



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. NPF-3
TOLEDO EDISON COMPANY
AND
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
DAVIS-BESSE NUCLEAR POWER STATION, UNIT NO. 1
DOCKET NO. 50-346

Introduction

By letter dated August 18, 1983, the Toledo Edison Company (TED) proposed several amendments to the Davis-Besse Unit No. 1 operating license. One of these proposed amendments concerns rewording the present Technical Specification (TS), Appendix A, Section 3.3.3.2.c. and the addition of Section 3.3.3.2.d. to differentiate between the required minimum number of operable symmetric incore detectors needed for Quadrant Power Tilt measurements and the minimum incore detectors in each quadrant needed for the calculation of hot channel factors $F_{\Delta H}^N$ and F_Q .

Evaluation

Section 3.3.3.2.c. of the current Davis-Besse TSs and Standard Technical Specification (STS) for Babcock and Wilcox Reactors, NUREG-0103 Rev. 4 are identical but as stated in the TED proposal the wording incorrectly implies that the hot channel factors $F_{\Delta H}^N$ and F_Q use only the Symmetric Incore Detector System detectors when actually all incore detectors are used in calculating these factors. TED has proposed adding a new Section 3.3.3.2.d. to correct this misstatement in the specifications. Proposed Section 3.3.3.2.d. requires that 75% of all incore detectors in each quadrant be operable. Section 3.3.3.2.c. has been reworded to reflect this additional section, requiring 75% of the symmetric incore detectors be operable for Quadrant Power Tilt measurements. Since these proposed changes to Page 3/4 3-35 of the specifications only clarify which incore detectors will be used for each function and also maintain the 75% operability requirement, we find them acceptable and agree that they correctly reflect the intent of the STS.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area. The staff has determined that the amendment involves no significant increase in the amounts of any

effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 6, 1984

The following NRC personnel have contributed to this Safety Evaluation:
K. R. Ridgway