

DUKE POWER

February 27, 1992

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Subject: McGuire Nuclear Station Unit 1

Docket No. 50-369

Licensee Event Report 369/92-01

Gentlemen:

Pursuant to 10 CFR 50.73 Sections (a) (1) and (d), attached is Licensee Event Report 369/92-01 concerning failure to remove a steam generator tube from service. This report is being submitted in accordance with 10 CFR 50.73 (a) (2) (i). This event is considered to be of no significance with respect to the health and safety of the public. A supplemental report will be submitted on 4/8/92.

Very truly yours,

T.C. McMeekin

TLP/bcb

Attachment

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Mr. P.K. Van Doorn NRC Resident Inspector McGuire Nuclear Station

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| McGuiré Nuclear Station, Unit 1 TITLE(4) A Technical Specification Tiolation Occurred Because Of Fails | | | | | | | | | | | ER(2) 69 | inger en majourne en en | OF 1 |
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| EVENT DATE(5) LER NUMBER(6) | | | | | | | REPORT D | ATE(7) | OTHER FACILITIES INVOLVED[8] | | | | |
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| OPERATI | NO. | Ti | THIS | REPORT IS SUBMI | TIED PURSUANT | TO REQU | IREMENTS | OF LOCKE | (Check one or | more of the f | ollowi | ng)(11) | |
| MODE(9) | | | 20.402(b) | | 20.405(c) | | | 50.73(a)(2)(1v) | | | 73,71(b) | | |
| | | | 20.405(a)(1)(1) | | 50.36(c)(1) | | | 50.73(a)(2)(v) | | | 73.71(c) | | |
| POWER 97% | | | | 20,405(a)(1)(11) | | 50.36(c)(2) | | 50.73(a)(2)(vii) | | | OTHER (Specify in Abstract belo and in Text) | | |
| | | | | 20.409(a)(1)(iii) X | | 50.73(a)(2)(1) | | 50.73(a)(2)(viii)(A) | | | | | |
| | | | | 20.405(a)(1)(i | 05(a)(1)(iv) | | 50.73(a)(2)(11) | | 50.73(a)(2)(v111,(3) | | | | |
| | | | | | | 50.73(a)(2)(311) EE CONTACT FOR THIS LER(12 | | 50.73(a)(2)(s) | | | | | |
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| name Terry | L. Pe | ederse | | pervisor, M | | | | | 704 | DD8 87 | 5-44 | | |
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| SUPPLEMENTAL REPORT EXPECTED(14) EXPECTED | | | | | | | | | | | ONTH | DAY | YEAR |
| SUPPLEMENTAL REPORT BATESTAN SUBMISSION | | | | | | | | | | | | 8 | 9.2 |

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines (16) On January 16, 1992, at 1912, Operations Control Room personnel, commenced an orderly shutdown of Unit 1, as a result of primary to secondary leakage, which had increased to aprroximately 235 gallons per day, on Steam Generator (SG) ID. Prior to the shutdown, Unit 1 was operating in Mode 1 (Power Operation) at 97 percent power. Unit 1 was taken off line at 0649, January 17, 1992, and entered Mode 5 (Cold Shutdown) on January 18, 1992, at 0552. The leakage was attributed to SG tube 47-46, leaking 30 drops per minute at secondary side head pressure, and SG tube 36-30, leaking 2 drops per minute at 180 psig above head pressure. Subsequent investigation of the event determined that SG tube 47-46 had an indication in excess of the Technical Specification limit of 40 percent. Tube 36-30 leakage was attributed to an inadequate kinetic weld bond. Re-evaluation of Motorized Rotating Pancake Coil (MRPC) Eddy Current Test (ECT) data collected during the 1991 End Of Cycle (EOC) 7 refueling cutage, revealed that the indication on tube 47-46 existed at that time and was greater than the 40 percent through wall limit. Duke Power personnel and industry experts conducted an extensive historical ECT data review, acquired additional MRPC data, and developed new plugging criteria. Tubes 47-46 and 36-30 were plugged, along with other suspect tubes located in SGs 1A, 1B, 1C, and 1D. A supplemental report will be submitted when the investigation has been completed.