



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

AUG 1 2 1981

Bob Bauer
Wayne Reinmuth
Consider input
into adverse test
results Info Notice

MEMORANDUM FOR: Robert L. Tedesco, Assistant Director
for Licensing
Division of Licensing

Thomas H. Novak, Assistant Director
for Operating Reactors
Division of Licensing

FROM: James P. Knight, Assistant Director
for Components & Structures Engineering
Division of Engineering

SUBJECT: REPORTING OF UNSATISFACTORY EPRI/PWR TEST RESULTS FOR
CROSBY MODEL 3K6 SAFETY VALVE

EPRI related

Info Notices:
81-29
82-52

The attached memorandum from EPRI for the period July 25 - 31, 1981 discusses the results of steam and water tests performed at Combustion Engineering on the Crosby 3K6 safety valve. As discussed in the memorandum, the safety valve failed one or more of the test screening criteria in these tests. The two screening criteria failed most in these tests are:

- (a) Valve must open within plus or minus 3% of valve design set pressure.
- (b) Valve fully closes when inlet pressure is less than the pressure at which the valve opened and greater than 2250 psig.

In addition to failing some specific screening criteria, there was evidence of minor galling and damage to valve internals and indication of fairly high valve leakage - 15 gpm after several high pressure water tests.

Note that all of these tests were performed with a short inlet piping configuration. The valve has not been tested as yet with the long loop seal inlet piping configuration that is normally used on Westinghouse plants. The next series of tests to be performed on the valve will be with the long loop seal piping.

It is our understanding that the Operating License and Construction Permit Holders that utilize or plan to utilize these safety valves and the PWR NSSS vendors have been notified of these test results. These licenses and permit holders have the responsibility for independently assessing the safety significance of the observed valve behavior for their plants, including the reporting of significant findings to the NRC.

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Our information from EPRI indicates that the Crosby 3K6 valve is being used or will be used on the following plants:

St. Lucie 1 and 2
Fort Calhoun 1

Although the specific safety significance of these test results is still being evaluated, this information may be relevant for Board notification.

If a decision is made to forward this information to one or more Boards, we recommend that a copy of the July 22, 1981 memorandum from Frank Cherny to Robert J. Bosnak be forwarded to the Board also. A copy of the memorandum was previously distributed to you. It contains the minutes of the July 17, 1981 meeting between the staff and EPRI and the PWR Owners Group at which the status of all of the Safety Valve and PORV testing in the EPRI program was discussed.



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