

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE			
01	V	A	S	P	S	I				0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	0	0	1	
7	8	9					14			15										25	26					30	31	32

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER							EVENT DATE					REPORT DATE									
01	CON'T	P	O	T	L	0	5	0	-	0	2	8	0	0	8	0	5	7	6	0	8	2	0	7	6
7	8	57	58	59	60	61							68	69					74	75					80

EVENT DESCRIPTION	DATE	TIME	LOCATION	STATUS
...	...	...	...	...

02 The Virginia Electric and Power Company was informed by its nuclear steam supply  
 7 8 9  
 03 system vendor that the rod bow effect on thermal margins was determined to be greater  
 7 8 9  
 04 than previously anticipated. This event is reportable per Technical Specification  
 7 8 9  
 05 6.6.2.a.8 (AO-S1-76-06).  
 7 8 9  
 06  
 7 8 9

## CAUSE DESCRIPTION

08 Experimental data collected by the nuclear steam supply system vendor shows that the  
7 8 9  
09 effects of a bowed fuel rod are worsened if a control rod thimble is located next to  
7 8 9  
10 the bowed fuel rod. The physical reasons for this phenomenon are not well known (cont)  
7 8 9

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	E	1	0	0	N/A	D	Notification by Vendor		
7	8	9	10	11	12	13	44	45	46
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE	
1	Z	Z	N/A				N/A		
7	8	9	10	11	12	13	44	45	46

## PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION	
1	3	0	0	0	Z	N/A

## PERSONNEL INJURIES

NUMBER				DESCRIPTION	
1	4	0	0	0	N/A

## OFFSITE CONSEQUENCES

15 | No offsite consequences resulted from this event.  
7 8 9

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION
16	Z	N/A

## PUBLICITY

17	N/A	0610419 780820 PDR ADOCK 05000280 S PDR	
7 8 9			80

### ADDITIONAL FACTORS

10 This event is the subject of a detailed technical report forwarded to the NRC under

19 Virginia Electric and Power Company letter serial number 194, dated August 18, (cont)

NAME: Tyndall L. Baucom

PHONE: (804) 357-3184

CAUSE DESCRIPTION (con't)

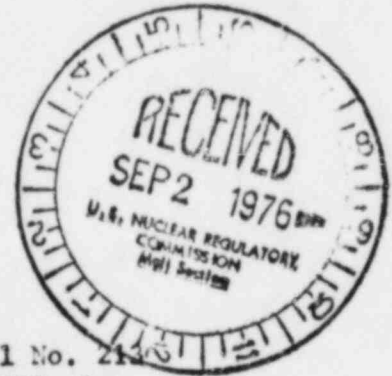
at this time, however, the enthalpy rise hot channel factor  $F_{\Delta H}^N$  is being limited to offset the effect.

ADDITIONAL FACTORS (con't)

1976. This report presents the operational restrictions that have been applied to Surry Unit Nos. 1 and 2 as a result of this event. The health and safety of the public was not affected by this event.

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

August 31, 1976



Mr. Norman C. Moseley, Director  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Region II - Suite 818  
230 Peachtree Street, Northwest  
Atlanta, Georgia 30303

Serial No. 2145  
PO&M/ALH:clw

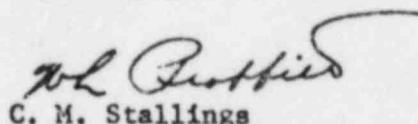
Docket No. 50-280  
License No. DPR-32

Dear Mr. Moseley:

Pursuant to Surry Power Station Technical Specification 6.6.2, the Virginia Electric and Power Company hereby submits a copy of Licensee Event Report No. AO-S1-76-06.

The substance of this report has been reviewed by the Station Nuclear Safety and Operating Committee and will be placed on the agenda for the next meeting of the System Nuclear Safety and Operating Committee.

Very truly yours,



C. M. Stallings  
Vice President-Power Supply  
and Production Operations

Enclosure

cc: Mr. Robert W. Reid, Chief ✓  
Operating Reactors Branch 4  
(40 copies AO-S1-76-06)

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