PHILADELPHIA ELECTRIC COMPANY

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January 14, 1975

Mr. A. Giambusso
Deputy Director of Reactor Projects
United States Atomic Energy Commission
Directorate of Licensing
Washington, DC 20545

Dear Mr. Giambusso:

The following occurrence was reported to Mr. Walt Baunack, A.E.C. Region 1 Regulatory Operations Office on January 4, 1975. Written notification was made to Mr. James P. O'Reilly, Region 1 Regulatory Operations Office on January 6, 1975. In accordance with Section 6.7.2.A of the Technical Specifications, Appendix A of DPR-56 for Unit 3 Peach Bottom Atomic Power Station, the following report is being submitted to the Directorate of Licensing as an Abnormal Occurrence.

Reference: License Number DPR-56
Technical Specification Reference: 4.7.A

Report No.: 50-278-75-2
Report Date: January 14, 1975
Occurrence Date: January 4, 1975
Facility: Peach Bottom Atomic Power Station
R.D. 1, Delta, Pennsylvania 17314

Identification of Occurrence:

Drywell equipment door leak.

Conditions Prior to Occurrence:

Unit 3 shutdown. Primary coolant temperature approximately 150°F.

Description of Occurrence:

During a routine local leak rate test of the drywell equipment door on the Unit 3 containment, leakage from the double O-ring gasketed joint was found to be excessive. Leakage was past both seals into the drywell and into the reactor building. Additional testing verified that leakage from the containment to the reactor building was slightly in excess of 2,000 cc's per minute.

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Designation of Apparent Cause of Occurrence:

Investigation showed that the outer sealing gasket on the hatch was twisted and pinched over a 3 inch section.

Analysis of the Occurrence:

This occurrence resulted in a leakage from the drywell to the reactor building of slightly more than 2,000 cc's per minute. The total measured leakage to date is 8,632 cc's per minute. The combination of these two values still results in a leakage rate which is within 60% LA (16,400 cc's per minute). The safety implications of this occurrence are therefore minimal.

Corrective Action:

The equipment hatch was removed and new gaskets installed after being coated with a thin film of high temperature grease. The hatch was reinstalled. A local leak rate test performed after repairs resulted in negligible leakage. A local leak rate test performed on a similar closure associated with the drywell air lock also indicated negligible leakage.

Failure Data:

None.

Very truly yours,

M. J. Cooney
Ass't Gen'l Superintendent
Generation Division

cc: Mr. J. P. O'Reilly
Director, Region 1
United States Atomic Energy Commission
631 Park Avenue
King of Prussia, PA 19406