

June 19, 1995

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555-0001

Subject: WCAP-14046, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 3 (Proprietary)

Byron Nuclear Power Station, Unit 1
Facility Operating License NPF-37
NRC Docket No. 50-454

Braidwood Nuclear Power Station, Unit 1
Facility Operating License NPF-72
NRC Docket No. 50-456

- References:
1. George F. Dick, Jr. (NRC) letter to D. L. Farrar (ComEd), Issuance of Amendments (TAC Nos. M90052 and M90053), dated October 24, 1994
 2. Denise M. Saccomando (ComEd) letter to NRC Document Control Desk, Schedule for LOCA+SSE Steam Generator Load Evaluation for Byron Station Unit 1 and Braidwood Station Unit 1, dated November 23, 1994
 3. Harold D. Pontious, Jr. (ComEd) letter to NRC Document Control Desk, Evaluation of LOCA + SSE Load Effects on Steam Generator Tubes, dated February 15, 1995

Gentlemen:

In Reference (1), the United States Nuclear Regulatory Commission (NRC) granted approval for the application of a voltage-based Tube Support Plate (TSP) Interim Plugging Criteria (IPC) to disposition steam generator (SG) tubes for continued service

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that are experiencing outside diameter stress corrosion cracking (ODSCC) confined within the thickness of the TSPs for Byron Unit 1, Cycle 7. Reference (1) also contained a requirement for the Commonwealth Edison Company (ComEd) to submit a schedule for evaluating the effects of the loads from a postulated loss-of-coolant accident (LOCA) in combination with a postulated safe shutdown earthquake (SSE) on certain tubes in the vicinity of the TSP wedges that react with the tube bundle load in the Byron Unit 1 SGs. In Reference (2), ComEd committed to provide that evaluation to the NRC by February 17, 1995. ComEd also stated that due to the similarities between Byron Unit 1 and Braidwood Unit 1, the evaluation would also be applicable to Braidwood Unit 1. Reference (3) transmitted the results of that evaluation.

In Reference (3), the evaluation of the effects of combined LOCA + SSE loads on certain tubes in the vicinity of the TSP wedges that react with the tube bundle load in the Byron Unit 1 and Braidwood Unit 1 SGs was contained in WCAP-14046, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 2 (Proprietary). WCAP-14046, Revision 2 modified only Section 4.7 of WCAP-14046, Revision 1. Since no other sections of WCAP-14046, Revision 1 were modified as a result of this evaluation, they were not included in WCAP-14046, Revision 2.

WCAP-14046, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 3 (Proprietary) was prepared to compile all information previously provided in WCAP-14046, Revisions 1 and 2 into one document. No new technical information is being provided. This effort was undertaken to ease in the retrievability of information previously provided.

Accordingly, please find enclosed one copy each of Westinghouse authorization letter, CAW-95-807, accompanying affidavit, Proprietary Information Notice, Copyright Notice, WCAP-14046, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 3 (Proprietary), and WCAP-14047, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 3 (Non-Proprietary).

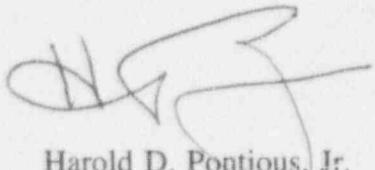
As WCAP-14046, Revision 3 contains information proprietary to Westinghouse Electric Corporation, it is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in Paragraph (b)(4) of Section 2.790 of the Commission's regulations. Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR 2.790 of the Commission's regulations.

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Correspondence with respect to the proprietary aspects of the items listed above or the supporting Westinghouse Affidavit should reference CAW-95-807 and should be addressed to N. J. Liparulo, Manager of Nuclear Safety & Regulatory Activities, Westinghouse Electric Corporation, P. O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

Please address any comments or questions regarding this matter to this office.

Very truly yours,



Harold D. Pontious, Jr.
Nuclear Licensing Administrator

- Enclosure 1: Westinghouse Letter CAW-95-807
- Enclosure 2: WCAP-14046, Revision 3 (Proprietary)
- Enclosure 3: WCAP-14047, Revision 3 (Non-Proprietary)

cc: J. B. Martin, Regional Administrator - RIII
M. D. Lynch, Senior Project Manager - NRR
R. R. Assa, Braidwood Project Manager - NRR
G. F. Dick Jr., Byron Project Manager - NRR
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