

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE
37401



August 23, 1973

Mr. John F. O'Leary, Director
Directorate of Licensing
U.S. Atomic Energy Commission
Washington, DC 20545

Dear Mr. O'Leary:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 -
DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - ABNORMAL
OCCURRENCE REPORT BFAO-735W

The purpose of this report is to provide details concerning the
failure of a pressure switch at Browns Ferry Nuclear Plant on
August 13, 1973.

This occurrence was reported to the AEC Regulatory Operations
Inspector by telephone on August 13, 1973, and by telegram
on August 14, 1973.

Description of Incident

During monthly routine surveillance testing, pressure switch
PdIS-1-36A, main steamline "C" high flow, did not operate at its
technical specification setpoint of less than, or equal to,
140-percent rated steamflow which corresponds to 108.75 psid.
The as-found setpoint was 111.5 psid. The reactor was in a
cold, open-vessel condition in preparation for initial
criticality.

Investigation and Corrective Action

The pressure switch is a part of the one out of two taken twice,
initiation logic in the main steamline isolation valve isolation
system. There are four pressure switches on each steamline. The
switches are Barton Model 278 differential pressure indicating
switches with a range of 0 to 150 psid. The investigation did

9308290088 730823
PDR ADOCK 05000259
S PDR

Handwritten:
50-259
260
296

6561

COPY SENT REGION II

2

Mr. John F. O'Leary
August 23, 1973

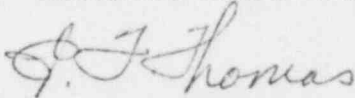
TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 -
DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - ABNORMAL
OCCURRENCE REPORT BFAO-735W

not indicate any reason that would account for the setpoint drift. The switch was within the proper accuracy when previously tested on July 17, 1973. The test standard was within its calibration interval, and a recheck confirmed its accuracy. The pressure switch was reset to 106.5 psid and operated several times to ensure repeatability.

This is not considered a violation since the reactor has never been critical and steam has not been generated from nuclear heat. This report is made to provide information data for your instrument reliability record system.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



E. F. Thomas
Director of Power Production

CC: Mr. Norman C. Moseley, Director
Region II Regulatory Operations Office, USAEC
230 Peachtree Street, NW.
Atlanta, Georgia 30303