

FROM:

SUBJECT:

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

50-244

FEB 0 4 1982

MEMORANDUM FOR: Darrell J. Eisenhut, Director Division of Licensing, NRR

> Edward L. Jordan, Director Division of Engineering and Quality Assurance, IE

Carlyle Michelson, Director Office for Analysis and Evaluation of Operational Data

Richard W. Starostecki, Director Division of Resident and Project Inspection, Region I

Ronald C. Haynes, Regional Administrator Region I

NRC INQUIRY REGARDING GINNA EVENT OF JANUARY 25, 1982

The following is a brief summary of our discussion today regarding the Ginna event and attached is an outline for the inquiry.

- As pointed out by Vic Stello, we are to come up with a draft report
 to the Commissioners within 45 days which is a factual report of what
 happened. We are to have findings but not recommendations. As currently
 envisioned, this report would be used by the various NRC offices as the
 basic data from which analyses could be performed and recommendations,
 as appropriate, be formulated.
- I am accountable for the conduct of the inquiry and will be assisted by personnel*from the various NRC offices.
- 3. Guidance, advice and review will be provided by a steering group comprised of Jack Heltemes, Darrell Eisenhut, Ed Jordan, and Rich Starostecki. I will function as chairman of this group. It is expected that up to four work days of effort will be required of members of the steering group.
- The inquiry will be performed by NRC personnel from the various offices managed by the EDO.
- I will select the Director of the inquiry group with the advice and counsel of the steering group.

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ENCLOSURE E

- 6. The inquiry will require the full-time services of seven to ten key persons to be selected from the various NRC offices. Additional support will be provided on an "as needed" basis by other personnel from the various NRC offices.
- 7. Work space will be provided at Region I for the key persons conducting the inquiry. It is expected that these key personnel will spend a major part of their time at the Region I office during this work.
- 8. The sequence of events for this inquiry will cover the period from 9:25 am on January 25, 1982 to 10:45 am on January 26, 1982 when the event was de-escalated to the "Recovery Phase."
- 9. It is expected that the report will consist of two volumes. The report will be in the format and style for issuance as a NUREG document. The first volume will cover information according to the attached outline. The second volume will contain reference data, e.g., instrument charts, procedures, copies of data sheets, etc.

You are requested to review the attached outline and provide information/ suggestions/questions/issues to be covered in each of the six separate sections. Of course, if additional sections are appropriate in your view, please advise. The steering group will meet at 3:00 pm on Monday, February 8, 1982 at the AEOD Conference Room, Second Floor, East West Building (South). Richard Starostecki will act for me as Chairman in this meeting. Also at this meeting, please be prepared to recommend persons who should participate as a key person in the inquiry working group.

Ronald C. Haynes, Regional Administrator Region I

Attachment: As stated

cc w/attachment: WDircks, EDO VStello, DEDROGR HDenton, NRR RDeYourg, IE

Outline

Ginna Steam Generator Tube Rupture Event

- NRC Investigation -

Region I I. Introduction and Executive Summary

Begin I II. Chronology of Events

Signature of events to place further discussion in perspective)

III. Plant Performance

- . Systems Response
 - Secondary system
 - Reactor protection system
 - Reactor coolant system
 - Safety injection system
 - Containment isolation system
 - Instrument air system
 - Auxiliary feedwater system
 - Charging/letdown systems
 - Residual heat removal

B. Instrument Response

- Secondary plant instrumentation (S/G levels, pressures, feedwater flows, air ejector condenser monitors, steam flows)
- Primary plant instrumentation
 - parameters (temps pressures, levels, flow)
 - ii thermocouples
 - iii tail pipe temps on pressurizer PORV
 - iv saturation/subcooling meters

C. Equipment Response

- Pressurizer PORV Ponderer, Autin't'-Son Sur' Steam generator stressment relief Process Computer (put in IV-A alama
- Steam generalor safety valves
- Pressurizer valuef tank - Control rod habud fars
- Control room indicators/alarms
- Main steam line B up to MSIV (stress)

IV. Procedures/Human Factors

- A. Control Room Responses
 - Licensed operators
 - STA
 - Other
 - + Alarms, recorders, and indicators frame devices

B. Procedures

- Identification and selection
- Bases (generic guidelines used to develop procedures) - Procedural guidelines svailable and those not provided
- Impact, if any, of degraded condition
- Small break LOCA procedure

C. Technical Support Center

- Pole of plant superintendent

D. Facility Description move to "A" - Laugus.

- Control Room Seg Laugus.

Radio

V. Radiological Response

- A. Releases to Environment
 - Air ejectors

 - Steam generators Primoru Coolant Achinty
- B. Releases to Containment
- C. Plant/Onsite Contemination

i meteorological factors

ii quantification

iii impacts, if any

D. Environmental Results

i offsite sampling - licensee and analyses

- other

ii dose essessment (calculated)

iii dose assessment (measured)

-iv meteorological factors

VI Organizational Response (Communication, transportation, level of support, etc.)

- A. Licensee TSC Corpurate, etc. Public affairs
- management team (HQ) B. NPC response team (Region I) regional office public affairs state liaison technical support

- C. NY State and Counties
- D. FEMA, EPA, and Other Federal Agencies
- VII. Post Event Activities/Results
 - A. Event termination RHR, normal shutdown mode
 - B. Licensee Rerovery Plan
 - C. Preliminary Results
 - Leakage rate
 - Tube rupture