Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

LIC-91-326R
December 13, 1991

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: November Monthly Operating Report (MOR)

Enclosed is the November 1991 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification Section 5.9.1.

If you should have any questions, please contact me.

Sincerely,

for W. G. Gates
Division Manager
Nuclear Operations

WGG/sel

Enclosures

c: LeBoeuf, Lamb, Leiby & MacRae
R. D. Martin, NRC Regional Administrator, Region IV
R. P. Mullikin, NRC Senior Resident Inspector
D. K. Sentell, Combustion Engineering
R. J. Simon, Westinghouse
Office of Management & Program Analysis (2)
INPO Records Center
American Nuclear Insurers

IE24 1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	Fort Calhoun Station
DATE	December 9, 1991
COMPLETED BY	M. L. Edwards
TELEPHONE	(402)636-2451

MONTH November 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	487	17	487
2	487	18	487
3	487	19	486
4	487	20	486
5	487	21	485
6	487	22	486
7	487	23	486
8	487	24	486
9	487	25	486
10	487	26	486
11	488	27	486
12	488	28	487
13	488	29	487
14	488	30	487
15	487		
16	487		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE December 9, 1991
COMPLETED BY M. L. Edwards
TELEPHONE (402)636-2451

OPERATING STATUS

1. Unit Name: Fort Calhoun Station — Notes
2. Reporting Period: November 1991
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (Item Numbers 3 through 7) Since Last Report, Give Reasons:
   N/A
9. Power Level to Which Restricted, If Any (Net MWe): N/A
10. Reasons for Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	720.0	8,016.0	159,386.0
12.	Number of Hours Reactor was Critical	720.0	7,286.0	124,074.7
13.	Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14.	Hours Generator On-Line	720.0	7,203.2	122,633.1
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	1,076,325.2	9,228,684.4	160,512,173.7
17.	Gross Electrical Energy Generated (MWH)	366,750.0	3,047,480.0	52,797,606.2
18.	Net Electrical Energy Generated (MWH)	350,526.0	2,887,287.2	50,372,063.5
19.	Unit Service Factor	100.0	89.9	76.9
20.	Unit Availability Factor	100.0	89.9	76.9
21.	Unit Capacity Factor (Using MDC Net)	101.8	75.4	68.4
22.	Unit Capacity Factor (Using DER Net)	101.8	75.4	66.9
23.	Unit Forced Outage Rate	0.0	10.1	3.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    Refueling outage scheduled to begin February 1, 1992 and last approximately three months.
25. If Shut Down at End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): Forcast Achieved

INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending November 1991

1. Scheduled date for next refueling shutdown. — February 1, 1992

2. Scheduled date for restart following refueling. — April 29, 1992

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Incorporate specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — November 1991

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — New fuel supplier
   New LOCA Analysis

6. The number of fuel assemblies:
   a) in the core — 133 Assemblies
   b) in the spent fuel pool — 477 Assemblies
   c) spent fuel pool storage capacity — 729 Assemblies
   d) planned spent fuel pool storage capacity — Planned to be increased with higher density spent fuel racks.

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1995 *

* Capability of full core offload of 133 assemblies lost. Reracking to be performed between the 1993 and 1995 Refueling Outages.

Prepared by Kevin Walker — Date 12-6-91

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St. ion
DATE December 10, 1991
COMPLETED BY M. L. Edwards
TELEPHONE (402) 636-2451

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH NOVEMBER 1991

No.	Date	Type (1)	Duration (Hours)	Reason (2)	Method of Shutting Down Reactor (3)	Licensee Event Report #	System Code (4)	Compone Code	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns or significant reductions in power during November 1991.

1
F-Forced
S-Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

(9/77)

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

November 1991
Monthly Operating Report

I. OPERATIONS SUMMARY

Fort Calhoun Station (FCS) operated at a nominal 100% power throughout the month of November 1991. Early in November, the 345 KV electrical grid system experienced numerous disturbances due to the winter storm and associated icing that began on October 31, 1991. One of the disturbances on the turbine generator was so severe that a turbine plant cooling water pump tripped.

