



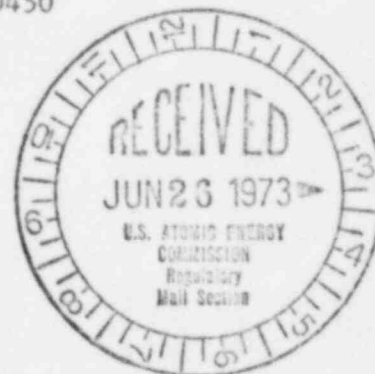
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WPW Ltr. # 67-73

Dresden Nuclear Power Station  
R. R. #1  
Morris, Illinois 60450  
June 20, 1973

Mr. A. Giambusso  
Deputy Director for Reactor Projects  
Directorate of Licensing  
U. S. Atomic Energy Commission  
Washington, D. C. 20545



SUBJECT: LICENSE DPR-25, DRESDEN NUCLEAR POWER STATION, UNIT #3,  
SECTION 6.6.C.1 OF THE TECHNICAL SPECIFICATIONS.

Dear Mr. Giambusso:

This is to report a condition relating to the operation of the unit in which, during a surveillance of the Automatic Pressure Relief System valve 203-3A failed to operate. This condition was reported to Mr. Fred Maura of Region III Regulatory Operations via telephone on May 29, 1973. *shown*

PROBLEM AND INVESTIGATION (Reference P & ID M-345)

On May 28, 1973 at 0630 hours, the unit was heating up following a maintenance outage with the mode switch in the "START" position, reactor pressure at 240 psig, and reactor water level at 30 inches. As required by Technical Specifications Section 3.5.D.1, the operability of the Automatic Pressure Relief System was being checked. When the automatic pressure relief valve 203-3A was tested, its pilot valve functioned as indicated by an "OPEN" light actuated in the control room but the main valve showed no indication of operating (i.e., no valve discharge temperature change and no surge in reactor water level.) Although the Technical Specifications Section 3.5.D.2. allows operation for thirty days with one automatic relief valve inoperable (the High Pressure Coolant Injection System operability had been demonstrated earlier in the heat-up), an orderly shutdown and cooldown was performed so that relief valve 203-3A could be repaired. The reactor was shutdown and below 90 psig at 0918 on May 28, 1973.

EVALUATIONS AND CORRECTIVE ACTIONS

The Automatic Pressure Relief System is designed such that four of the five installed relief valves will provide adequate pressure relieving capability. Therefore, loss of one valve does not materially affect the operating safety of the reactor plant. In addition, the immediate cessation

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of reactor startup in order to repair the valve provided an extra margin of safety even though it was not specifically required by the Technical Specifications.

Following reactor cooldown, the discharge elbow on relief valve 203-3A was removed so that the valve internals could be inspected. A 3/16" thick full-face asbestos gasket was found installed on the discharge side of the valve. The gasket had been used in conjunction with a blind flange and another gasket to seal the discharge of the valve earlier in the maintenance outage. Prior to start-up, the blind flange and one gasket had been removed but this gasket was inadvertently left in the valve discharge. Distortion of the gasket showed that valve 203-3A had, in actuality, operated when tested but due to the thickness of the gasket and the low reactor pressure, the valve experienced no flow.

The full-face gasket was removed and the discharge elbow reassembled. Valve 203-3A was subsequently retested and performed satisfactorily. It should be noted that since the gasket was deformed at 200 psig, sufficient pressure would have existed under operating conditions to burst the gasket and thereby allow the valve to relieve reactor pressure. Also, the other four relief valves in the system were demonstrated to be fully operational and were capable of providing 100% of design pressure relieving capacity.

Since this is the first time this type of problem has arisen, no maintenance procedure changes have been issued. However, the personnel involved have been cautioned to be more careful in the future and to ensure that maintenance on a system or component is indeed complete prior to releasing it for operation.

Sincerely,

*Fred S. Monner*  
for W. P. Worden  
Superintendent

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