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HAL B. TUCKER VICE PRESIDENT NUCLEAR PRODUCTION

March 23, 1984

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PROPRIETARY

MATERIAL TRANSMITTED HEREWITH

CONTAINS 2.790 MATERIAL

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief Licensing Branch No. 4

Subject: McGuire Nuclear Station Docket Nos. 50-369, and 50-370 McGuire 1/Cycle 2 OFA Reload

Dear Mr. Denton:

My letter of December 12, 1983 transmitted proposed License Amendments to Facility Operating Licenses NPF-9 and NPF-12 for McGuire Nuclear Station Units 1 and 2, respectively. These amendments, which basically change plant operating limitations given in the Technical Specifications affected by use of the Optimized Fuel Assembly (OFA) Design for McGuire Unit 1/cycle 2 to ensure plant operation consistent with the Design and Safety Evaluations, were subsequently revised by my letter dated February 20, 1984. The "Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Proposed no Significant Hazards Consideration Determination and Opportunity for Hearing" related to the submittals was published in the Federal Register on March 2, 1984.

Ms. E. G. Adensam's letter dated March 5, 1984 requested additional information necessary to support NRC Staff Review of the Proposed Amendments. This information was provided by my letter of March 9, 1984. As a result of continuing NRC Staff Review, the following revisions to the Proposed Amendments were requested by Messrs. Y. Hsii and M. S. Dunenfeld in Telecons on March 15 and 16, 1984, respectively.

Attachment 1 is a revision to Proposed Technical Specification Bases Page B 2-1 which clarifies where the minimum DNBR value of 1.30 for Unit 2 comes from. This clarification was necessitated in view of the differing correlations used for this application on Units 1 and 2.

Attachment 2 is a revision to Proposed Technical Specification Bases Page B 3/4 2-4 insert "X" which provides information on the thermal margin available to offset various penalties for Unit 1. The original submittal provided this information only for Unit 2.

Attachment 3 is a revision to Proposed Technical Specification Page 6-23 which reflects the fact that the Radial Peaking Factor Limit Reporting Requirements for Units 1 and 2 differ as a result of the OFA reload on Unit 1 (i.e. Fxy limit for Unit 2, W(z) function for Unit 1). Since the Unit 2/cycle 2 reload is planned to use the OFA design, the December 12, 1983 submittal deleted the current F_{xy} Unit reporting requirements for both units in anticipation of the W(z) Function

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Report being required for Unit 2/cycle 2 startup. However, this did not take into account the possibility (although unlikely) of a limit report submittal becoming necessary at some time during Unit 2/cycle 1 core life (as was the case with the Unit 1 Mid Cycle 1 Core Redesign), in which case an $F_{\rm XY}$ limit report would be appropriate. The revision provides for the separate reporting requirements of each unit.

Since the attached revisions are corrections/clarifications to the previous submittal which is currently under review and are bounded by the analyses of that submittal, no additional justification and safety analysis, significant hazards consideration, or amendment fee is required.

In reference to the above mentioned March 9, 1984 letter which provided additional information requested by the NRC Staff, it was requested that portions of the information which is proprietary to Westinghouse Electric Corporation be withheld from public disclosure. In support of that request, enclosed is one copy of the application for withholding, CAW-84-18 (non-proprietary), and the Westinghouse Affidavit, AW-76-60 (non-proprietary). Also enclosed are two copies (one "proprietary" version and one "non-proprietary" version) of the responses on the McGuire Nuclear Station instrumentation uncertainties for the Improved Thermal Design Procedure (Attachment No. 1 of the March 9, 1984 letter-response to questions No. 10.(3), 10.(4), and 10.(6)), and the uncertainties used in the McGuire ITDP DNBR Analyses (Tables 1,2, and 3 of the March 9, 1984 letter-response to question No. 10.(5)). These items are submitted in support of the ITDP for the McGuire Nuclear Station OFA reload transition core and is to be treated as proprietary information of Westinghouse Electric Corporation. The information is marked and bracketed accordingly. The information will be separately resubmitted in whole in conformance with the requirements of 10CFR2.790 should it be employed as part of a license application or other action identified in 10CFR2.790(a).

Correspondence with respect to the affidavit or application for withholding should reference AW-84-18 and should be addressed to R. A. Wiesemann, Manager of Regulatory and Legislative Affairs, Westinghouse Electric Corporation, P.O. Box 355, Pittsburgh, Pennsylvania 15230.

Should there be any further questions concerning these matters, please advise.

Very truly yours,

Hal B. Tucker

Hal B. Tucker

PBN:glb Attachments Enclosures

cc: (w/attachments-enclosures)
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cc: (w/attachments-enclosures)
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Mr. W. T. Orders Senior Resident Inspector McGuire Nuclear Station