On November 6, 1991, the overhaul of main feedwater pump FW-4C was completed. Upon subsequent starting of the pump, the stainless steel flexible tube to the suction relief valve failed resulting in a high pressure steam leak. The pump was isolated by Operations personnel and allowed to cool. Later that week, replacement of the flexible tube allowed the pump to be put into service.

A four hour report was made to the NRC on November 6, 1991 due to an inadvertent actuation of Ventilation Isolation Actuation Signal (VIAS). This occurred when an electrical maintenance worker pulled the incorrect fuse block during the performance of a tagging clearance. The report was made pursuant to 10 CFR 50.72(b)(2) because of the inadvertent actuation of an Engineered Safety Feature (VIAS).

A one hour report was made to the NRC pursuant to 10 CFR 50.72(b)(1) on November 14, 1991 because two support hangers on the Safety Injection system were determined to be outside the seismic design criteria of the FCS USAR Appendix F. The installed hangers have a safety margin of 2.4 on the anchor bolts while the USAR Appendix F criteria requires a margin of 3.0. The hangers are still considered operable based upon interim operability criteria allowing hangers with a safety margin of 2.0 or above on the anchor bolts to remain operable.

On November 19, 1991 the NRC notified the Training Department that all candidates (7 Reactor Operators and 7 Senior Reactor Operators) passed the initial licensed operator examination.

Due to severe winter weather on November 30, 1991, two unexpected fast transfers of vital 4160 volt buses 1A3 and 1A4 from 161KV (off-site power source) to 345 KV (station power source) occurred. The first transfer occurred as a result of a voltage fluctuation on the electrical grid system due to the loss of a 69 KV line which runs between Fort Calhoun and Blair, Nebraska. The electrical distribution system was returned to normal alignment within approximately 10 minutes following confirmation from System Operations that 161 KV was available. The second unexpected fast transfer occurred due to failure of an insulator. After repairs were completed on the 161 KV line, the electrical distribution system was returned to its normal alignment. In both instances, immediate notification of the loss of 161 KV was made to the NRC.

The following NRC inspection took place during the month of November 1991:

IR 91-23 Routine Inspection

The following LERs were submitted during November 1991:

LER NO.	Description
91-21	Inadvertent Containment Isolation Actuation Signal
91-22	Nuclear Instrumentation Outside Design Basis (RG 1.97)
91-23	Failure to Meet Technical Specification Regarding Core Power Distribution Verification
91-10, R1	HELB Issue Related to Auxiliary Steam in Room 57

A. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

None

B. RESULTS OF LEAK RATE TESTS

The total reactor coolant system (RCS) leakrate for November 1991 averaged approximately 0.35 gpm. The known leakrate to the reactor coolant drain tank (RCDT) increased to an average of 0.40 gpm. The known leakrate was greater than the total leakrate because water downstream of the RCDT was leaking back into the RCDT through leaky check valves. As a result of the larger known leakrate than total leakrate, negative unknown leakrate values were obtained for most of November 1991.

To minimize the effect of the check valve leakage on the RCS leakrate value, the containment isolation valves from the RCDT to the waste holdup tanks are now kept closed except when pumping the contents of the RCDT to the waste holdup tanks. This change was implemented on November 21, 1991 and has since resulted in the known leakrate falling to about 0.25 gpm with the unknown leakrate averaging less than 0.1 gpm. The leaky check valves are to be inspected and repaired during the 1992 refueling outage.

C. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10 CFR 50.59

AMENDMENT NO.	DESCRIPTION
139	Adopted the guidance of Regulatory Guide 1.35 Rev. 3 for the surveillance of the containment tendons.

D. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF NOVEMBER 1991:

Completed replacement of the demineralized water/rust inhibitor coolant from the jacket water system on both Emergency Diesel Generators with an antifreeze solution in preparation for cold weather.

During surveillance testing, a low flow condition was discovered on hydrogen analyzer pump VA-81B. The pump was found to be within specifications, but the flow rate was found out of tolerance due to the pump suction valve not opening fully. The valve was repaired and the surveillance test was performed to verify proper flow.