolve the incompatibilities between the Technical Specification set point limits and the sensor performance limits. It is felt that the conservative design margins associated with the derivation of the plant safety limits will permit a change in the Technical Specifications to be made which will take into account the expected sensor performance variations. This will eliminate instances of abnormal occurrence reports caused by the normal variation in a sensor set point within the design margins of the plant safety limits.

Failure Data

Manufacturer data pertinent to these switches are as follows:

Manufacturer - Dresser
 Type - 1539VX
 Serial Nos. - BK3340 (1A83C) and BK3338 (1A83D)

Previous abnormal occurrence reports:

Abnormal Occurrence Report No. 50-219/74-28


Abnormal Occurrence Report No. 50-219/74-39

Abnormal Occurrence Report No. 50-219/75-16

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MEMBER OF THE
 General  Public Utilities Corporation

September 8, 1975

Mr. A. Giambusso
 Director, Division of Reactor Licensing
 Office of Nuclear Reactor Regulation
 United States Nuclear Regulatory Commission
 Washington, D. C. 20555

Dear Mr. Giambusso:

Subject: Oyster Creek Station
 Docket No. 50-219
 Abnormal Occurrence Report No. 50-219/75-24

The purpose of this letter is to forward to you the attached abnormal occurrence report in compliance with paragraph 6.6.2.a of the Technical Specifications.

Very truly yours,

Donald A. Ross, Manager
 Generating Stations-Nuclear

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Enclosures

cc: Mr. J. P. O'Reilly, Director
 Office of Inspection and Enforcement, Region 1



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