Public Service Company of Collorado

P. O. Box 361, Platteville, Colorado 80651

March 14, 1975

Mr. E. Morris Howard, Director Nuclear Regulatory Commission Region IV Office of Inspection & Enforcement 611 Ryan Plaza Drive Suite 1000 Arlington, Texas 76012

Dear Mr. Howard:

REF: Facility Operating License No. DPR-34

Docket No. 50-267

Enclosed please find a copy of Abnormal Occurrence Report No. 50-267/75/4-A, submitted per the requirements of the Technical Specifications.

Very truly yours,

H. Larry Brey

Superintendent-Operations Fort St. Vrain Nuclear

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Generating Station

HLB:il

cc: Mr. Angelo Giambusso

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COPY SENT REGION

HARRICH CO. F. SER TO SERVICE STRUCTURE

REPORT DATE: February 5, 1975

OCCURRENCE DATE: January 25, 1975

*Subsequently determined not to be an Abnormal Occurrence

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
P. O. BOX 361
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/75/4-A

Final

IDENTI	FI	CAT	ION	OF
OCCURR	EN	CE:		

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When the secondary closure of refueling region #1 was loosened, Health Physics personnel detected Tritium leaking from the penetration, indicating a possible breach of the primary closure.

Such a breach would be a violation of paragraph 2.1.c of the Technical Specifications.

CONDITIONS PRICE	Steady State Po	wer	Routine Shutdown
	Hot Shutdown		Routine Load Change
X	Cold Shutdown	Other (specify)	
	_ Refueling Shutd		
	Routine Startup		
The major parameters at the	time of the event	were as foll	ows:
Power	Rtr		MWth
	Elect.	0	MWe
Secondary Coolant	Pressure	N/A	psig
	Temperature	N/A	_ °F
, Primary Coolant	Flow	N/A	#/hr.
	Pressure	87	psig
	Temperature _	275	*F Core Inlet
	_	260	*F Core Outlet
	Flow Less th	an 100,000	0/hr.

Note: Average core temperature is lower than core inlet because the Primary Coolant was being heated by passing steam through the reheat section of the steam generators.

DESCRIPTION OF OCCURRENCE:

Testing of the operability of Reserve Shutdown Hoppers after modifying the pressurizing supply lines to a "soft seat" type revealed the inability to trip the test switch in No. 1 CRD with 35# pressure. It was decided to check the pressurizing line in No. 1 refueling penetration.

The secondary closure plate on refueling penetration #1 was being removed to examine the Reserve Shutdown pressurizing line. Routine Health Physics coverage established that tritium levels around the secondary seal went to approximately twice background when the cover hold down bolts were loosened. This indicated a possible communication of the penetration interspace with the primary coolant (i.e.; a breach of the primary closure).

The secondary cover plate was retightened to stop any outleakage and Tritium levels dropped to the previous background level.

The leak, if any, through the primary closure, is less than the allowable for the group, as shown on flow gauges.

APPARENT CAUSE OF OCCURRENCE:

The helium used to pressurize the interspaces on all refueling penetrations is obtained from the high pressure helium storage bottle. This source of helium is replenished from the helium storage system. During depressurization of the primary coolant system, helium is bled from the purification system ahead of the hydrogen getter unit to storage. This helium would, therefore, contain hydrogen and tritium above background levels. Therefore, when the secondary closure plate was lifted and an increase in tritium levels was noted, the increase above background was due to the normal presence of tritium in the pressurizing gas.

ANALYSIS OF OCCURRENCE:

There was no Abnormal Occurrence. Examination of the primary closure seals and pressurization of the interspace revealed no leakage.

CORRECTIVE ACTION:

Not necessary.

FAILURE DATA/ SIMILIAR REPORTED OCCURRENCES:

None

PROGRAMMATIC IMPACT:

None expected due to concurrent maintenance requirements.

CODE IMPACT:

None

RECOMMENDED:

Frank Mathie

Superintendent-Maintenance

Fort St. Vrain Nuclear Generating Station APPROVED:

Frederic E. Swart

Superintendent Nuclear Production

Public Service Company

of Colorado